

Progress in the Design of a Hybrid HTS-Nb₃Sn-NbTi Central Solenoid for the EU DEMO



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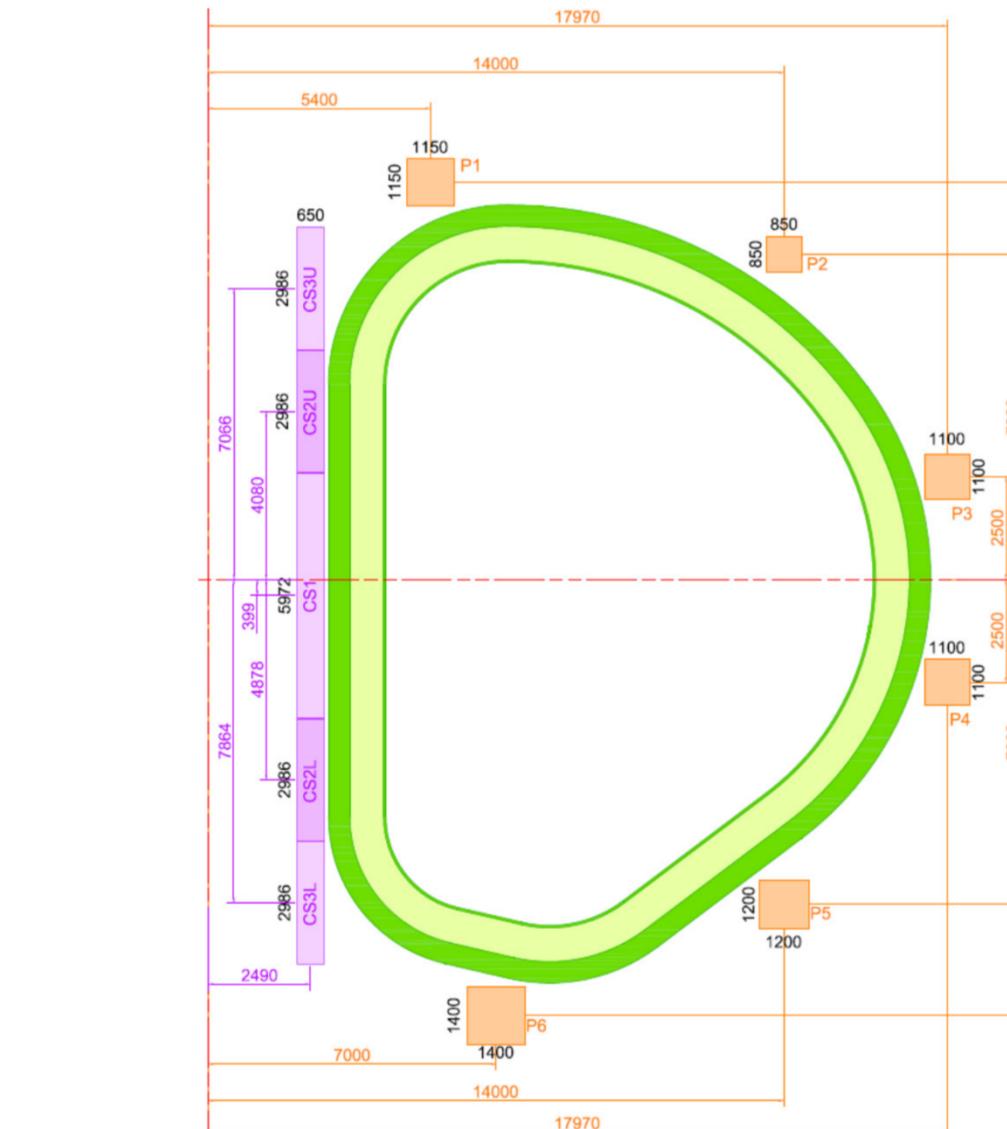
Abstract

- Benefits of a **hybrid Central Solenoid (CS)** in a pulsed tokamak:
 - Reduce the **outer radius** (i.e., reduced overall size and cost), or
 - Increase the **magnetic flux** (which can extend the plasma burn time and efficiency).
- A **hybrid CS** is proposed for the EU DEMO based on **10 layer-wound sub-coils** using **HTS**, **Nb₃Sn**, and **Nb-Ti** conductors.
- The design makes use of **superconductor** and **steel grading**.
- **Mechanical fatigue** is the **main design driver** of the EU DEMO CS.
- Alternatives to reduce the sensitivity of the proposed design to fatigue are under investigation.

