

Examination of C, Be, Mo, ²³⁸U, Fe, and Zr using the RPI HES Data with Current ENDF, JEFF, and JENDL Evaluations

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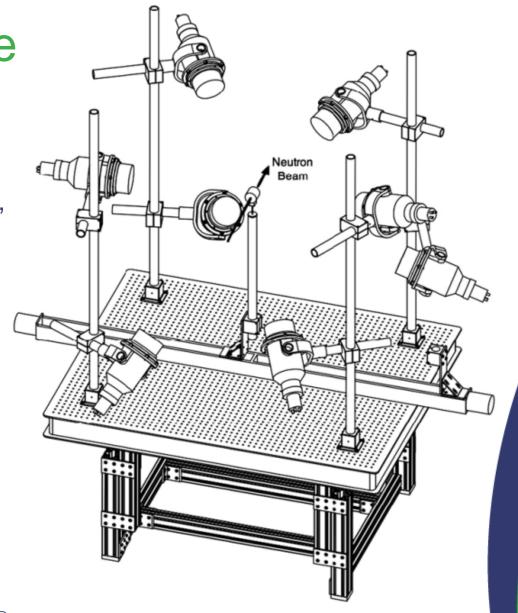
Rensselaer Polytechnic Institute
Peter Brain, Hunter Belanger, and Yaron Danon

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Outline

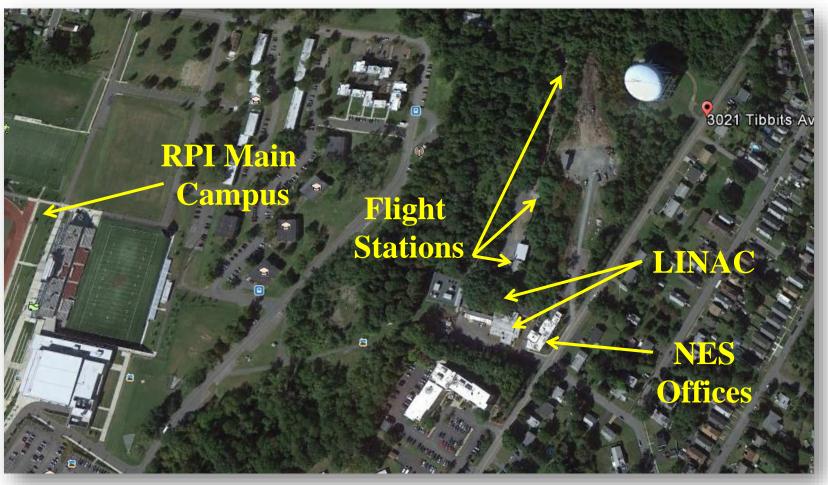
- RPI Gaerttner LINAC Center
 - LINAC specifications and history
 - Neutron production targets, detector systems, and capability matrix
- High Energy Scattering System
 - Eight EJ301 proton recoil fast neutron detectors
 - Neutron flux measurements and detector efficiencies
 - Data processing and reference sample
- High Energy Scattering Results
 - Carbon, molybdenum, zirconium, beryllium, ²³⁸U, and iron





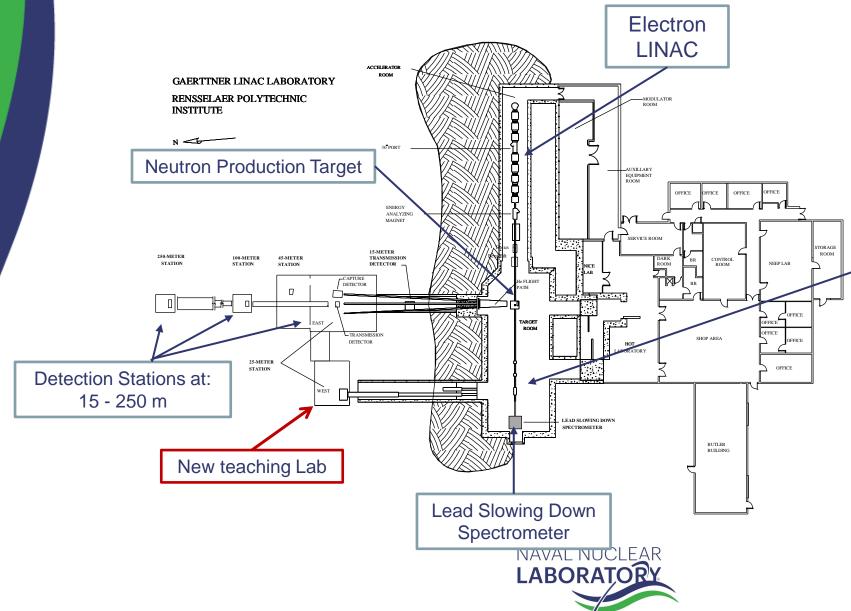
Where is the RPI Gaerttner LINAC Center?

