# The SPARC Toroidal Field Model Coil Project

Zach Hartwig Associate Professor | MIT PI and Head | TFMC Project

> CERN Seminar July 27 2022

PS

# All credit to the exceptional team that delivered the TFMC Project

Commonwealth Fusion Systems Fusion Center

### All credit to the exceptional team that delivered the TFMC Project

Chief Engineer: Rui Vieira Group leaders: Brian LaBombard (E&M), Chris Lammi (Analysis), Joy Dunn (Manufacturing), Ted Golfinopoulos and Phil Michael (Test)

Team:

Sue Agabian **Dave Arsenault** Raheem Barnett **Mike Barry Bill Beck Dave Bellofatto** Willie Burke Jason Burrows Bill Byford **Charlie Cauley** Sarah Chamberlain David Chavarria Jessica Cheng Jim Chicarello Karen Cote **Corinne** Cotta Mary Davenport

Van Diep Eric Dombrowski Jeff Doody Raouf Doos **Brian Eberlin** Jose Estrada Vinny Fry Matt Fulton Sarah Garberg **Bob** Granetz Aliya Greenberg Sam Heller Amanda Hubbard Ernie Ihloff Jim Irby Mark Iverson **Peter Jardin** 

Sergey Kuznetsov **Rich Landry** Ed Lamere **Rick Lations Rick Leccacorvi** Matt Levine George MacKay Kristen Metcalfe Phil Michael Kevin Moazeni **Bob Mumgaard** John Mota Theodore Mouratidis JP Muncks **Rick Murray** Tesha Myers Dan Nash

Ben Nottingham Andy Pfeiffer Sam Pierson **Clayton Purdy** Alexi Radovinsky **DJ** Ravikumar Veronica Reyes Ron Rosati Mike Rowell **Dior Sattarov** Wayne Saunders **Pat Schweiger** Shane Schweiger **Maise Shepard** Syunichi Shiraiwa Maria Silveira **Brandon Sorbom** 

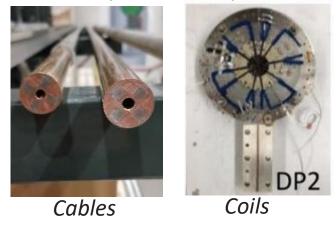
Pete Stahle Ken Stevens Joe Stiebler Deepthi Tammana Tom Toland Dave Tracey **Ronnie Turcotte** Kiran Uppalapati Matt Vernacchia **Chris Vidal** Alex Warner **Amy Watterson** Dennis Whyte Sidney Wilcox Michael Wolf \*\* **Bruce Wood** Lihua Zhou Alex Zhukovsky

### All work funded by Commonwealth Fusion Systems

# What was the SPARC Toroidal Field Model (TFMC) Coil Project?



1. Developed REBCO conductor technologies (2017-2019)



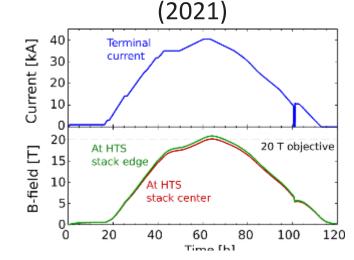
3. Built and commissioned the test facility (2020-2021)



2. Designed and built the TF model coil (2019-2021)



4. Achieved 20 tesla full performance test



Completed in ~4 years by MIT and CFS in partnership with our vendors

### MIT The TFMC Project accelerated high-field SC magnets for fusion **PSFC** Phase 3: **High-field** Phase 2: Phase 1: **Demonstration Commercialization** fusion science Technology R&D **Alcator C-Mod** REBCO CIC concepts **SPARC** Q>2 P > 100 MW ARC Q>10 *No-insulation REBCO concepts* P > 200 MWe 2016 2021 **2030s** 2025

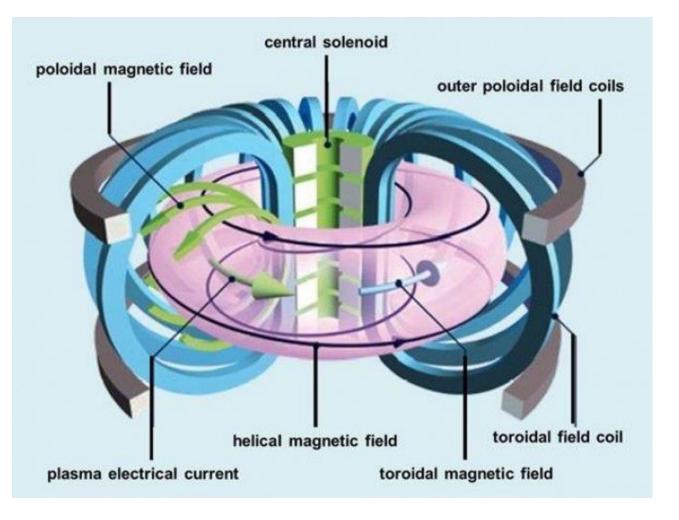
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#### MIT The TFMC Project accelerated high-field SC magnets for fusion **PSFC** Phase 3: **High-field** Phase 2: Phase 1: **Commercialization** fusion science **Demonstration** Technology R&D Alcator **C-Mod** REBCO CIC condepts **SPARC** SPARC Q>2 magnet R&D HTS model coil (2017-2019) (2019-2021) P > 100 MW ARC Q>10 No-insulation REBCO concepts The TFMC Project P > 200 MWe 2016 2021 **2030s** 2025

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# Tokamaks are fusion devices with 3 main magnet systems





**Toroidal Field (TF) magnet:** Creates the primary magnetic field that provides plasma confinement

### **Central Solenoid (CS) magnet:** Creates a poloidal magnetic field that gives the toroidal field a "twist" by driving current inductively in the toroidal direction

**Poloidal Field (PF) magnets:** Creates Magnetic fields that position, shape, stabilize, and divert the plasma

# The TF magnet is a steady-state, high-field magnet

### MIT PSFC

### What is it:

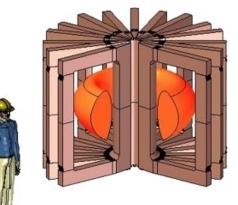
- A toroidally-shaped solenoid electromagnet that is composed of many individual coils
- Coils are typically copper magnets (resistive) but increasingly superconducting (non-resistive)

### **Purpose:**

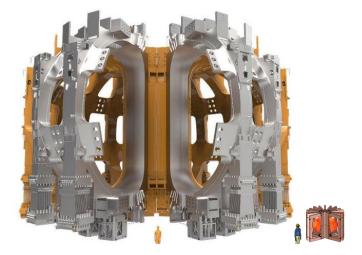
• Create the primary toroidal magnetic field that provides the main plasma confinement

