

# Design of magnet and shielding systems for compact fusion machines

Gurdeep S. Kamal, Mayank Rajput,  
Jonathan Naish





# DELIVERING FUSION POWER AND TRANSFORMATIVE SUPERCONDUCTING MAGNET TECHNOLOGY

2025

Tokamak Energy 

# WE ARE TOKAMAK ENERGY

- THE LEADING GLOBAL FUSION ENERGY COMPANY
- THE LEADING HIGH TEMPERATURE SUPERCONDUCTING MAGNETS COMPANY



PRINCETON, NJ

U.S. SUBSIDIARY



OXFORD, UK

GLOBAL HEADQUARTERS



TOKYO, JP

JAPANESE SUBSIDIARY

260 people worldwide

\$335 million raised

Partnering with:



OAK RIDGE  
National Laboratory

UNIVERSITY OF  
ILLINOIS  
URBANA-CHAMPAIGN



GENERAL ATOMICS



UK Atomic  
Energy  
Authority

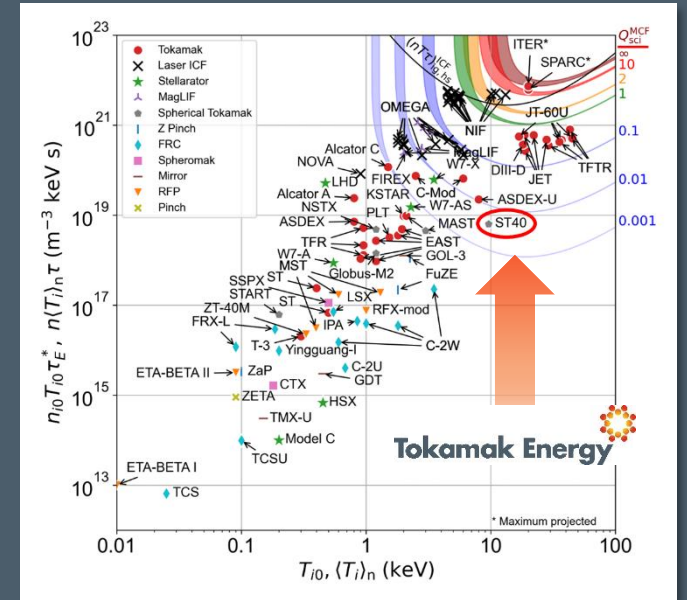
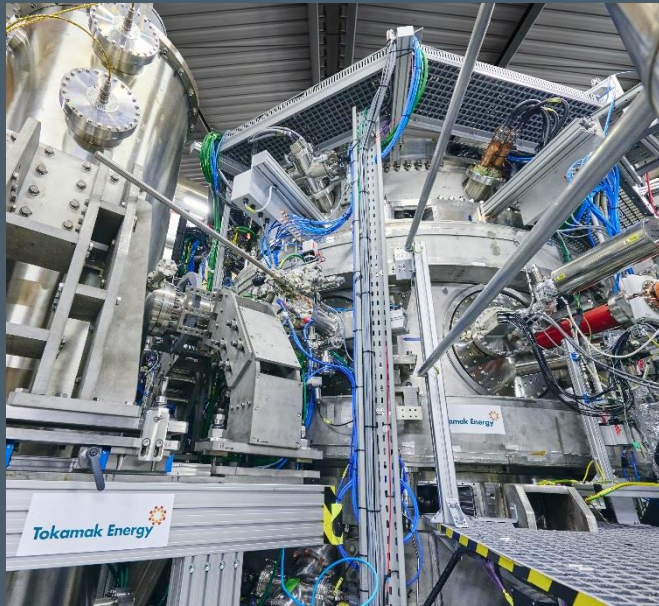
Sumitomo Corporation

KYOTO  
FUSIONEERING

東京大学  
THE UNIVERSITY OF TOKYO

FURUKAWA  
ELECTRIC GROUP

# OUR RECORD



Only company with 10+ years' experience designing and building tokamaks and HTS magnets

Unrivalled experience commissioning, operating and upgrading tokamaks

Peer reviewed highest 'triple product' in a privately funded tokamak

**$6 \times 10^{18} \text{ keV.s/m}^3$**



# OUR FUSION TECHNOLOGY

Spherical tokamak with High Temperature Superconducting (HTS) magnets



## Spherical tokamak

- High efficiency (high bootstrap current)
- Stable plasma
- Steady state running
- Requires 50% less magnet material\*