It is on the highest point in Troy, NY





The RPI Gaerttner LINAC Center





Current LINAC Specifications

| | Three Sections (Low Energy Port) | Nine Sections (High Energy Port) |
|--------------------------|---|---|
| Electron Energy | 5 to 25 MeV | 25 to over 60 MeV |
| Pulse Width | 6 to 5000 ns | 6 to 5000 ns |
| Peak Current | 3A (short pulse: 6 to 50 ns) 400 mA (long pulse: 50 to 5000 ns) | 3A (short pulse: 6 to 50 ns) 400 mA (long pulse: 50 to 5000 ns) |
| Average Power | 10 kw@ 17 MeV, 5000 ns | >10 kw@ 60 MeV, 5000 ns |
| Peak Dose Rate | >10 ¹¹ Rads/sec (in Silicon) | n/a |
| Neutron Production | n/a | ~4 X 10 ¹³ neutrons/sec |
| Pulse Repetition Rate | Single pulse to 500 pps (short pulse) Single pulse to 300 pps (long pulse) | Single pulse to 500 pps (short pulse) Single pulse to 300 pps (long pulse) |









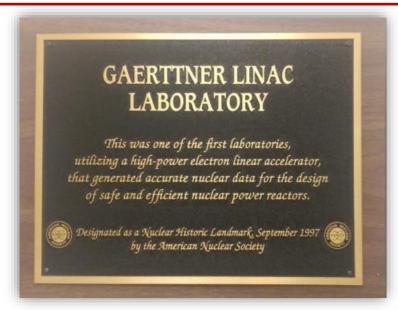
RPI LINAC History

December 1961 - The RPI LINAC started operation Working "continuously" since.

September 1997- LINAC was designated as Nuclear Historic Landmark by the American Nuclear Society







Graduated over 190 students who utilized the LINAC as part of their graduate thesis research

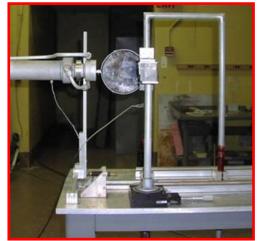
Many years of accumulated experience

2014 - Started a major refurbishment and upgrade project



Neutron Production Targets (electrons → neutrons)

Bare Bounce Target (BBT)

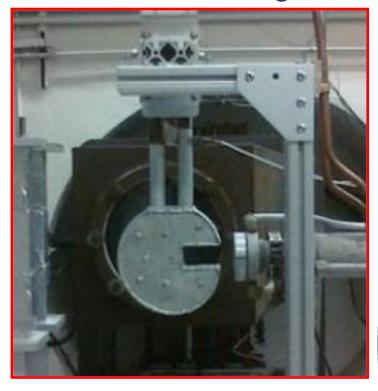




Enhanced Thermal Target (ETT)