Requirements and assumptions

- Geometrical and operational requirements are based on the output provided by the reactor systems code PROCESS [1].
- The present reference for magnet design is the Baseline 2018 [2].

Premag	SOF	EOF
ICS3U [MAt]	29.11	13.63
ICS2U [MAt]	29.11	-1.42
ICS1 [MAt]	58.23	-9.6
ICS2L [MAt]	29.11	-7.17
ICS3L [MAt]	28.13	25.14
IPF1 [MAt]	16.53	14.19
IPF2 [MAt]	3.86	-5.22
IPF3 [MAt]	-8.47	-5.12
IPF4 [MAt]	12.03	-3.98
IPF5 [MAt]	-9.91	-7.76
IPF6 [MAt]	24.03	15.46
Iplasma [MA]	-	10.25
	17.86	17.86

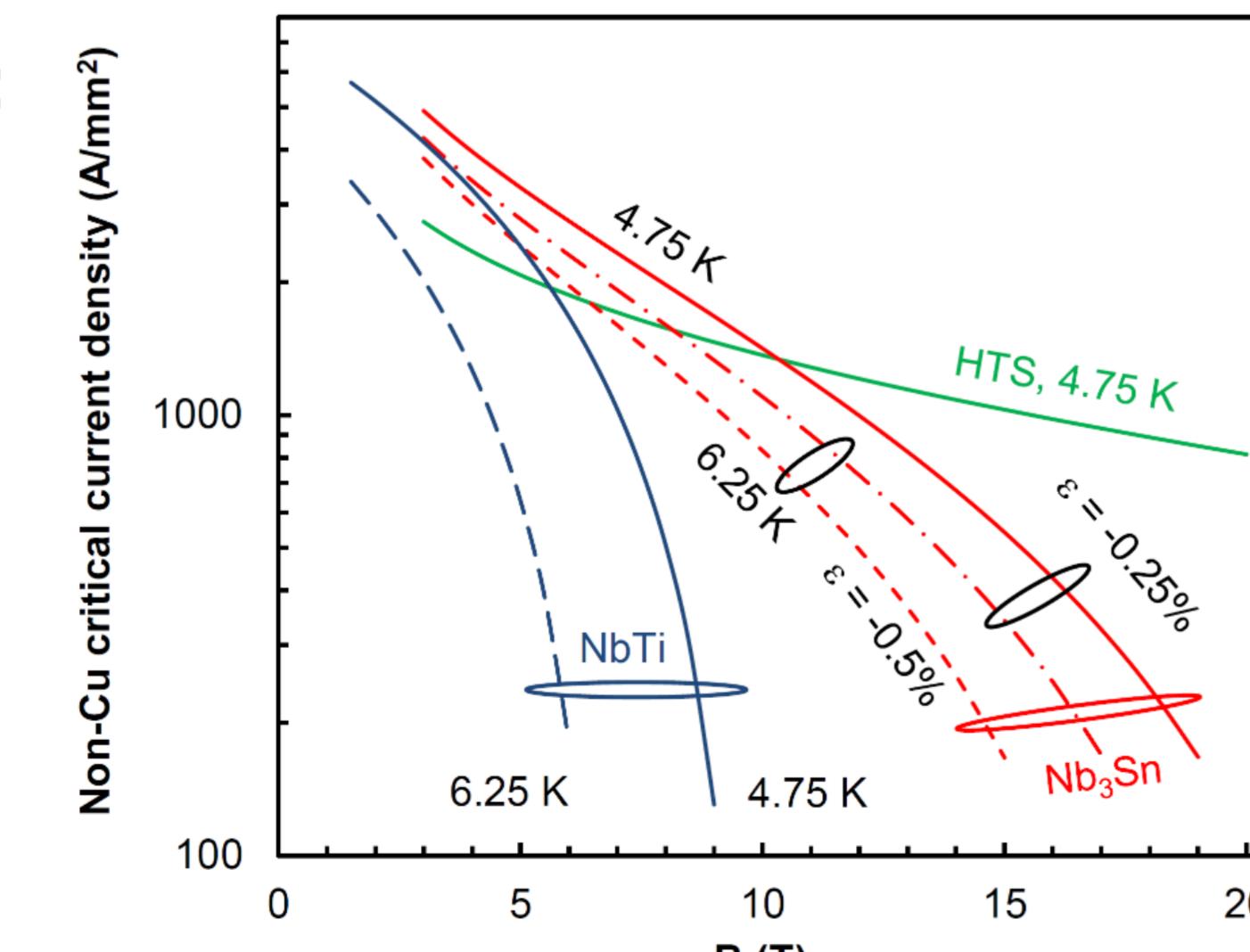


The EU DEMO is designed to operate 20,000 plasma cycles, thus the CS design has to ensure survival during **40,000 mechanical cycles**.

The studies described focus on the design of the CS1 winding pack (most mechanically demanded).

Superconducting properties:

- Nb₃Sn and Nb-Ti:
 - Scaling laws from [3]
 - $T_{op} = 4.75$ K
 - $\Delta T_m = 1.5$ K
- HTS tapes:
 - $J_{op} = 0.8 \times J_c$
 - $T_{op} = 4.75$ K