## HTS magnets

- Quench-safe, robust magnets
- Ultra-high magnetic field (20 Tesla +)
- Operate at 20K (no liquid cryogenes)

\* Compared to a conventional tokamak (CT)

# SPHERICAL TOKAMAK: ENGINEERING REQUIREMENTS

The advantages of the spherical tokamak come with some specific engineering challenges associated with the low aspect ratio and slim centre column:

## Magnet shielding

Reduced space for neutron and thermal shielding of the magnets, given fixed linear dimensions, means that novel shielding materials will be required to ensure the HTS magnets can operate for the lifetime of a power plant. *NOTE: These materials are likely to have many commercial applications.*



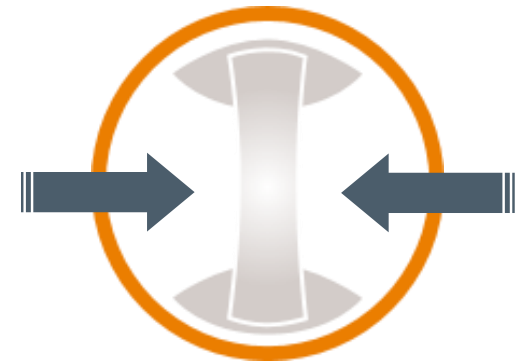
## Outboard breeder blankets

Insufficient space to accommodate an inboard tritium breeder blanket means that only outboard breeder blankets are available, with the same tritium breeding ratio (TBR) required for tritium self-sufficiency. *NOTE: The spherical geometry is favourable for outboard blankets.*

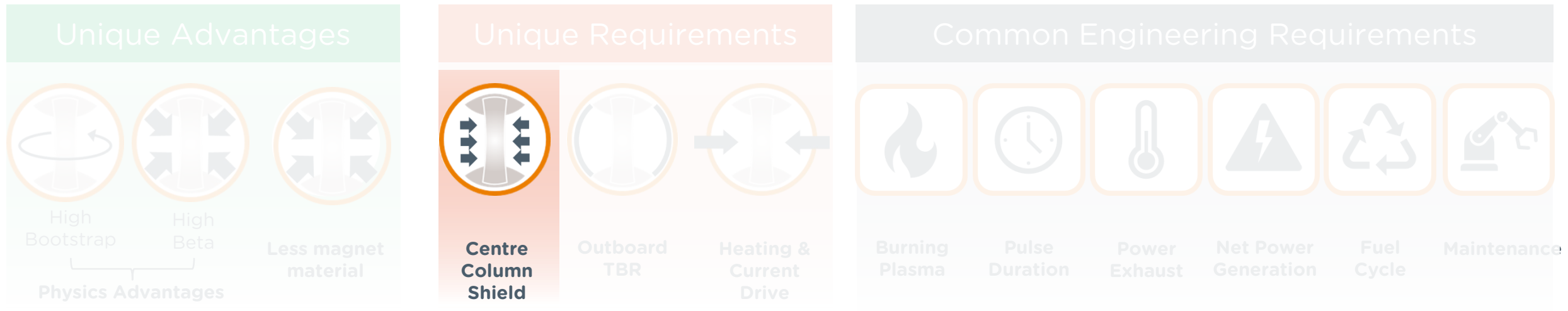


## Heating and current drive

Reduced space for a large solenoid to be used for inductive current drive means the plasma heating and non-bootstrap current drive must be driven by non-inductive means such as radio frequency (RF). *NOTE: The high bootstrap fraction helps reduce the amount of RF heating needed.*



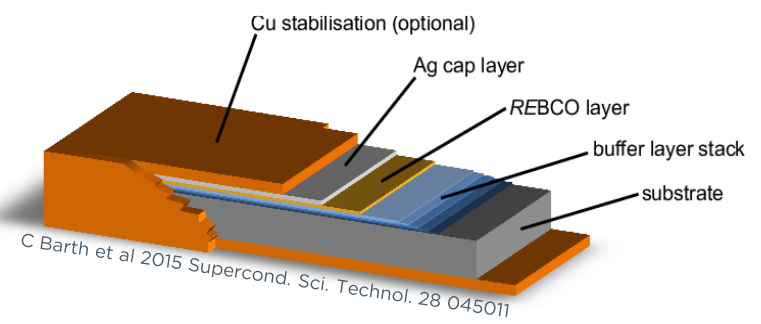
# SPHERICAL TOKAMAK: THE PATH TO DEMONSTRATION



Our path to fusion focusses on validating the physics advantages and resolving the engineering requirements.