PACMAN target







Detector Systems

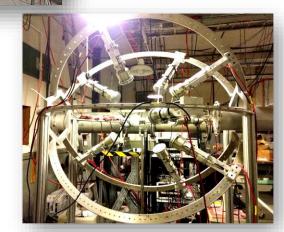
Capture/Multiplicity

Tuene

Transmission

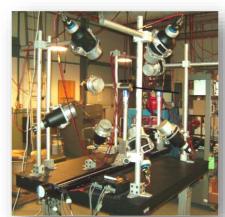


Fast Scattering and PFNS @ 30m



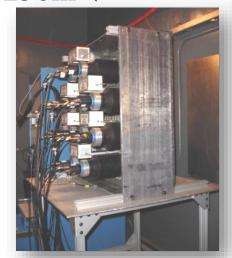
25_m

keV Scattering @ 35m





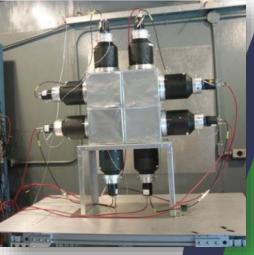




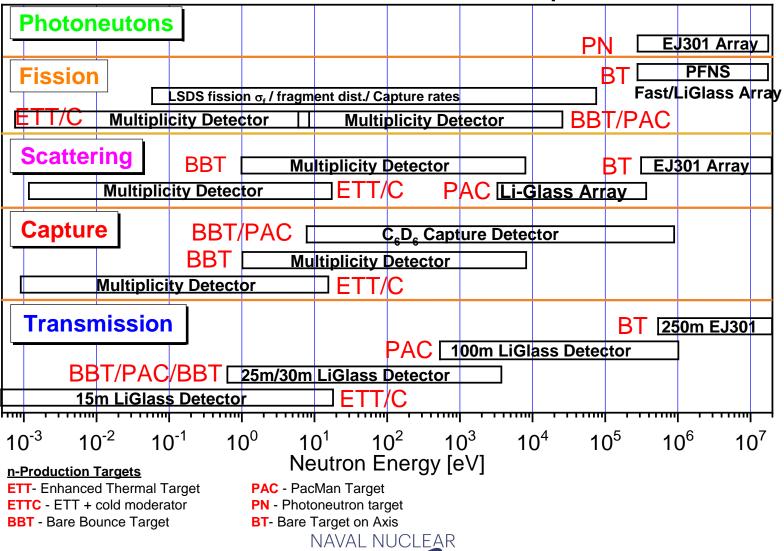
1000

35m_





Capability Matrix and Development RPI LINAC - Nuclear Data Measurement Capabilities 2023

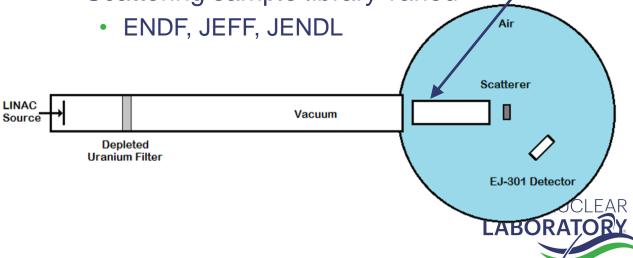


LABORATOR

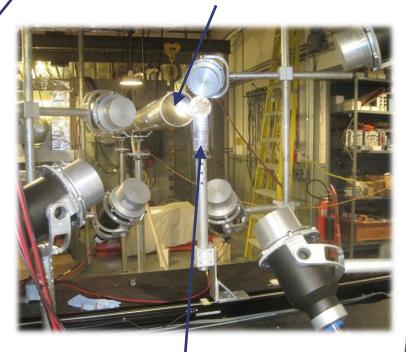
High Energy Scattering System

- Each experiment collected three sets of data:
 - Sample of Interest, e.g., ²³⁸U or Iron
 - Graphite Reference Sample
 - Open Beam
- Fluctuations in neutron intensity recorded by beam monitors
- Experiments were modeled in MCNP

Scattering sample library varied



Upstream vacuum reduced open beam contribution[‡]



