

Compound nuclear reactions induced by neutrons and the R-matrix formalism

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Neutron-nucleus reactions

Reaction: • X + $a \rightarrow Y + b$ • X(a,b)Y



Cross section:

Cross section: function of the kinetic energy of the particle a $\sigma(E_a) = \int \int \frac{d^2\sigma(E_a, E_b, \Omega)}{dE_b d\Omega} dE_b d\Omega$

Differential cross section:

function of the kinetic energy of the particle a and function of the kinetic energy or the angle of the particle b

$$rac{d\sigma(E_a,E_b)}{dE_b} = rac{d\sigma(E_a,\Omega)}{d\Omega}$$

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Double differential cross section:

function of the kinetic energy of the particle a and function of the kinetic energy **and** the angle of the particle b

 $rac{d^2\sigma(E_a,E_b,\Omega)}{dE_bd\Omega}$















Decay of a nuclear state

(shell model and other states)

Statistical model

The nucleus at energies around S_n can be described by the Gaussian Orthogonal Ensemble (GOE)

The matrix elements governing the nuclear transitions are random vairables with a Gaussian distribution.

- Consequences:
 - The partial width have a Porter-Thomas distribution
 - The spacing of levels with the same J^π have approximately a Wigner distribution.

Spacing distribution of two consecutive levels

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Compound nucleus reactions

- calculate potential (optical model calculations, smooth cross section)
- use eigenstates (R-matrix, resonances)

External region: solve Schrödinger equation

central force, separate radial and angular parts.

$$\psi(r,\theta,\phi) = R(r)\Theta(\theta)\Phi(\phi)$$

solution: solve Schrödinger equation of relative motion:

- Coulomb functions
- special case of neutron particles (neutrons): **Bessel functions**

Internal region: Schrödinger equation cannot be solved directly solution: expand the wave function as a linear combination of its eigenstates. using the R-matrix:

$$R_{cc'} = \sum_{\lambda} \frac{\gamma_{\lambda c} \gamma_{\lambda c'}}{E_{\lambda} - E}$$

The R-matrix formalism

The wave function of the system is a superposition of incoming and outgoing waves:

$$\Psi = \sum_{c} y_c \mathcal{I}_c + \sum_{c'} x_{c'} \mathcal{O}'_c$$

Incoming and outgoing wavefunctions have form:

$$\mathcal{I}_c = I_c r^{-1} \varphi_c i^{\ell} Y_{m_{\ell}}^{\ell}(\theta, \phi) / \sqrt{v_c}$$
$$\mathcal{O}_c = O_c r^{-1} \varphi_c i^{\ell} Y_{m_{\ell}}^{\ell}(\theta, \phi) / \sqrt{v_c}$$

The physical interaction is included in the collision matrix **U**:

$$x_{c'} \equiv -\sum_{c} U_{c'c} y_c$$

The wave function depends on the R-matrix, which depends on the widths and levels of the eigenstates.

$$\Psi = \Psi(R_{cc'})$$
$$R_{cc'} = \sum_{\lambda} \frac{\gamma_{\lambda c} \gamma_{\lambda c'}}{E_{\lambda} - E}$$

The R-matrix formalism

The relation between the R-matrix and the collision matrix:

$$\mathbf{U} = \mathbf{\Omega} \mathbf{P}^{1/2} [\mathbf{1} - \mathbf{R} (\mathbf{L} - \mathbf{B})]^{-1} [\mathbf{1} - \mathbf{R} (\mathbf{L}^* - \mathbf{B})] \mathbf{P}^{-1/2} \mathbf{\Omega}$$

with: $L_c = S_c + iP_c = \left(\frac{\rho}{O_c} \frac{dO_c}{d\rho}\right)_{r=a_c}$

The relation between the collision matrix and cross sections:

channel to one other channel: $\sigma_{cc'} = \pi \lambda_c^2 |\delta_{c'c} - U_{c'c}|^2$

channel to any other channel:

$$\sigma_{cr} = \pi \lambda_c^2 (1 - |U_{cc}|^2)$$

channel to same channel:

$$\sigma_{cc} = \pi \lambda_c^2 |1 - U_{cc}|^2$$

channel to any channel (total):

$$\sigma_{c,T} = \sigma_c = 2\pi\lambda_c^2(1 - \operatorname{Re} U_{cc})$$

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The R-matrix formalism

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The Breit-Wigner Single Level approximation: total cross section:

$$\sigma_c = \pi \lambda_c^2 g_c \left(4 \sin^2 \phi_c + \frac{\Gamma_\lambda \Gamma_{\lambda c} \cos 2\phi_c + 2(E - E_\lambda - \Delta_\lambda) \Gamma_{\lambda c} \sin 2\phi_c}{(E - E_\lambda - \Delta_\lambda)^2 + \Gamma_\lambda^2/4} \right)$$

neutron channel:
$$c = n$$

only capture, scattering, fission: $\Gamma_{\lambda} = \Gamma = \Gamma_n + \Gamma_{\gamma} + \Gamma_f$
other approximations: $\ell = 0$ $\cos \phi_c = 1$ $\sin \phi_c = \rho = ka_c$ $\Delta_{\lambda} = 0$

total cross section:

$$\sigma_T(E) = 4\pi R'^2 + \pi \lambda^2 g \left(\frac{4\Gamma_n(E - E_0)R'/\lambda + \Gamma_n^2 + \Gamma_n\Gamma_\gamma + \Gamma_n\Gamma_f}{(E - E_0)^2 + (\Gamma_n + \Gamma_\gamma + \Gamma_f +)^2/4} \right)$$

total width

The R-matrix formalism

The Reich-Moore approximation:

Use the fact that there are many photon channels, with Gaussian distributed amplitudes with zero mean:

$$<\gamma_{\lambda c}\gamma_{\mu c}>=\gamma_{\lambda c}^2\delta_{\lambda\mu}$$

The sum over the amplitudes of the photon channels becomes then:

$$\sum_{c \in \text{photon}} \gamma_{\lambda c} \gamma_{\mu c} = \sum_{c \in \text{photon}} \gamma_{\lambda c}^2 \delta_{\lambda \mu} = \Gamma_{\lambda \gamma} \delta_{\lambda \mu}$$

Then photon channels can be eliminated in the R-matrix:

$$R_{cc'} = \sum_{\lambda} \frac{\gamma_{\lambda c} \gamma_{\lambda c'}}{E_{\lambda} - E - i\Gamma_{\lambda \gamma}/2} \qquad c \notin \text{photon}$$

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The resolution can be expressed equivalenty in time, distance and energy:

$$R_t(\delta t)d\delta t = R_L(\delta L)d\delta L = R_E(\delta E)d\delta E$$

Comparing neutron time-of-flight facilities

facility characteristics

- instantaneous flux (neutrons per pulse)
- average flux (neutrons per second)
- resolution (resonance shape analysis)
- background

facility equipment

- detectors
- samples
- data acquisition

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Comparing some neutron time-of-flight facilities

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Facility	Location	Particle	Beam energy (MeV)	Neutron target	Pulse width (ns)	Beam power (kW)	Pulse frequency (Hz)	Flight path lengths (m)	Neutron production (n/pulse)
RPI	RPI, Troy, USA	e-	60	Та	5	0.6	500	15–250	3.6 × 10 ⁹
		e-	60	Та	5,000	>10	300	15, 25	4.8×10^{11}
ORELA	ORNL, Oak Ridge, USA	e-	180	Та	2–30	60	12–1,000	9–200	1 × 10 ¹²
GELINA	EC-JRC-IRMM, Geel, Belgium	e-	100	U	1	10	40-800	5–400	4.3 × 10 ¹⁰
nELBE	FZD, Rossendorf, Germany	e-	40	L-Pb	0.01	40	500,000	4	5.4 × 10 ⁷
IREN	JINR, Dubna, Russia	e-	30	W	100	0.42	50	10–750	7.7 × 10 ¹⁰
PNF	PAL, Pohang, Korea	e-	75	Та	2,000	0.09	12	11	1.7×10^{10}
KURRI	Kumatori Japan	e-	46	Та	2	0.046	300	10, 13, 24	2 × 10 ⁹
		e-	30	Та	4,000	6	100	10, 13, 24	8 × 10 ¹⁰
LANSCE-MLNSC	LANL, Los Alamos, USA	р	800	W	135	80	20	7–60	7×10^{14}
LANSCE-WNR	LANL, Los Alamos, USA	р	800	W	0.2	1.44	13,900	8–90	8 × 10 ⁹
n_TOF	CERN, Geneva, Switzerland	р	20,000	Pb	6	10	0.4	185	2 × 10 ¹⁵
MLF-NNRI	J-PARC, Tokai, Japan	р	3,000	Hg	1,000	1,000	25	30	1.2×10^{17}

From: D. G. Cacuci (ed.), Handbook of Nuclear Engineering, R. C. Block, Y. Danon, F. Gunsing, R. C. Haight Chapter: Neutron Cross Section Measurements

Thank you for your attention.