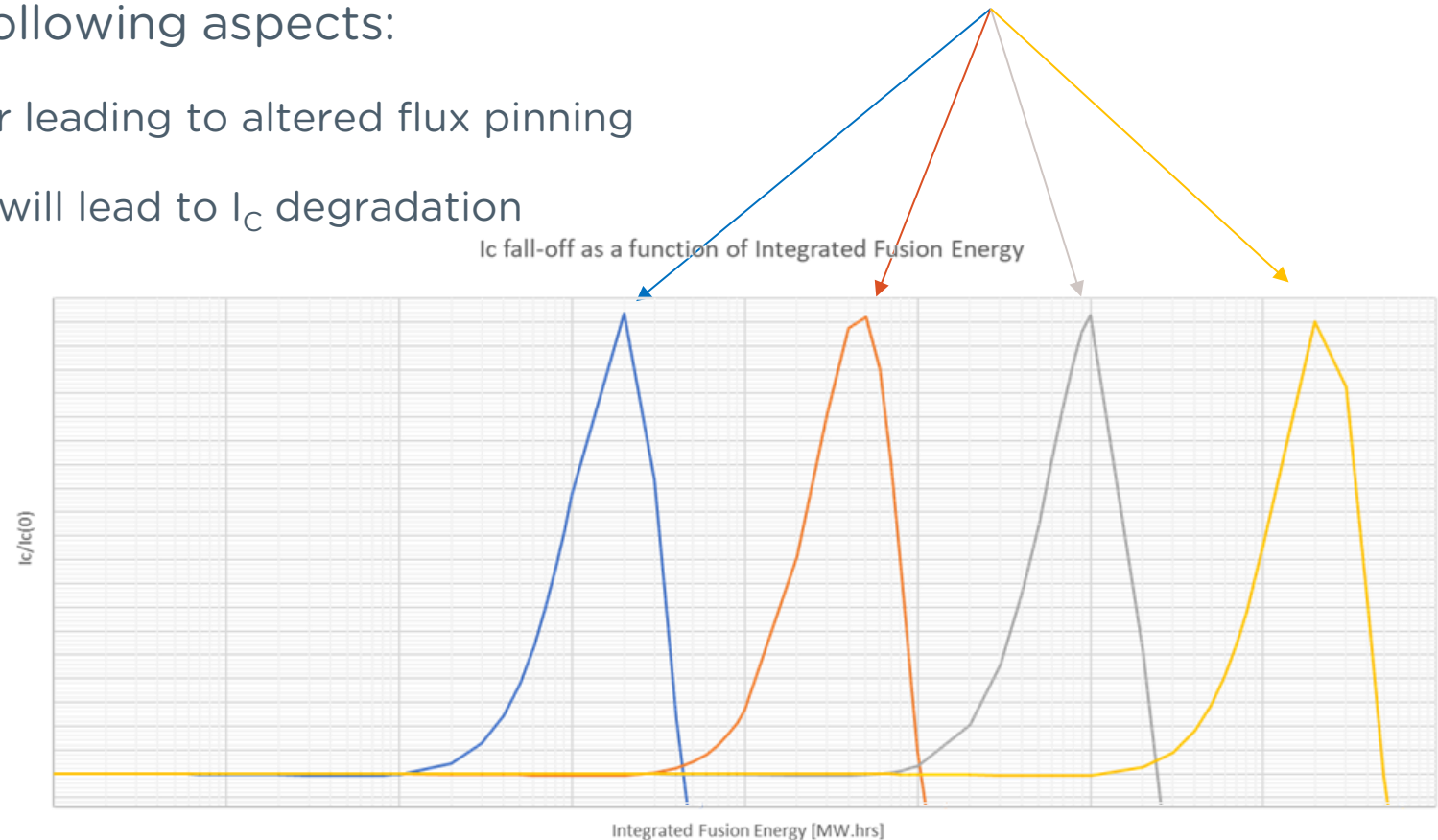
**By doing so, we are demonstrating that the HTS Spherical Tokamak provides the most commercially viable fusion energy solution.**