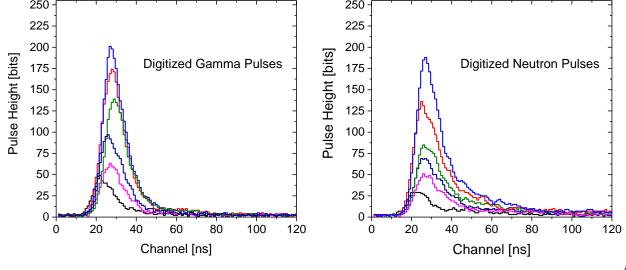
Low mass sample holder

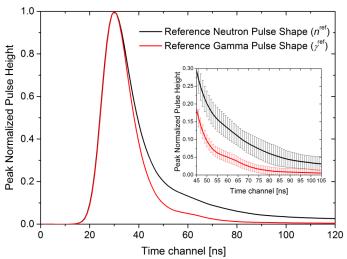
‡ Present after 2011

Data Processing and Analysis

- Acqiris AP240 DAQ Board:
 - Each digitized event contained 120
 8-Bit data points that were used to reconstruct the waveform

- Each digitized event was analyzed to extract useful parameters:
 - Channel
 - Acquisition time
 - Area (pulse height)
 - Shape







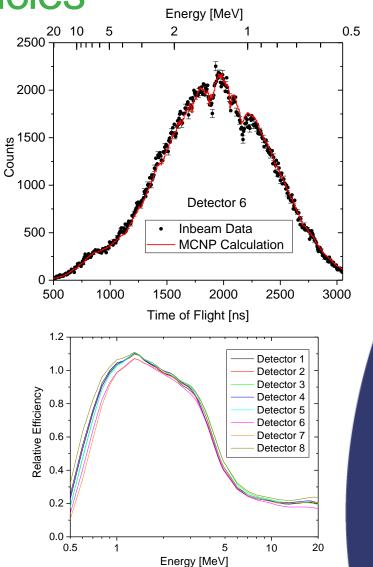
EJ-301 Detector Efficiencies

- Neutron flux measured with ²³⁵U Fission Chamber
- Small variations between efficiencies are present
 - Lower-level discriminator
 - Scintillator-to-detector Coupling
 - Photomultiplier Tube Gain
- MCNP was used to calculate the measured flux shape at the in-beam detector position
 - Flux shape and count rate were used to calculate individual detector efficiencies

$$\eta_i(E) = \frac{R_i(E)}{\phi(E)}$$

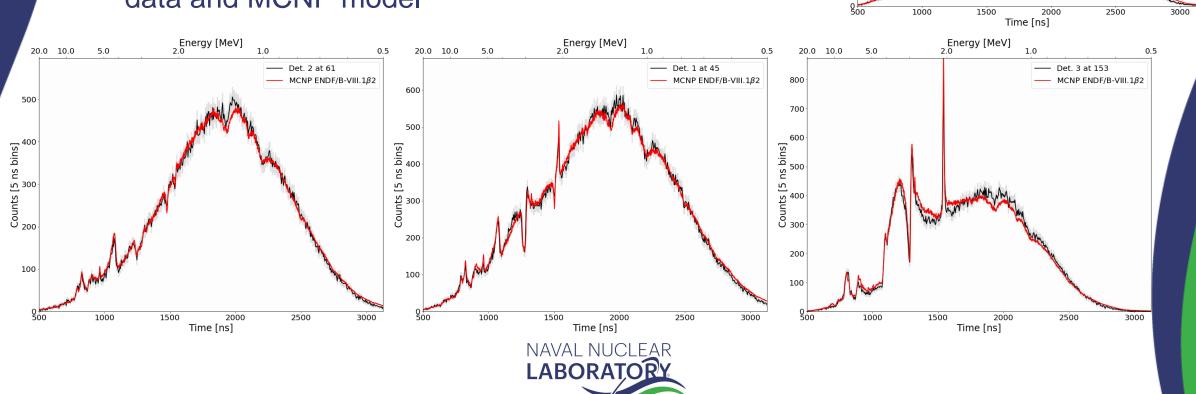
Relative efficiencies were used in MCNP models