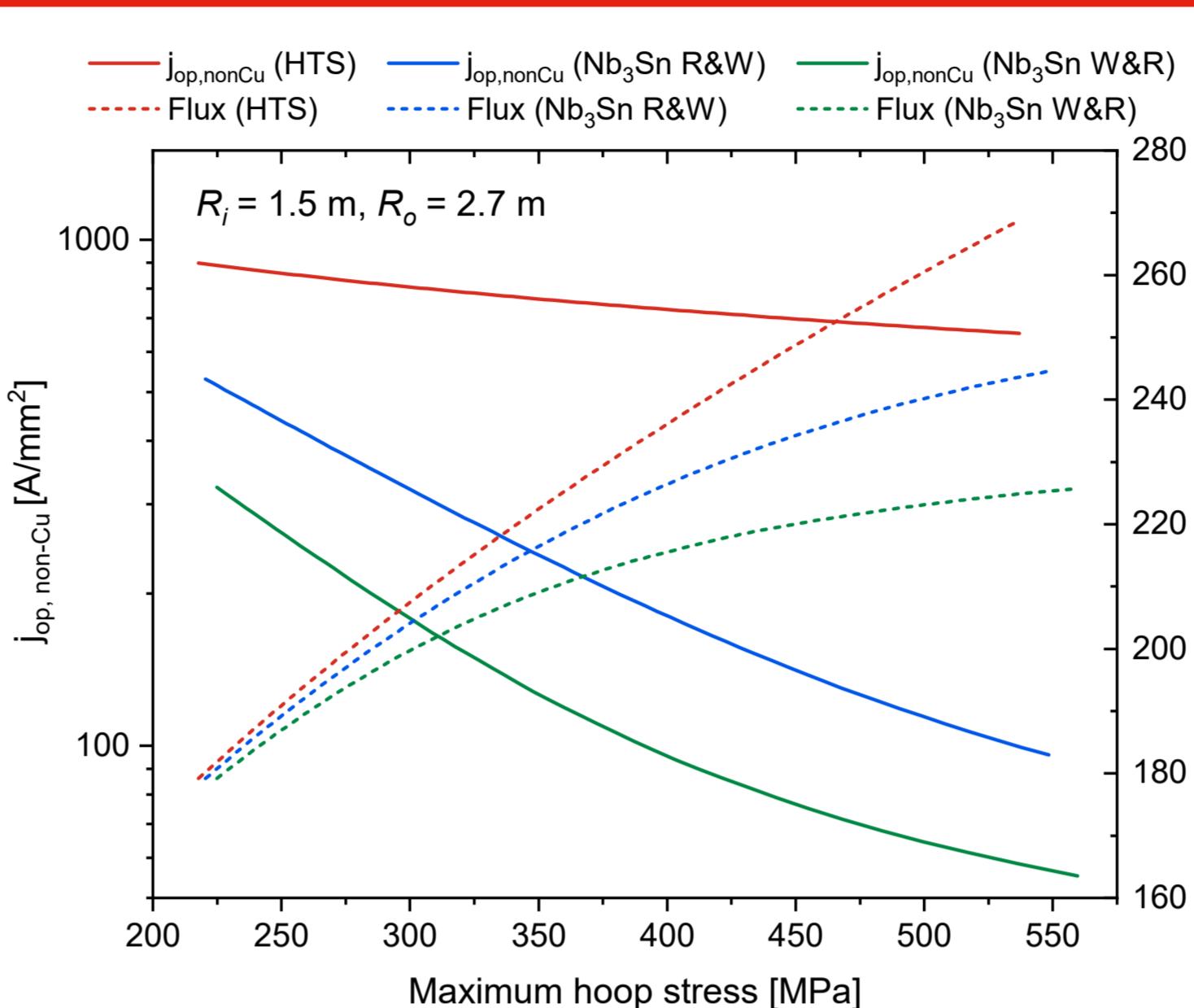


Mechanical properties:

- Static assessment: properties defined for design of DEMO WPs [4]
- Fatigue properties at 4 K taken from the ITER Material Database [5]

Methodology

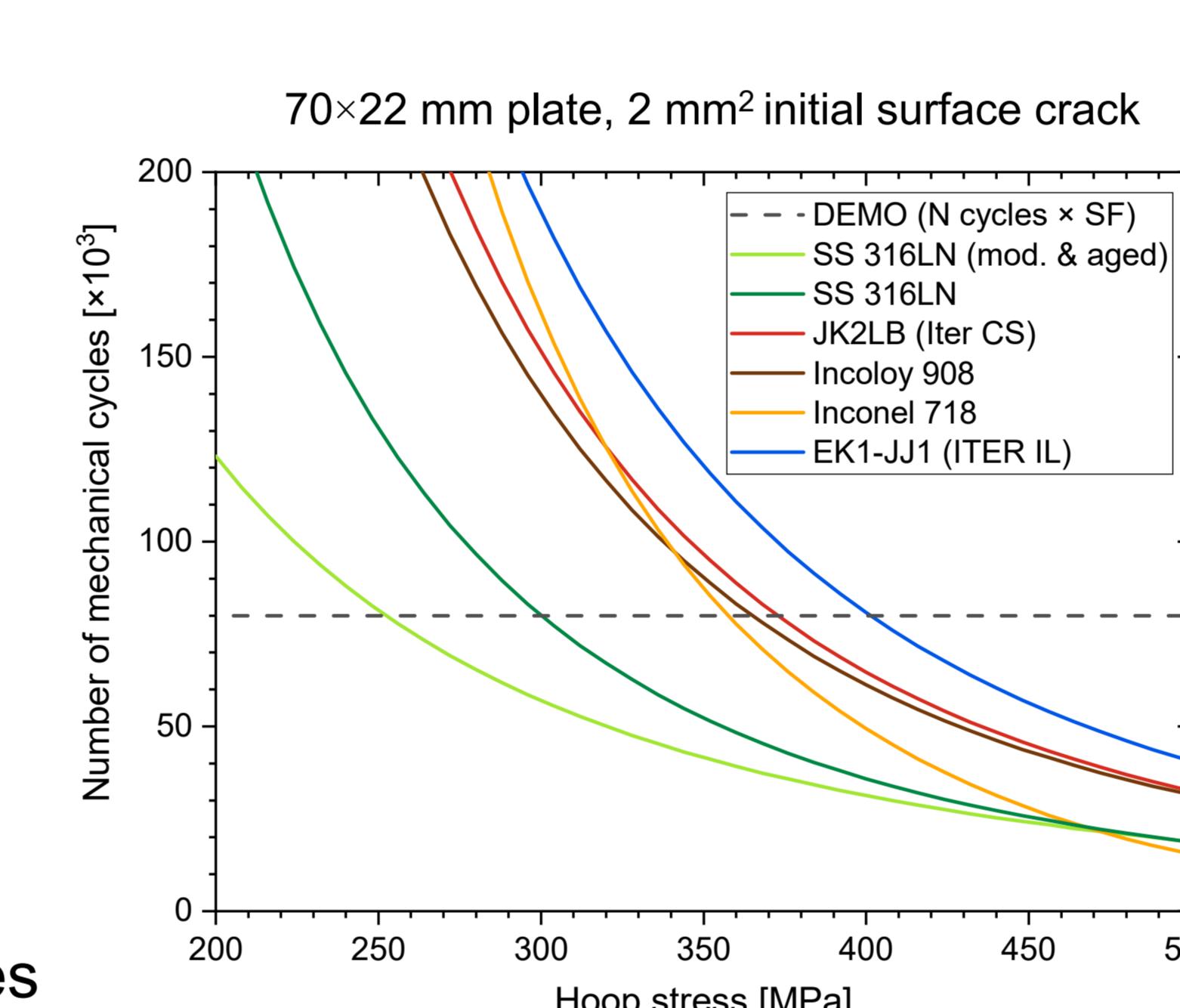
- Magnetic field, flux and hoop stress can be computed in a **uniform current density CS**.
- For a given solenoid R_i , R_o , and allowable hoop, the intersections of required and achievable $J_{op/non-Cu}$ provide maximum possible flux.