### Challenges:

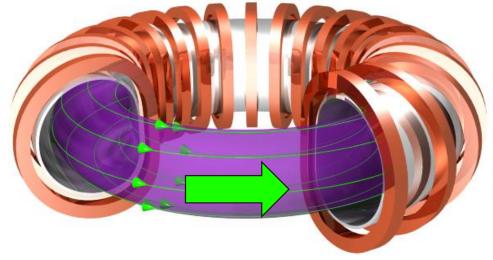
- Carry sufficient current to achieve required field
- Robust to massive electromechanical forces
- Ensure sufficient cooling access to coils
- Low toroidal field ripple to prevent particle loss
- Robust to quench (detection and protection)
- Support/translate superstructure loads



Alcator C-Mod's copper TF magnet



### ITER's Nb3Sn superconducting TF magnet



History lesson: Fusion has always maximized the toroidal field! Fusion performance metrics go like ~B<sup>4</sup>

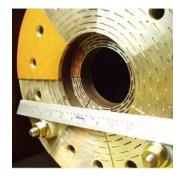


**1950-1960s:** Copper wire The pioneers



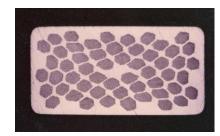
### 1960-1980s:

Cryogenic Bitter plates The Alcators at MIT



### 1980-2000s:

NbTi superconductors First SC fusion devices



**1990s-2010s:** Nb<sub>3</sub>Sn for higher field Reactor-class devices



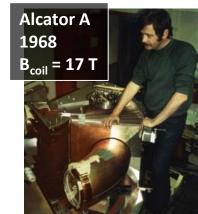
### 2010-2020s:

REBCO: very high magnetic fields



REBCO: Rare Earth Barium Copper Oxide









### Outline

- Early REBCO conductor/coil R&D
- TFMC magnet and test facility
- 20 T TFMC test program
- (If time/interest: Other REBCO R&D at MIT)

\*\* All the work presented here will be presented in more detail in upcoming publications and conference (e.g. ASC2022)

# Early REBCO conductor/coil R&D: 2017-2019

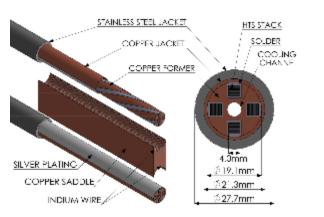
# Early R&D resulted in two viable high-field REBCO technologies

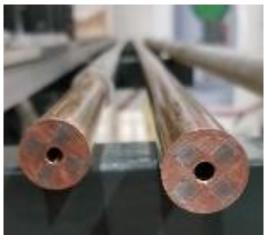


Starting in 2017, both technologies were built on REBCO magnet work pioneered at MIT

### VIPER HTS cables [1]

- Based on traditional SC CIC cables using the TSTC architecture for REBCO tapes [2]
- Developed for *multiple* SPARC applications:
  - High current feeder cables
  - AC magnets: SPARC CS, PF (w/ modifications)
  - DC magnets: SPARC TF magnet

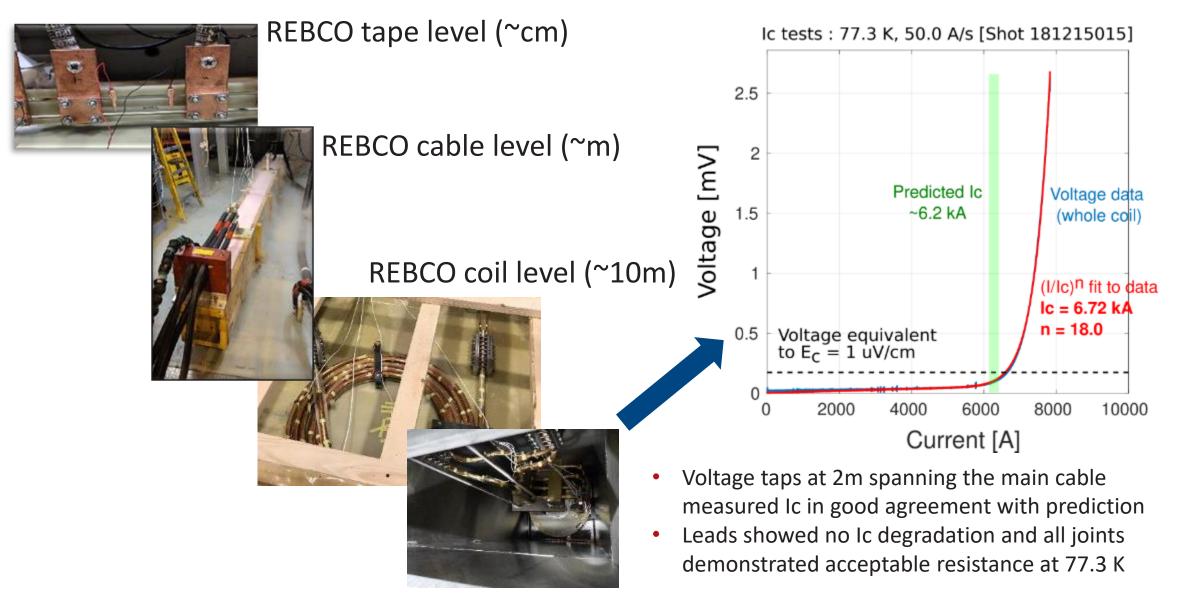




# [1] Z. Hartwig *et al.*, SuST, **33** (2021) 11LT01 [2] M. Takayasu *et al.*, IEEE TAS, **21** (2011) 2340

# VIPER cable: Small-to-medium scale 77 K testing at MIT

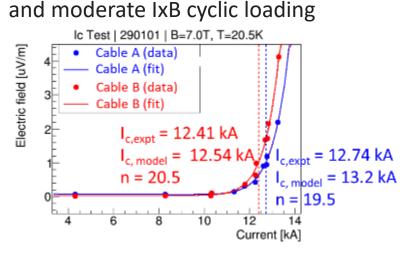




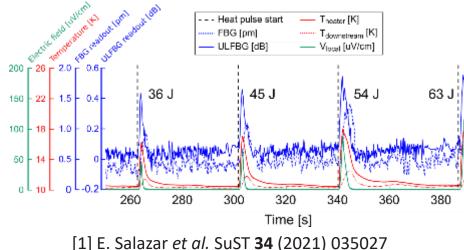
# VIPER cable: 4 SULTAN tests in 4 months w/ 4 objectives