Carbon

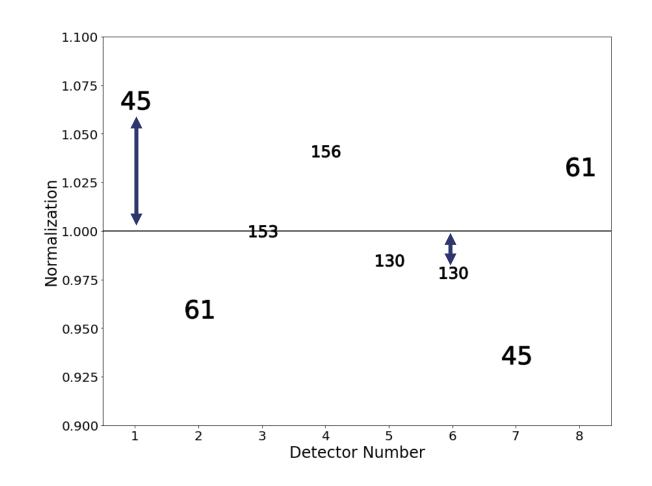
- Carbon (graphite) was used as a reference sample with all high energy scattering measurements
- Generally good agreement between measured data and MCNP model



20.0 10.0

Carbon – Continued

- Each detector's relative normalization
 - Detector locations represented by plotted symbols
- Differences in the normalization are used to quantify systematic uncertainties





Molybdenum

Molybdenum Energy [MeV]

> ENDF/B-VIII.0 JENDL-5

ENDF/B-VIII.1B2

Time [ns]

2000

1.0

JEFF-3.3 IENDL-4

1500

0.5

20.0 10.0

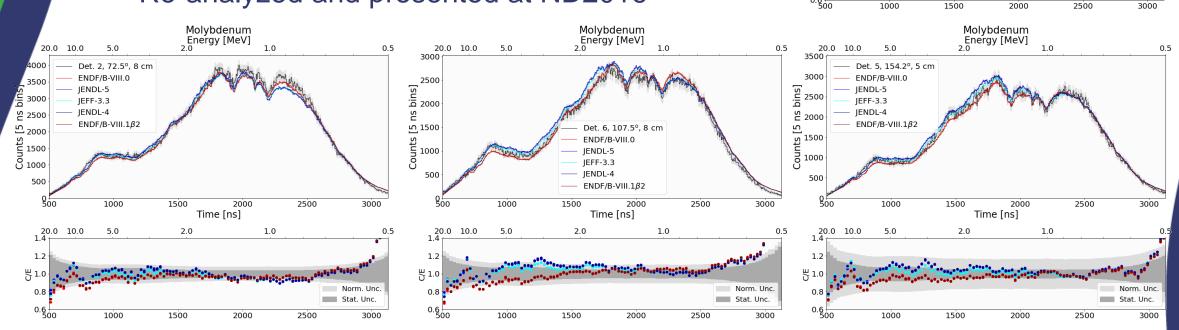
3500

suig 2500

2000 1500

ල් 1000

- Two molybdenum samples, 5cm and 8cm, were originally measured in 2008
- Re-analyzed and presented at ND2016*



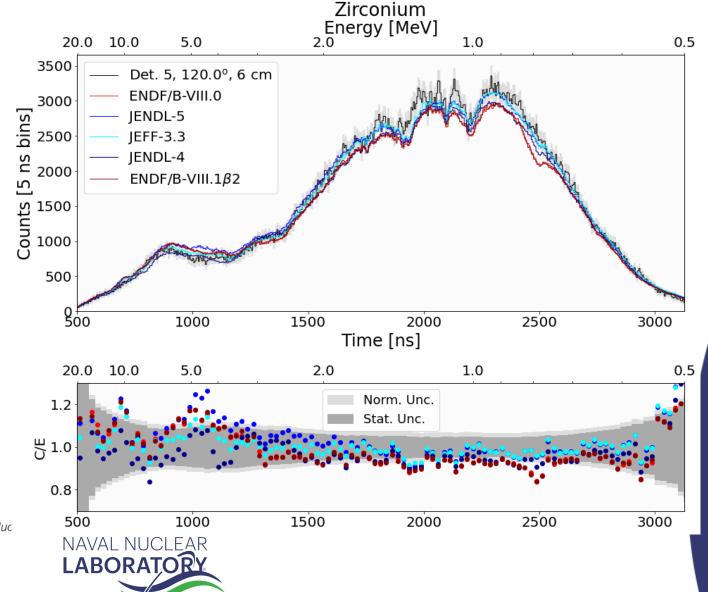
*Daskalakis et al., "Assessment of beryllium and molybdenum nuclear data files with the RPI neutron scattering system in the energy region from 0.5 to 20 MeV", EPJ Web Conf., vol. 146, pp. 11037, 2017