Fatigue Crack Growth Model

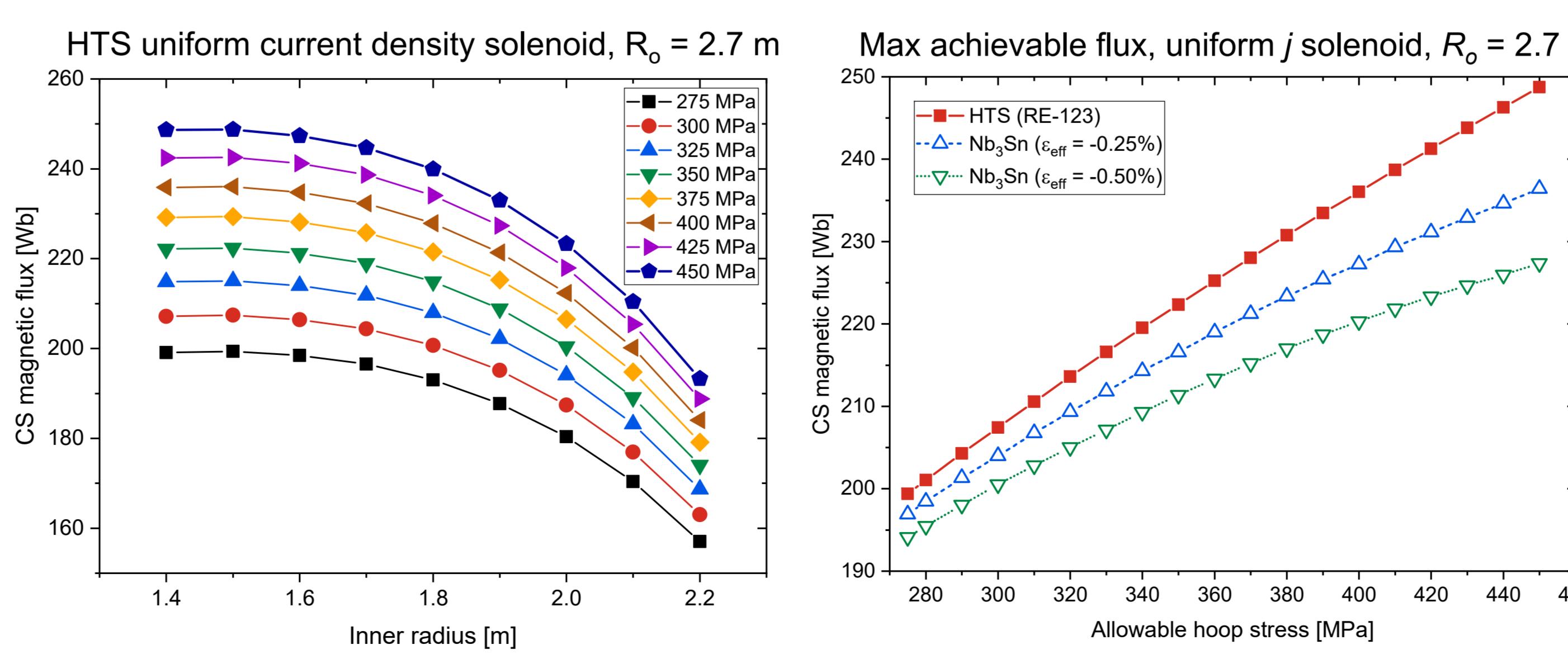
- Paris Law:

$$\frac{da}{dN} = C(\Delta K)^m$$
- Assumptions:
 - Initial defect size:
 - 2 mm² (surface)
 - 5 mm² (embedded)
 - Stress intensity factor:
 - Elliptical cracks
 - $\sigma_{residual} = 240$ MPa
 - Safety factors [5]:
 - 2x in number of cycles
 - 2x in defect area
 - 1.5x in fracture toughness



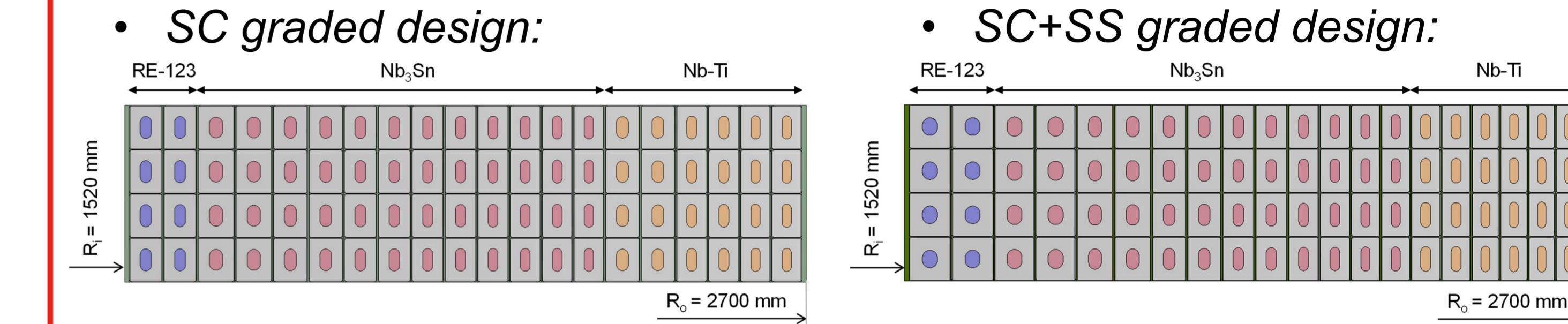
Uniform Current Density Design

- Allowable hoop defined by crack growth model.
- Higher hoop \rightarrow larger B and φ
- Conduits made of SS316LN:
 - $\sigma_{hoop} = 300$ MPa
 - $\varphi_{max} = 207.4$ Wb