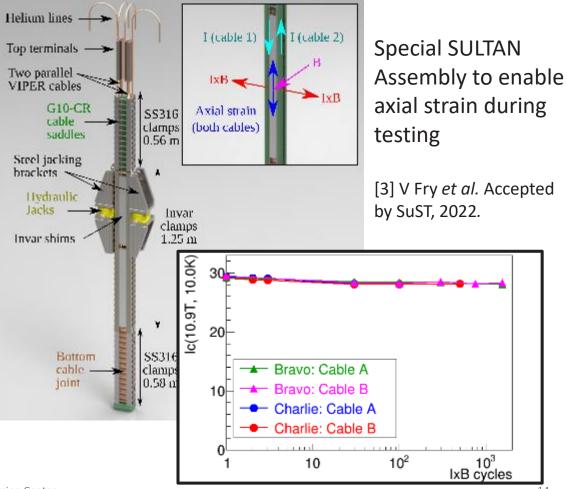
Alpha: Qualification, model validation



**Delta**: Quench detection (voltage, fiber optic (x4), acoustic (passive, active) and dynamics



Bravo: High IxB cyclic loading at 382 kN/m for 1500 cycles)
Charlie: High IxB cyclic loading with ~0.3% axial strain in REBCO



# Early R&D resulted in two viable high-field HTS technologies



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Starting in 2017, both technologies were built on HTS magnet work pioneered at MIT

**VIPER HTS cables** 

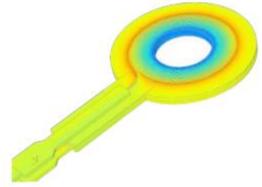
• Based on traditional SC CICC cables using the TSTC architecture for REBCO tapes [1]

Developed for *multiple* SPARC applications:

- High current feeder cables
- AC magnets: SPARC CS, PF (w/ modifications)
- DC magnet: SPARC TF magnet

### No Insulation No Twist (NINT) Coils

- HTS cable-based adaptation of single tape NI coils [2] with innovations to enable large-scale fusion magnets
- Developed for a *specific* SPARC application:
  - DC magnet: SPARC TF magnet





EM simulation of a NINT coil

# Decision was made to pursue a full-scale no-insulation coil test



VIPER would be the platform for AC coils (central solenoid, poloidal field coils) and backup for DC coils (toroidal field) in the SPARC tokamak



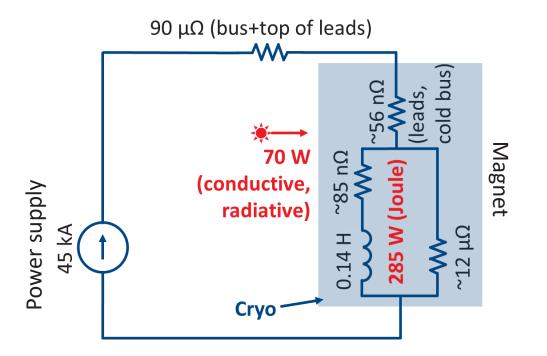


NI coils would be the platform for the DC coil (toroidal field magnet) in the SPARC tokamak

A comprehensive magnet test program was needed to prove out the potential advantages of NI coil technology at the scale and performance required for fusion-relevance

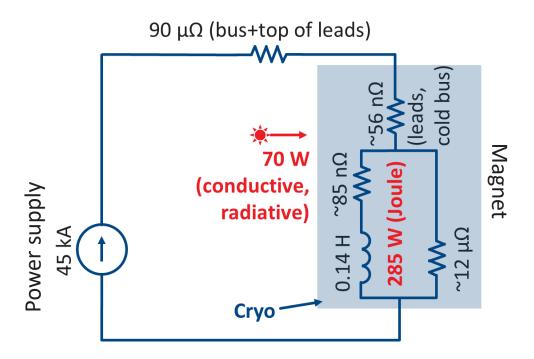
Propose Design Features	Advantage(s) to be proven in the TFMC Project
Modular, simple construction	Rapid assembly; Maintenance options; scalable for commercial production
Intrinsically low voltage (<1 V)	Minimal insulation; simple fabrication, low voltage leads and feeds, safety
Pressure vessel cooling approach	Enhanced heat removal; Local cooling optimization; simplified manifolding
High winding pack current density	Compact magnet; expanded design space
High thermal stability	Robust to damage, defects, and off-normal events
Resiliency to quench	No quench detection systems, no active mitigation systems

# I'll attempt large-scale NI coil physics in 1 slide due to time PSFC

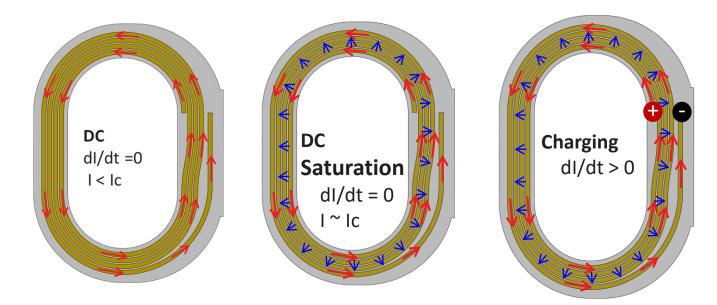


- Magnet and circuit can be thought of as a simple "lumped element" model
- No turn-to-turn insulation! Non-REBCO materials are effectively insulators under DC conditions
- Current can take one of two paths depending on the state (DC or charging) and performance
  - Azimuthal: REBCO with small R, large L
  - Radial: Turn-to-turn steel/copper large R

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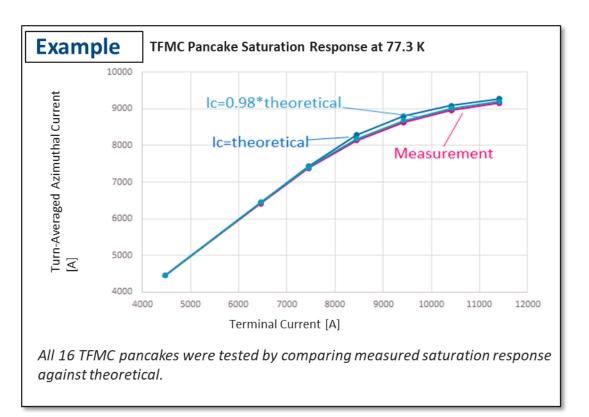


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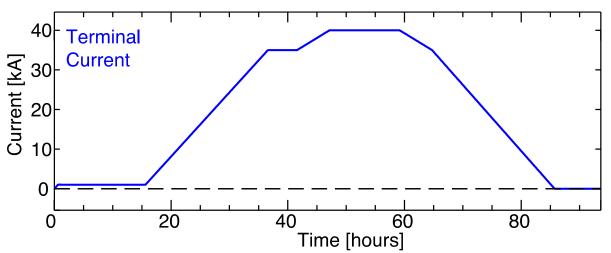


- **DC (I<Ic):** Current flows entirely azimuthally with
- **DC Saturation (I~Ic):** Current starts to shunt in the radial direction due REBCO transition voltage. Heating in the magnet occurs, lowering the cryostability
- **Charging/Discharging:** Current flows strongly in the radial direction and converts to azimuthal direction with L/R time. Heating proportional to ramp rate!