Zirconium

Two zirconium samples,
 6cm and 10cm, were
 measured in 2009*

 More zirconium results will be presented by G. Siemers



*Barry et al., "Quasi-Differential Neutron Scattering in Zirconium from 0.5 to 20 MeV", Nuc

Beryllium

Beryllium Energy [MeV]

2000

Time [ns]

1500

0.5

3000

20.0 10.0

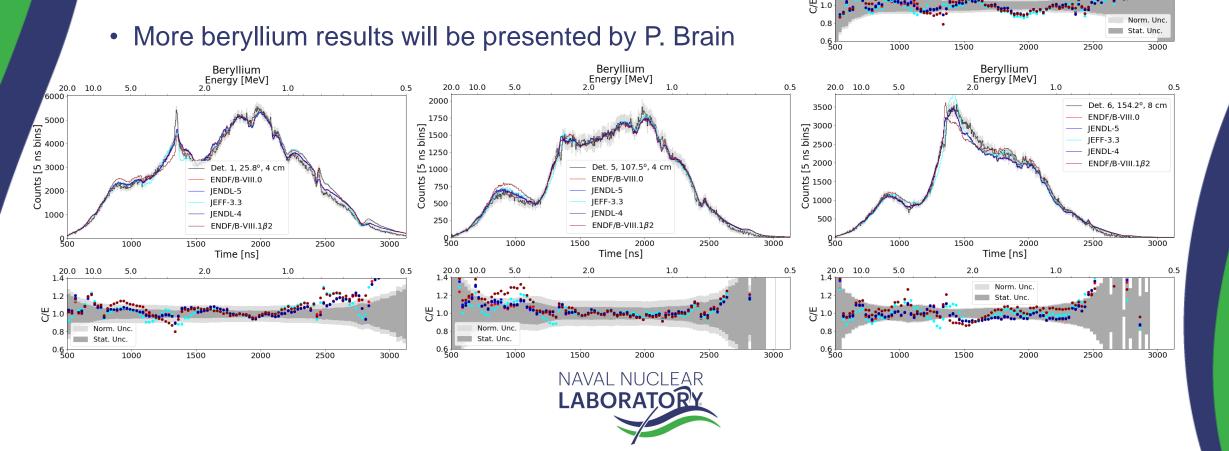
O 1000

Det. 2, 52.0°, 4 cm ENDF/B-VIII.0 JENDL-5