# HTS Tapes Must Be Resilient To Radiation Damage



- Our commercial HTS tape technology utilises a REBCO superconducting layer
- HTS tape can evolve due to the following aspects:
  - Neutron Damage of the REBCO layer leading to altered flux pinning
  - Bombardment of the REBCO lattice will lead to  $I_c$  degradation
- Additionally,
  - The HTS tape will activate causing additional damage
  - The REBCO will transmute potentially forming into something non-superconducting

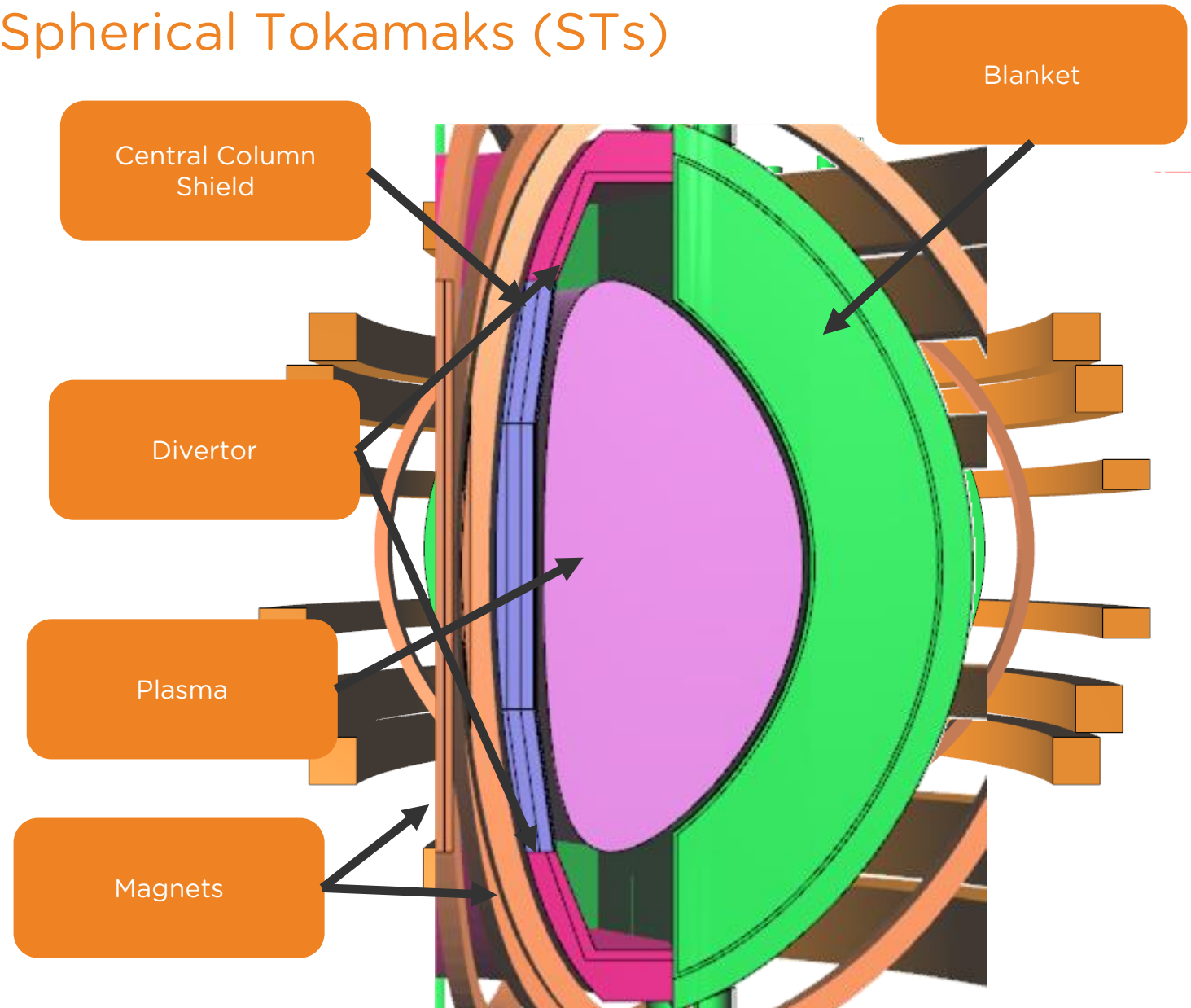
Various Shielding Tactics can help with reduction of HTS tape degradation