# Some consequences of NI coil physics



- Terminal current vs. magnet field is not linear! The coil saturates as I~Ic approached
- Result is voltage (e.g. power) and heating in the winding pack that can threaten cryostability



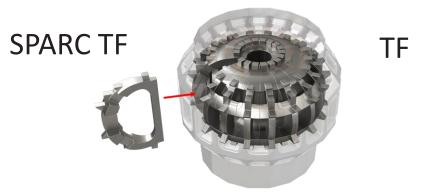
- Charging/discharging rates are limited by cooling system and heat removal efficiency, resulting in long charging times
- The TFMC had L/R ~ 3.5 h, resulting in long experimental campaigns and limits on testing (e.g. cycling)

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**PSFC** 

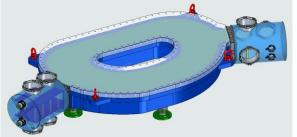
# TFMC magnet and test facility: 2019-2021

# TFMC requirements were set by the SPARC TF coil design PSFC



- □12.2 T magnetic field on-axis (22T on HTS tape)
- Acceptable toroidal field ripple; azimuthal current deficit less than 0.5% in every coil; design value of lop/lc attained
- HTS conductor capable of handing Lorentz loadsAcceptable joint resistances
- □Acceptable coil dissipation
- Electrical performance as projected by EM models
- Cryogenic coolant design capable of handling SPARC neutron loads (pressure vessel design)
- Employ modular, grooved stacked-plate pancake construction to facilitate rapid manufacturing





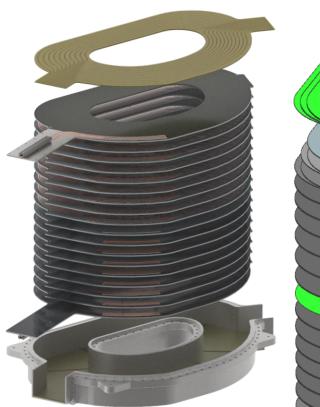
- ✓ Demonstrate 20T on HTS tape in large-bore D-shaped coil at > 50% linear size of TF
- ✓ Demonstrate accurate projections of HTS tape performance
- ✓ Demonstrate HTS conductor Lorenz loading at TF levels (> 700 kN-m)
- ✓ Demonstrate viable joint design and performance
- ✓ Demonstrate / validate accurate EM models
- ✓ Employ internal coil winding architecture that is essentially identical to that employed by TF
- ✓ Construct coil within 2 year time frame, using manufacturing methods that transfer to SPARC TF

# The TFMC's requirements were set by the SPARC TF magnet



# TFMC

- Modular, stacked-plate construction
- 256 turns total
- 16 full pancakes (16 turns each)
- HTS tape stacks, soldered into grooves
- Mirrored winding pack
- Integral pc-to-pc joints
- Pancake-to-pancake insulation
- Integrated coolant channels
- Case is supercritical helium
   pressure vessel



SPARC TF – V2B
<ul> <li>Modular, stacked-plate construction</li> <li>200 turns total</li> <li>12 full pancakes (16 turns)</li> <li>2 partial pancakes (10 turns)</li> <li>2 partial pancakes (6 turns)</li> <li>HTS tape stacks, soldered into grooves</li> <li>Mirrored winding pack</li> <li>Integral pc-to-pc joints</li> <li>Pancake-to-pancake insulation</li> <li>Integrated coolant channels</li> <li>Case is supercritical helium pressure vessel</li> </ul>

### Same winding pack architecture

# Winding PackR-widthZ-heightTFMC1.6 m2.6 mSPARC3.0 m4.3 m

The TFMC Project eliminated risk in the physics, fabrication, and operation of large-scale REBCO (NI) DC magnets

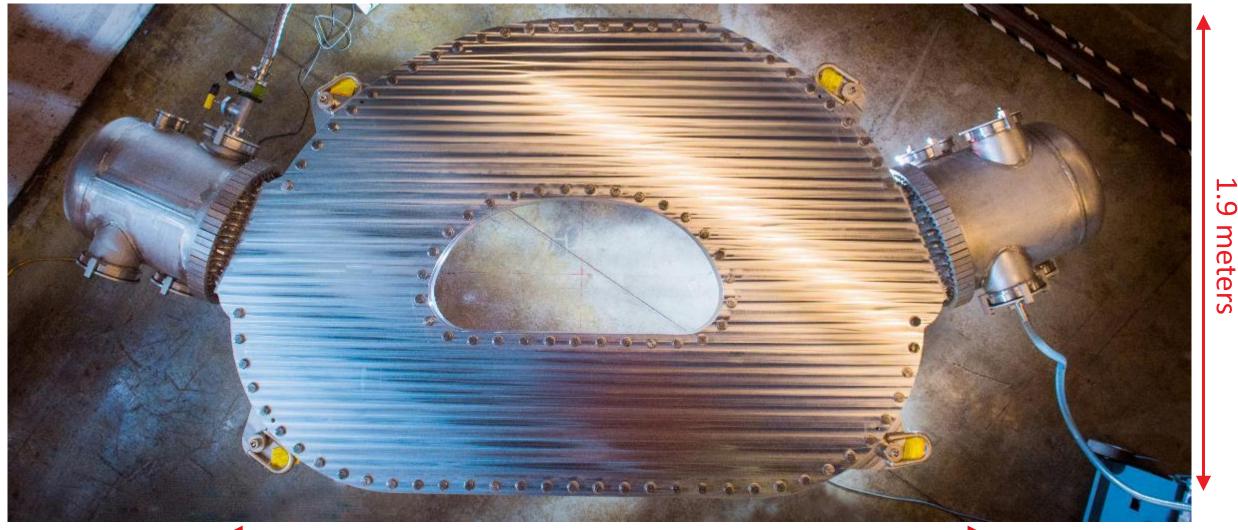


RISK		
Design	Achieve SPARC requirements (B <sub>peak</sub> , J <sub>wp</sub> , P <sub>cooling</sub> , etc); EM modeling tools;	
Supply Chain	HTS supply and characterization; Structural materials; large-scale vendors;	
Fabrication	Tooling; Manufacturing process and equipment; Process control, Scalability,	
High field	Structural loading; IxB and strain on HTS; I <sub>c</sub> limits on HTS;	
Operation	Current leads; Feeder cables; Instrumentation; Cooling system;	
Quench	Stored magnetic energy; high current; high pressure coolant;	
	<b>Objective:</b> Design, build, and test (1) a representative SPARC TF model coil and (2) a fully capable test facility in 2 years to maximize risk retirement for SPARC	

Team Size: 80+ people at 3 sites

# The TFMC is the first large-scale high-field REBCO magnet

Targeting peak fields >20 T with simple manufacturing, novel cooling, passive quench handling