1000

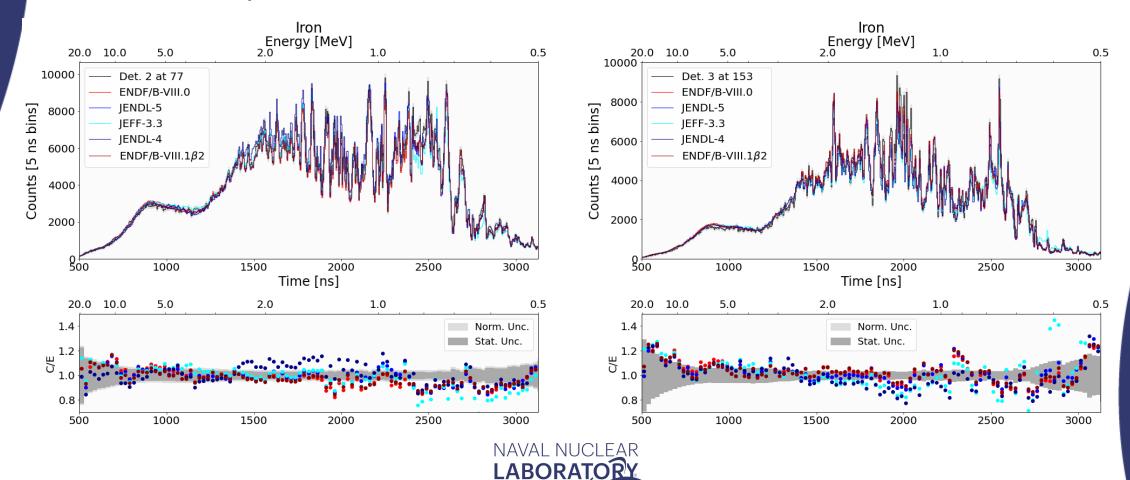
JENDL-4 ENDF/B-VIII.1β2

- Two beryllium samples, 4cm and 8cm, were originally measured in 2007
- Evaluations have difficulty between 2 and 3.5 MeV



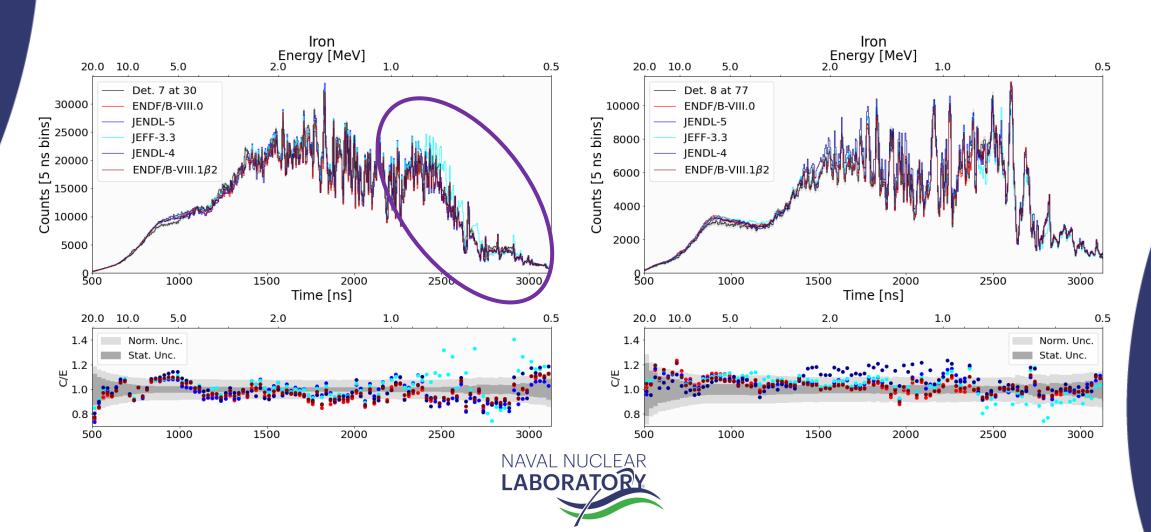
Iron

- Measured in 2012.
 - Total quasi-differential, inelastic-to-elastic ratios, and elastic-only
- Results first presented in Dresden at WINS2014