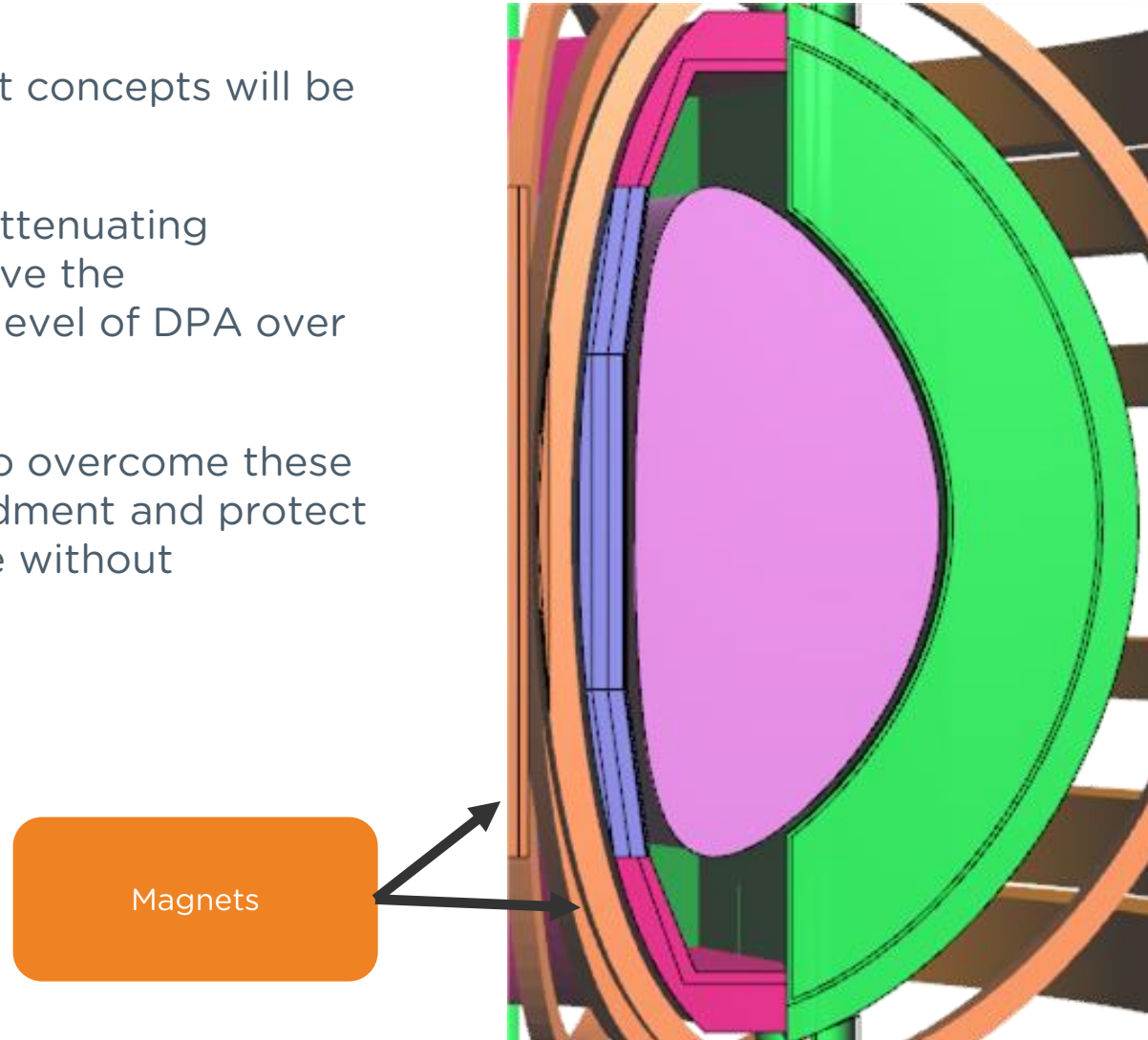
# Shielding Space Is Limited In Spherical Tokamaks (STs)

- The space available between the central column and the plasma is limited due to the compactness of a ST.
- The use of efficient shielding is necessary in order to protect the fusion plant by:
  - Reducing the instantaneous, local, heat to the magnets so they do not go above operational temperatures.
  - Reducing overall heat to the magnets so they do not impose significant cooling requirements on the cryogenic systems.
  - Reducing the dose to the magnets so they remain functional throughout the proposed lifetime (Activation & Dose).