# The TFMC at a glance









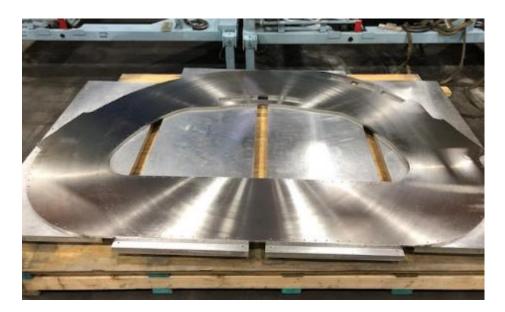
Nominal Design Parameter	Value
Number of pancakes	16
Total turns	256
Total REBCO tape	270 km
Operating temperature	20 K
Coolant type	Supercrit. He
Operating coolant pressure	20 bar (max)
Operating terminal current	40 kA
Peak magnetic field	20 T
Peak IxB force on REBCO	800 kN/m
Inductance	0.14 H
Magnetic stored energy	110 MJ
WP mass	5,113 kg
WP current density	153 A/mm <sup>2</sup>
WP + case mass	10,058 kg
WP + case linear size	2.9 x 1.9 m

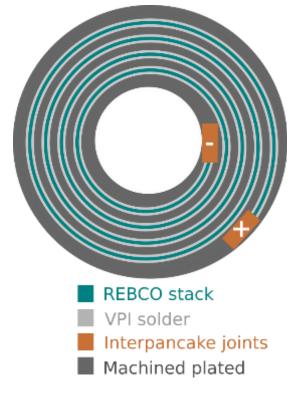
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# TFMC employs an HTS stack-in-plate design for pancakes



- Pancakes
  - A large steel plate is machined on both sides:
    - One side contains channels for HTS stacks
    - The other side contains pancake-specific cooling channels
  - The HTS is stack wound into the grooves
  - The HTS stack is terminated at internal pancake-to-pancake joints
  - VPI solder process bonds mechanically, electrically, and thermally

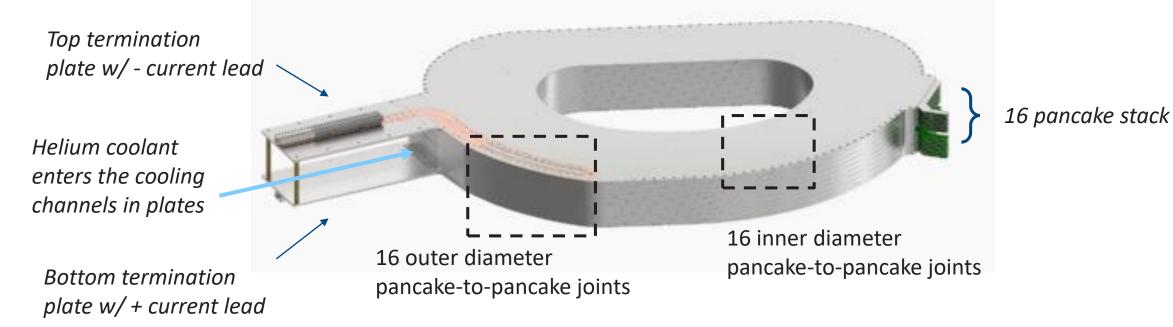




#### © MIT Plasma Science and Fusion Center

### 16 pancakes are combined into the winding pack

- Winding pack
  - The winding pack comprises 16 stacked, internally jointed pancakes
  - Winding pack has 2 top and bottom termination plates for current leads



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# The winding pack is enclosed in a "bucket-and-lid" style case

- Magnet
  - The winding pack is encased in a steel structural and pressure vessel case
  - High pressure plena enable current, cooling, and instrumentation





### A new magnet test facility was established at MIT



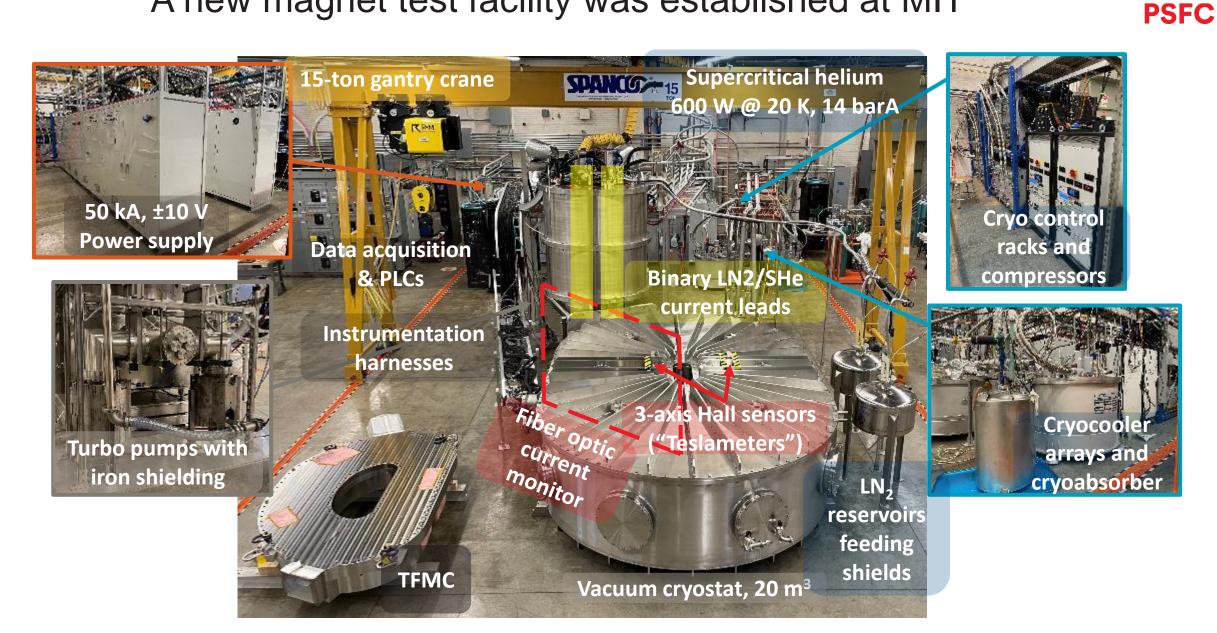
The test facility was designed, built, and commissioned at MIT