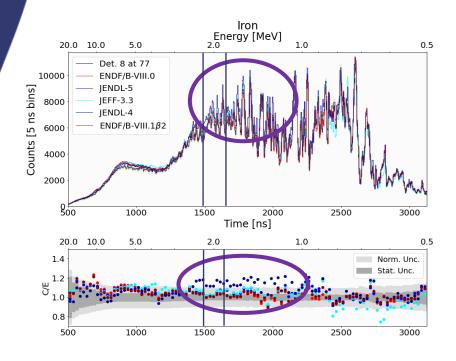
Iron - Continued

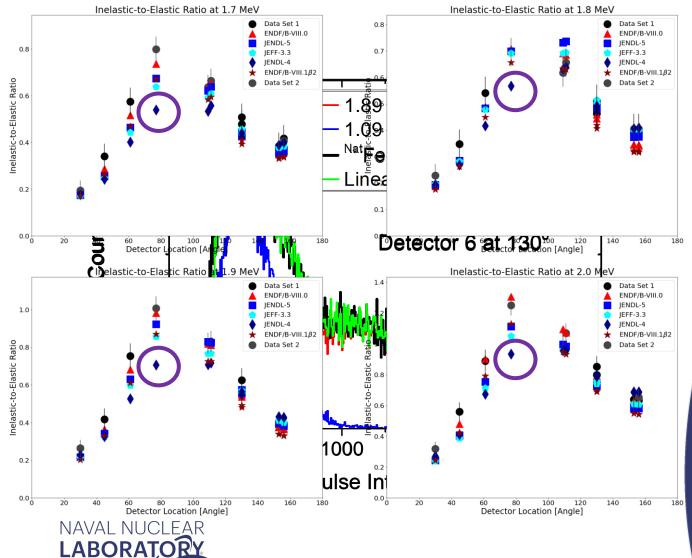
Additional iron data



Iron - Continued

- Isolate narrow time bin approximating a monoenergetic neutron beam
- Inelastic-to-elastic ratios





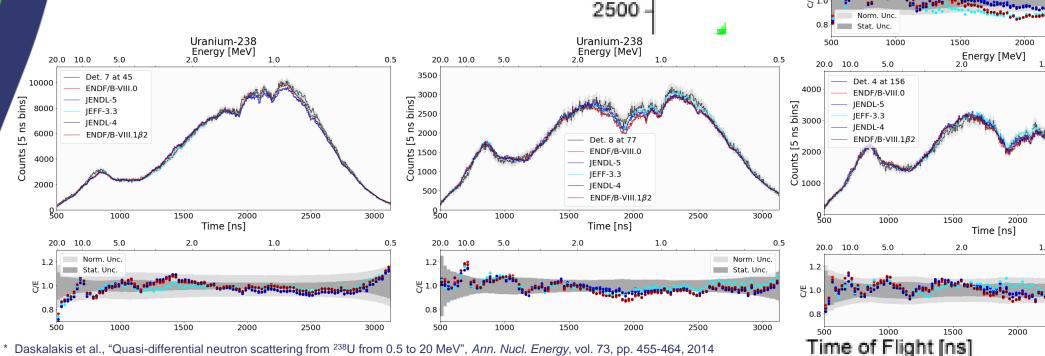
²³⁸U – Continued

3500

3000

 Measured in 2011 and first presented in Boston at WINS2012*

Collaboration with IAEA**



** Capote et al., "IAEA CIELO Evaluation of Neutron-induced Reactions on ²³⁵U and ²³⁸U Targets", *Nuclear Data Sheets*, vol. 148, pp. 254 - 292, 2018



Uranium-238 Energy [MeV]

2000

1.0

Time [ns]

2500

2500

2500

3000

3000

Stat. Unc.

1500

2.0

20.0 10.0

20.0 10.0

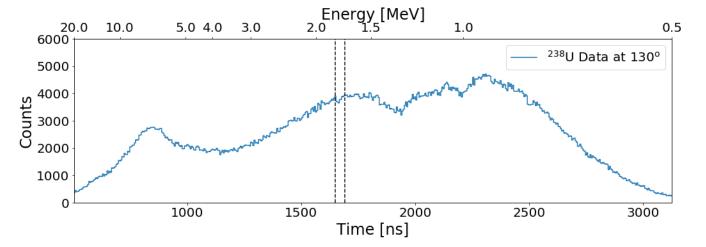
Counts 1000

ENDF/B-VIII.0 JENDL-5 JEFF-3.3 JENDL-4 ENDF/B-VIII.1*B*2

238

Fission neutron ratios

First presented at ND2022*



Summary

- Differences observed between ENDF and JEFF/JENDL with Molybdenum
- ENDF/B-VIII.1β2 compares poorly to beryllium data relative to ENDF/B-VIII.0
 - Between 2 and 3.5 MeV
- JENDL4 compares poorly to iron data
 - Inelastic and elastic ratios between 1 and 2 MeV
 - Better agreement observed with JENDL5
- Most evaluations agree favorably with ²³⁸U data
 - fission to total method further constrains