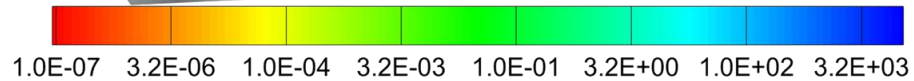
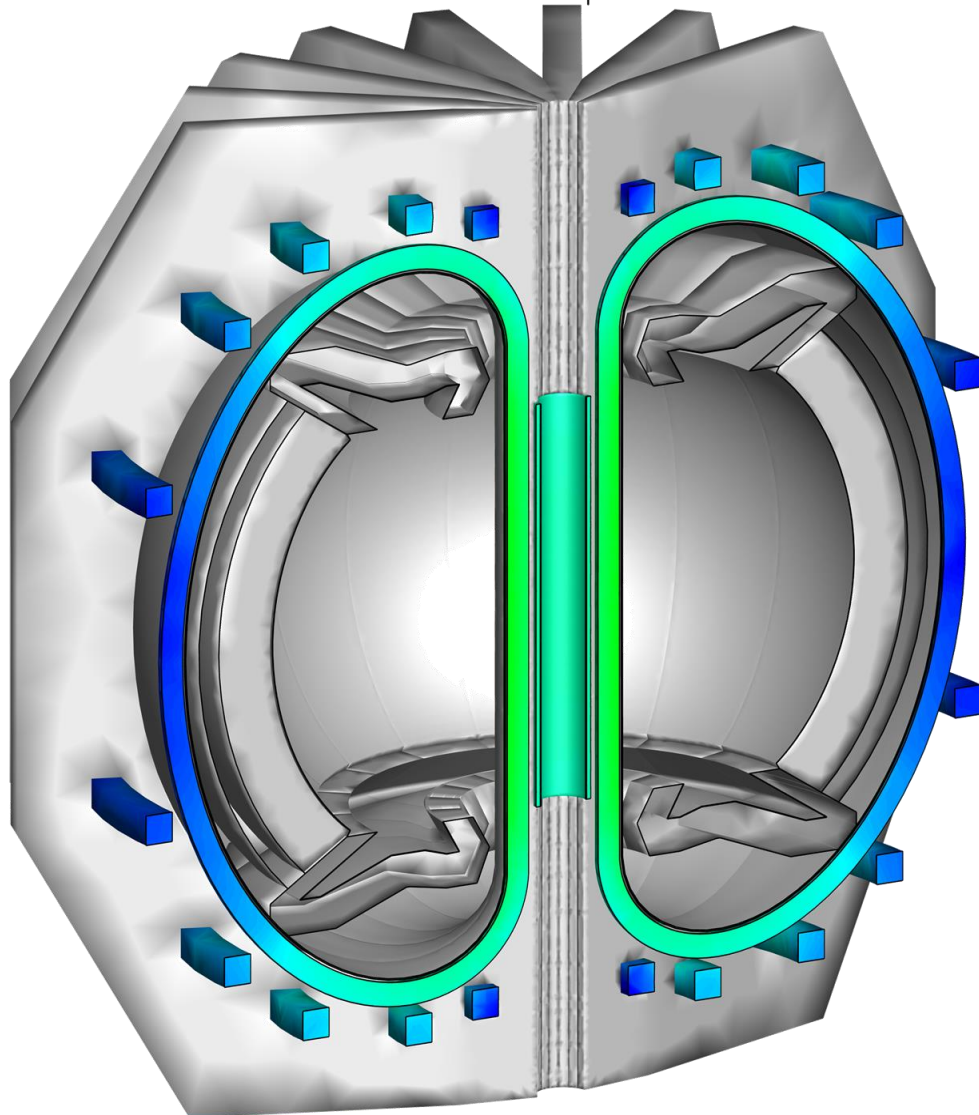
# Shielding Space Is Limited In Spherical Tokamaks (STs)

- The magnet lifetime with a typical blanket concepts will be less than 10 days for STs.
- “Traditional” shielding materials are not attenuating enough or are not robust enough to survive the environment of a tokamak; including the level of DPA over a reasonable duty cycle.
- Advanced shields are therefore needed to overcome these challenges; survive the radiation bombardment and protect the magnets over the determined lifetime without significantly increasing the size of the ST.