The TFMC was designed, fabricated (w/ vendors), and assembled at MIT

REBCO procurement + QA/QC and some pancake winding at CFS ΜΙΤ

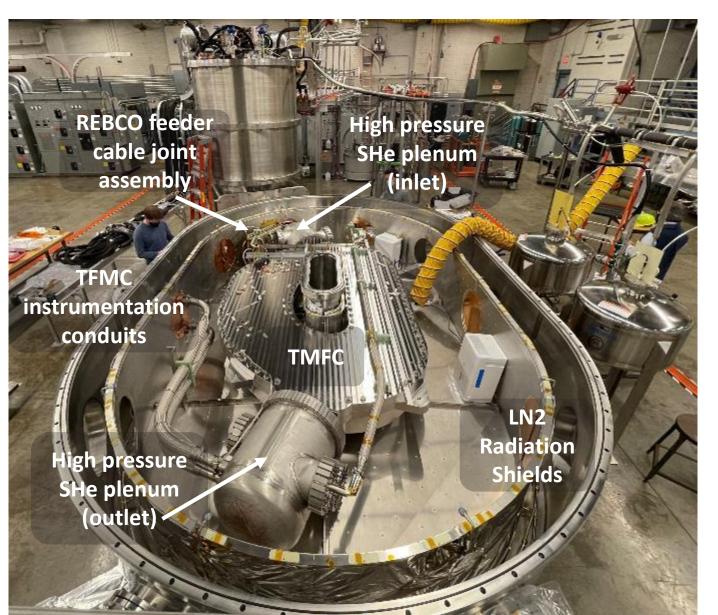
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### A new magnet test facility was established at MIT



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### A view of the TFMC installed within the main cryostat



MIT

**PSFC** 

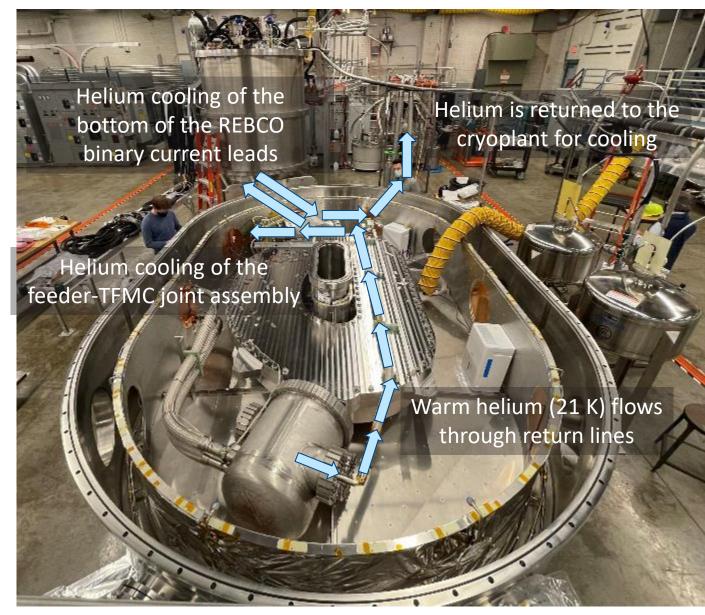
### The case functions as a cryogenic pressure vessel for cooling

MIT PSFC

Cold helium (20 K) from the supply enters the inlet plena Helium enters the cooling channels machined into the 16 pancakes making up the winding pack, cooling both the winding pack and case Warm helium (21 K) leaves the winding pack and is recombined in the outlet plena

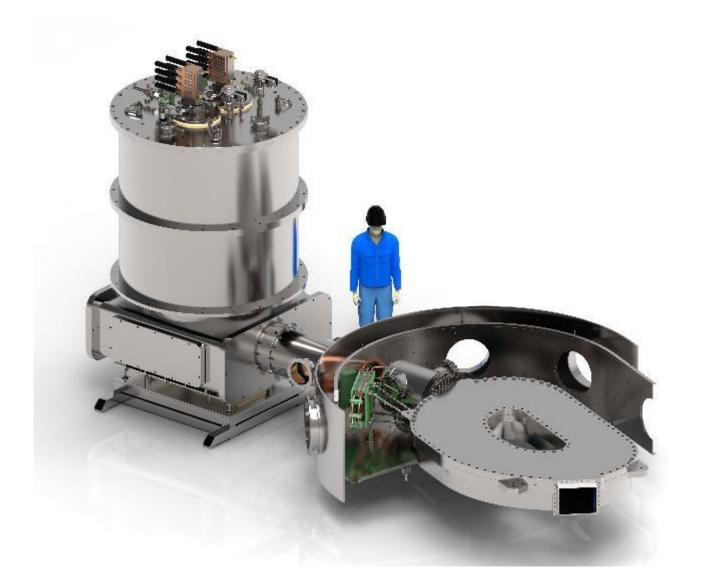
### Helium exhaust from the magnet cools the leads and feeders





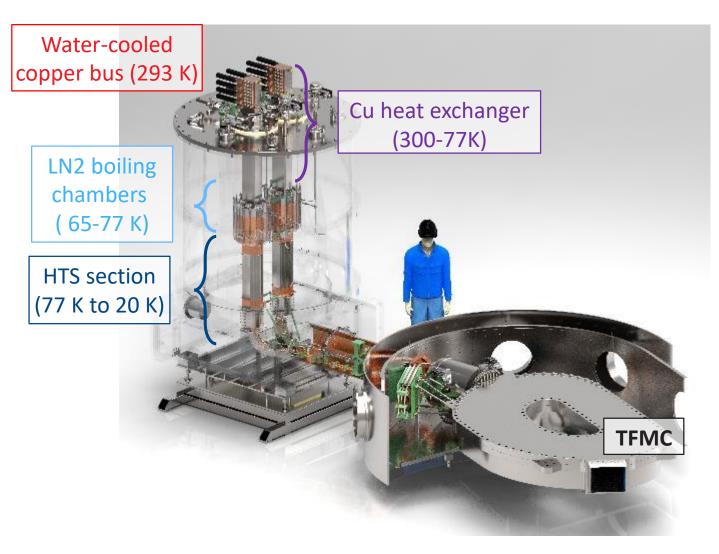
# 50 kA binary REBCO current leads and feeder system developed





# Binary 50 kA HTS current leads and feeder system proven

- REBCO current leads were designed, fabricated, and commissioned in-house
  - Designed to supply up to 50 kA for low voltage DC magnets
  - LN<sub>2</sub> (sub-cooled if desired) and SHe cooled
  - In-house development required to meet performance and schedule requirements