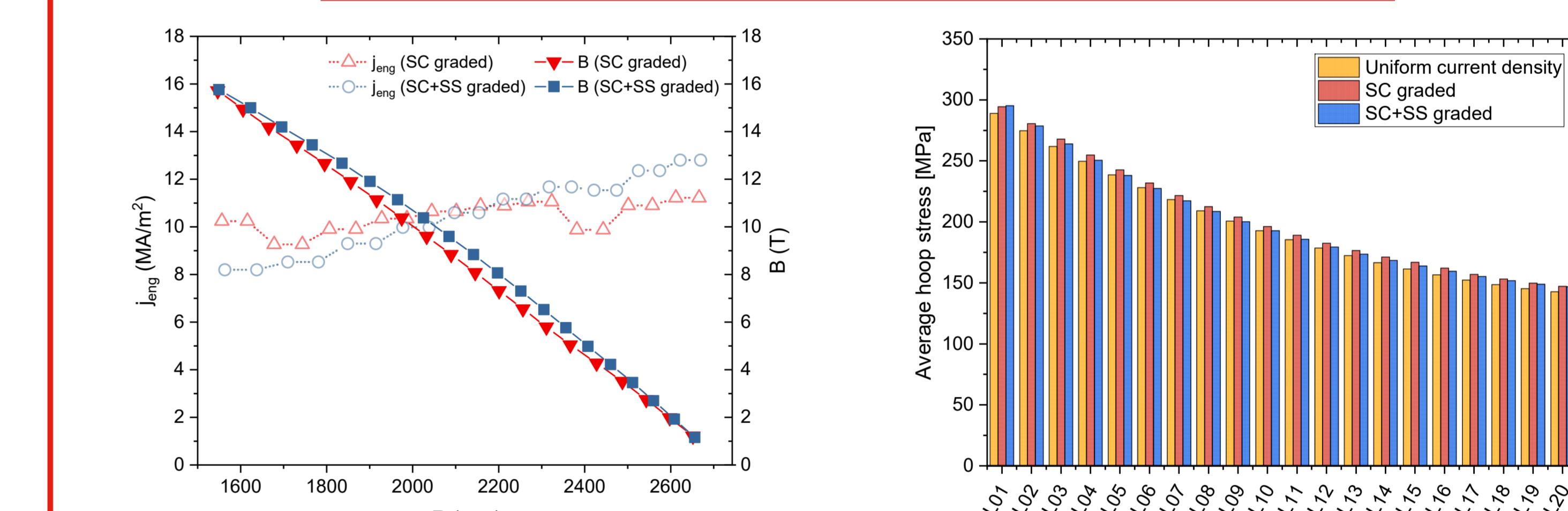


Graded Coil Designs

- A CS1 WP layout based on the use of HTS as superconductor and SS316LN for the conduits is used to explore the benefits of grading.
- **SC graded design:**



Design	UCD	SC graded	SC+SS graded
Total current [MA]	72.2	72.2	72.2
Cond current [kA]	46.3	46.3	46.3
R_i [mm]	1500	1520	1520
R_o [mm]	2700	2700	2700
SC material subcoils [HTS/Nb ₃ Sn/Nb-Ti]	10/-	1/6/3	1/6/3
Max B [T]	15.72	15.71	15.76
CS magnetic flux [Wb]	207.4	211.6 (+2.0%)	218.5 (+5.4%)
Membrane stress L01 [MPa]	356.0	362.1	350.0
Hoop stress L01 [MPa]	288.9	294.5	295.4
Cycles until break [#]	84.2×10^3	80.0×10^3	83.6×10^3



Conclusions

- In general, the use of HTS enhances the flux generated by the CS, but:
 - The relative gain depends on the allowable hoop stress (fatigue)
 - For more demanding fatigue constraints, the **effectiveness of HTS becomes smaller**
- Fatigue has profound implications in the design of a pulsed solenoid.
 - A fusion power plant based on a pulsed tokamak will experience even more demanding constraints (plasma pulses likely > 100,000)
- Grading provides a cost effective design and a modest gain in flux.

References

- [1] M. Kovari et al., Fusion Eng. Des., vol. 104, pp. 9–20, Mar. 2016.
- [2] R. Kembleton, <https://idm.euro-fusion.org/?uid=2N622S>.
- [3] V. Corato et al., EUROfusion WPMAG-Rep (16) 16565, 2016.
- [4] F. Nunio et al., <https://idm.euro-fusion.org/?uid=2MC8T4>.
- [5] ITER Structural Material Database, <https://user.iter.org/?uid=223BAC>.