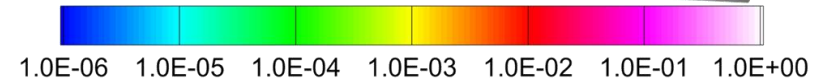
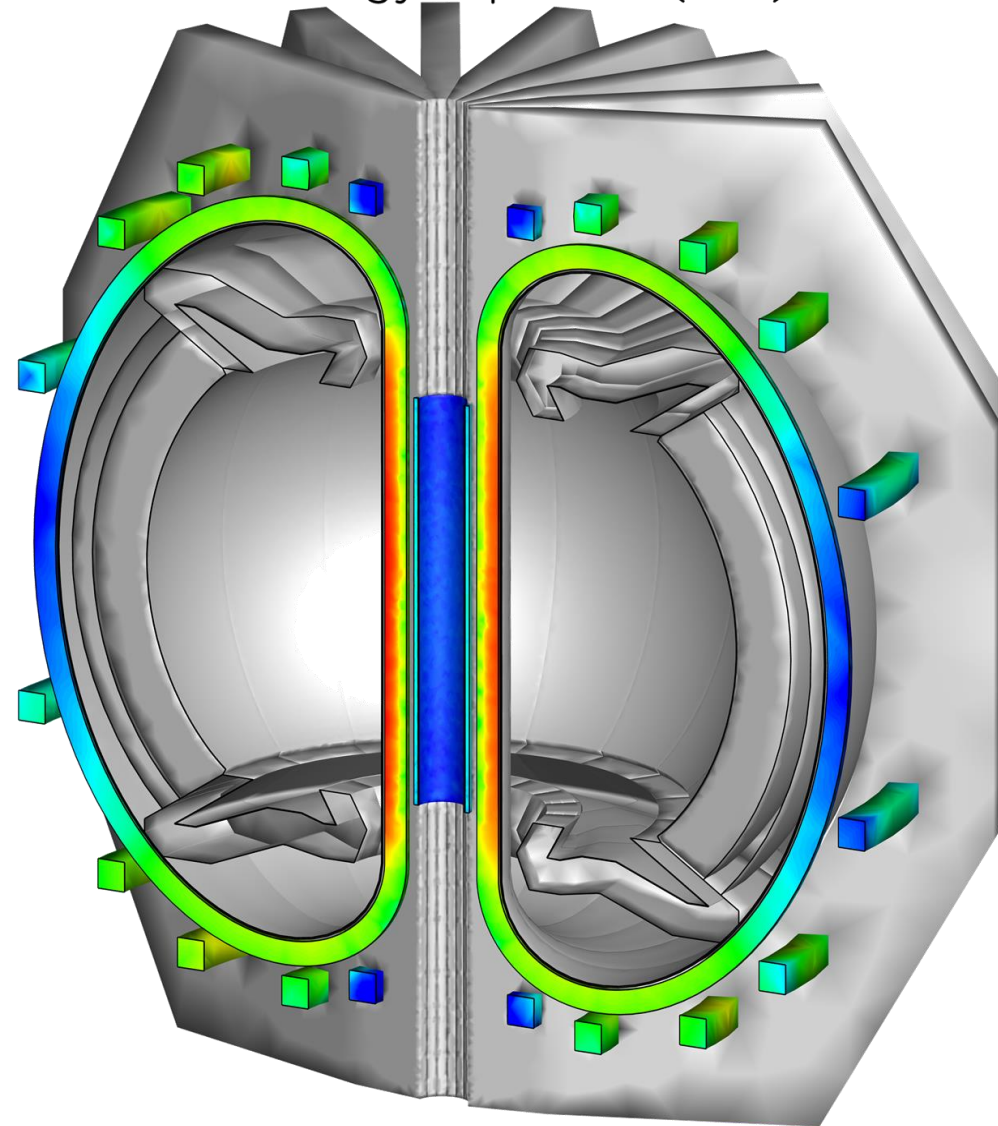


# Consequences Of No Shielding

Magnet Lifetime (1GW<sub>f</sub> Full Power Days)