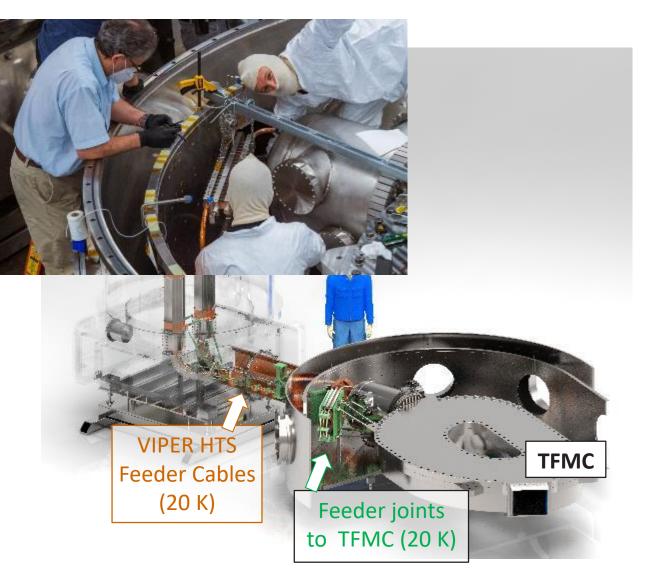
MIT

**PSFC** 

# Binary 50 kA HTS current leads and feeder system proven



- Binary HTS current leads were designed, fabricated, and commissioned in-house
  - Designed to supply up to 50 kA for low voltage DC magnets
  - LN2 section can be sub-cooled to enable high current performance
  - In-house development required to meet performance and schedule requirements
- Feeder system to magnet composed of 3 sets of VIPER HTS cables
  - Complex shape to mitigate thermally induced differential strain due to cooldown
  - 3 sets of joints to simplify assembly
  - Unique high-pressure feedthrough to enable connection to TFMC magnet
- Leads and feeder system commissioned in advance of TFMC installation
  - Tested to 41 kA (max required current)
  - All joints with 1.5 2.0 nOhm performance



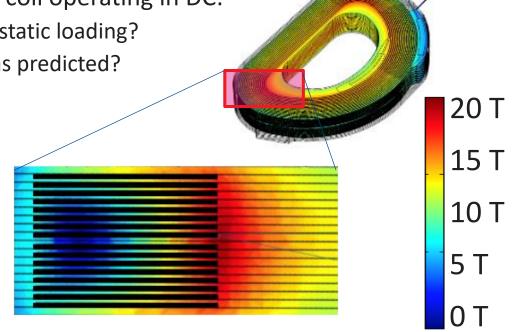


# 20 tesla test program : Sept 2021

# 1<sup>st</sup> Test: Assess DC operation of the TFMC at full performance



- First full-performance test asked 3 primary questions of the coil operating in DC:
  - Does the TFMC precisely match design B-field and withstand static loading?
  - Does the coil distribute current during charging and flat-top as predicted?
  - Does the TFMC distribute voltage (heating) as predicted?



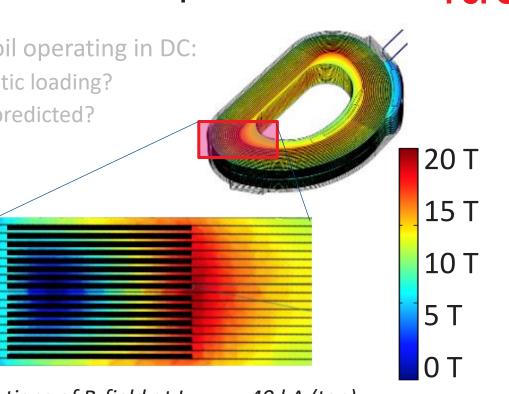
EM simulations of B-field at I<sub>terminal</sub>=40 kA (top) and the test plan for the approach to 20 T (bottom)

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  - Does the coil distribute current during charging and flat-top as predicted?
  - Does the TFMC distribute voltage (heating) as predicted?
- The winding pack contained an extensive array of *internal* embedded instrumentation to provide complete characterization of the coil electrically and thermally
  - >180 voltage taps (internal)
  - >30 Cernoxes RTDs (internal)
  - 4 embedded hall probes (internal)
  - Helium flow and pressure sensors (internal)
  - Strain gauges (external)
- Two external 3D hall probes were used to produce robust confirmation of magnetic field metrology
  - Calibrated against fiber optic current sensors (FOCS) measuring azimuthal current in the coil

EM simulations of B-field at I<sub>terminal</sub>=40 kA (top) and the test plan for the approach to 20 T (bottom)



# 1<sup>st</sup> Test: Assess DC operation of the TFMC at full performance



20 T

15 T

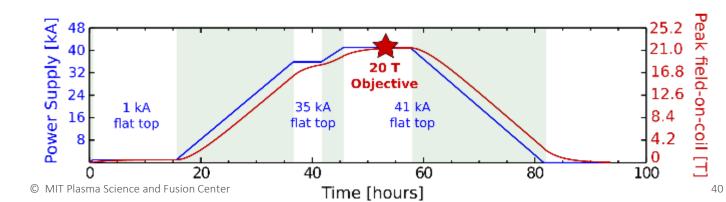
10 T

5 T

0 T

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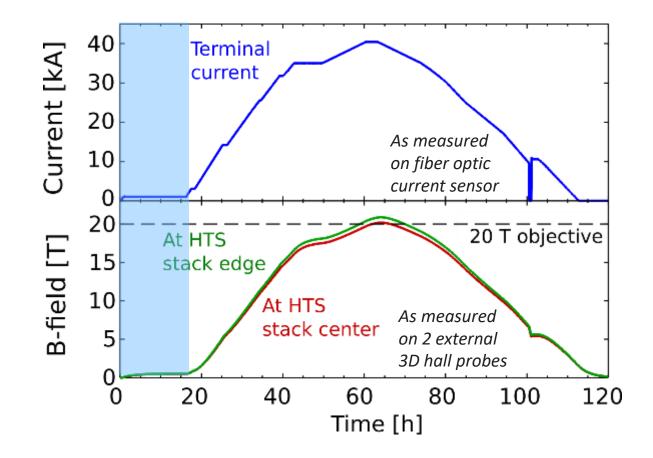
EM simulations of B-field at I<sub>terminal</sub>=40 kA (top) and the test plan for the approach to 20 T (bottom)



#### 1<sup>st</sup> Test: Key performance objectives met for the TFMC at 20 T **PSFC**

#### Step 1: Ramp to and hold at 1 kA for 15 hours

- Electrical and thermal stability check
- L/R radial current decay measurement to <1 %
- Calibration of FOCS against 3D teslameters



MIT

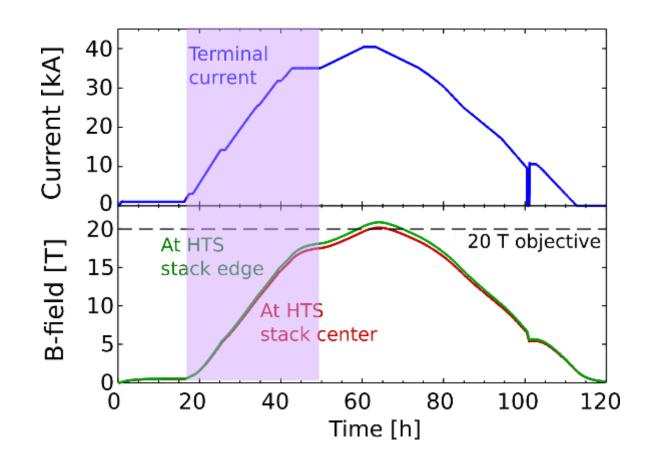
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Step 1: Ramp to and hold at 1 kA for 15 hours

- Electrical and thermal stability check
- L/R radial current decay measurement to <1 %
- Calibration of FOCS against 3D teslameters

#### Step 2: Ramp to and hold at 35 kA for 8 hours

- Pancake joint, HTS stack resistance meas'ts
- Evaluation of thermal stability, cooling capabilities
- Charging dissipation meas'ts: 245 W



MIT

**PSFC** 

# 1<sup>st</sup> Test: Key performance objectives met for the TFMC at 20 T

MIT PSFC

Step 1: Ramp to and hold at 1 kA for 15 hours

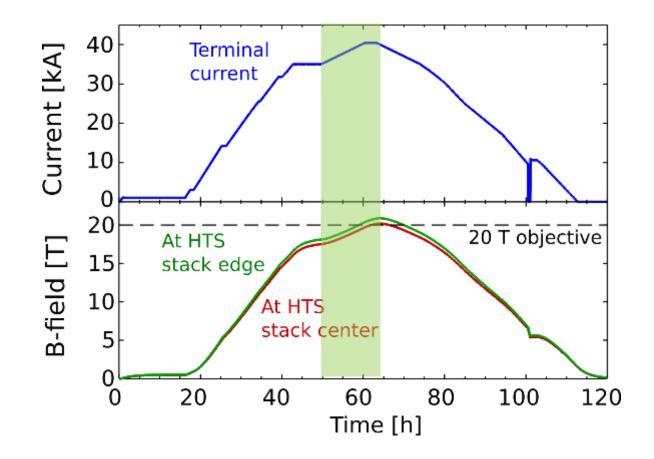
- Electrical and thermal stability check
- L/R radial current decay measurement to <1 %
- Calibration of FOCS against 3D teslameters

Step 2: Ramp to and hold at 35 kA for 8 hours

- Pancake joint, HTS stack resistance meas'ts
- Evaluation of thermal stability, cooling capabilities
- Charging dissipation meas'ts: 245W

#### Step 3: Ramp and hold at 40.5 kA for 5 hours

- Achievement of ~20.5 T peak field-on-coil
- Achievement of 109 MJ stored energy
- Steady state dissipation meas'ts: 112 W



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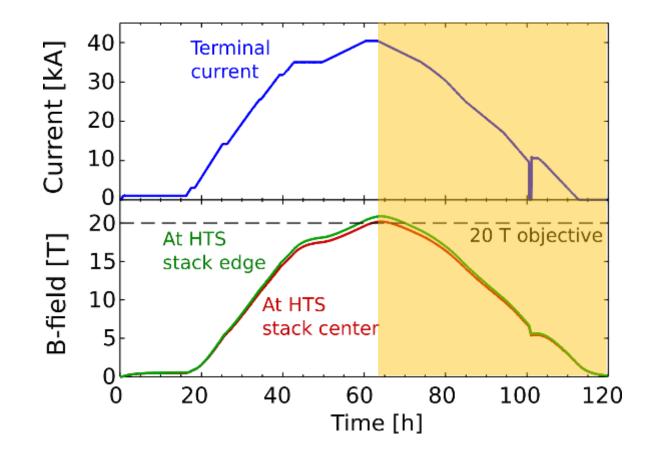
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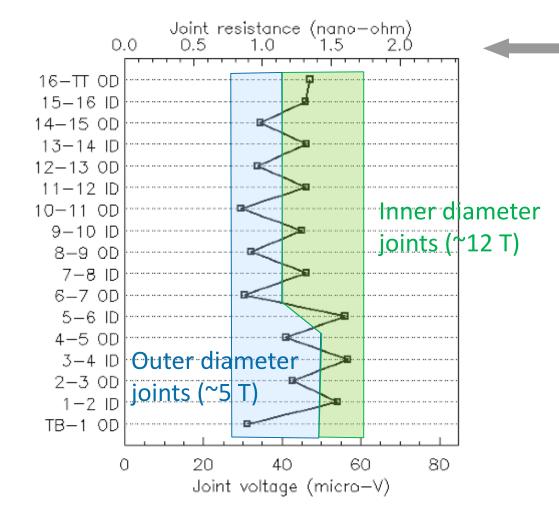
#### Step 4: Ramp down to 0 kA

- Cambridge power blew up substation around 100 h
- Coil remained charged and thermally stable despite
   ~2 kW of dissipation for 29 minutes.
- No quench and no damage sustained providing surprise initial data and confirmation of NI coil quench handling albeit at low currents (9.5 kA) and lower stored energy



# 1<sup>st</sup> Test: Other key performance objectives that were met





### Low-resistance internal pancake-to-pancake joints

- R of 1.0-1.5 n $\Omega$  at maximum current of ~40.5 kA
- Structural robust within 5 T (outer) and 12 T (inner)

### Excellent cryogenic performance, stability, control

- WP temperature control between 18 32 K
- WP temperature uniformity of 1 2 K

### Significant structural loading handled as designed

- Winding pack stress >800 MPa, case >900 MPa
- Smooth stress-strain; strain gauges matched prediction

### Excellent matches to simulated predictions

- Global B-field magnitude and 3D metrology
- Magnet charging/settling times
- Voltage distribution within pancakes
- Cryogenic cooling and temperature distributions

# The TFMC has established a solid foundation to design and operate large-scale, large-bore REBCO magnets exceeding 20 tesla

The TFMC has begun a new generation of superconducting magnets at unprecedented performance and compact size

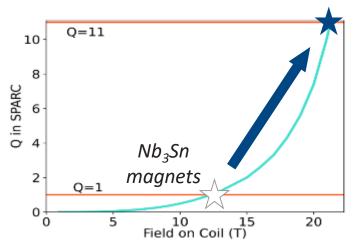
- Established manufacturing knowledge base to begin commercial-scale production
- Created enabling innovations (e.g. 50 kA HTS current leads, advanced EM modeling, etc.)
- Resulted in establishment of a new, highly capable magnet test facility at MIT

There remains important experiment work to be done

- Assessing the operational limits of quench resiliency
- Robustness to degradation from mechanical, thermal cycling

TFMC NI Design Feature	TFMC Scorecard
Modular, simple construction	Success
Intrinsically low voltage (<1 V)	Success
Pressure vessel cooling approach	Success
High winding pack current density	Success
High thermal stability	Partial success (but more work required)
Resiliency to quench	Incomplete (active program underway now)





20 T magnets enable compact highfield fusion tokamaks to achieve 10x the power out over power in.