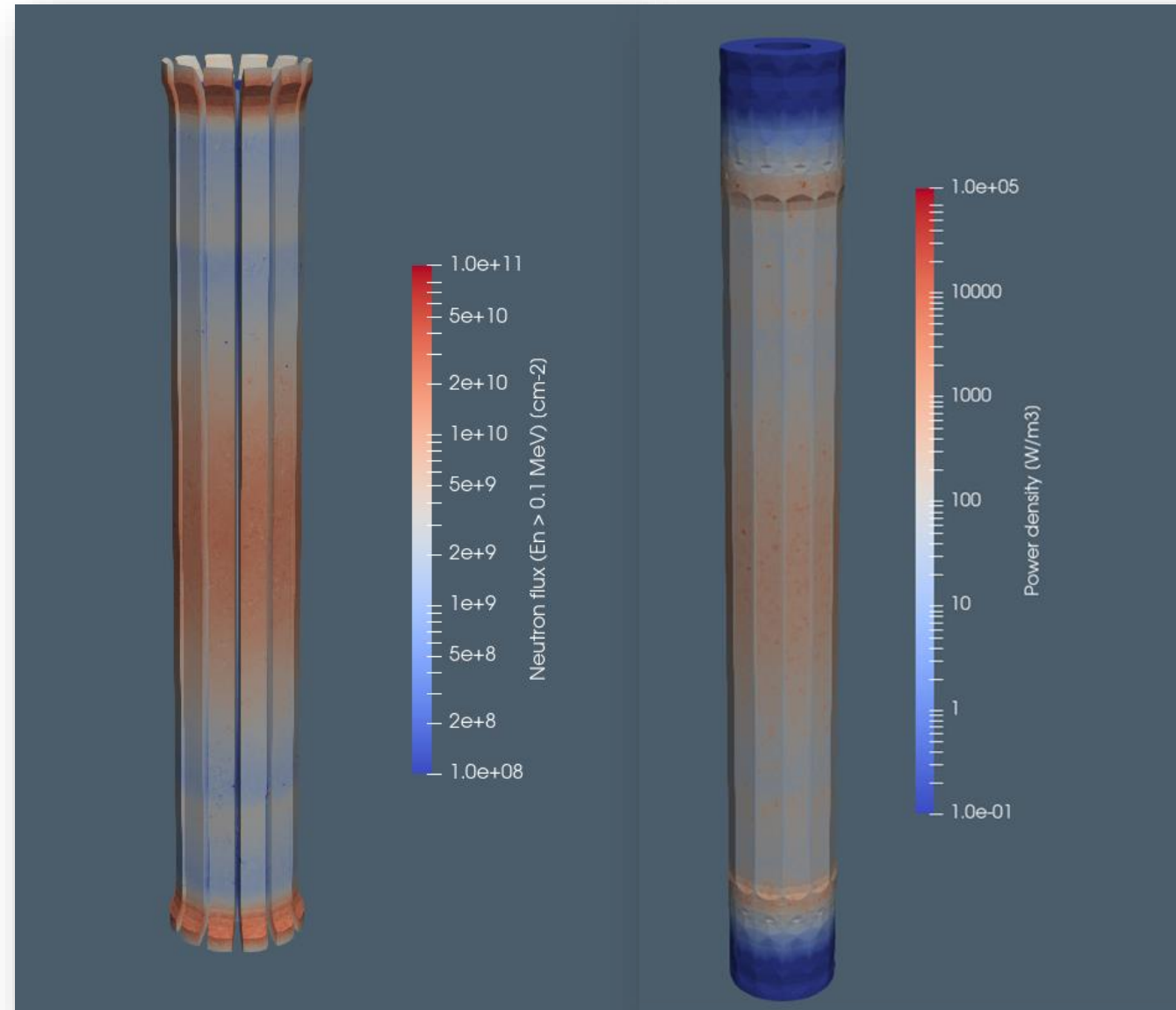


Total Energy Deposition (MW)



# Considerations Beyond Magnet Lifetime

- From a recent study, the fast neutron flux in the toroidal field windings, within the centre column, shows acceptable shielding performance for commercial fusion concept
- However, the total nuclear power in the cold mass was ~50 kW which would require significant cryo-plant capacity, with heavy cost and recirculating power implications.
- Protecting the magnets from the fast neutrons is not sufficient to protect them from high heat load, and the entire cold mass has to be well shielded to reduce the capacity of the cryo-plant and the associated plant recirculating power.



# Search For Advanced Material For Radiation Shielding

## PERIODIC TABLE OF ELEMENTS

Chemical Group Block

PubChem

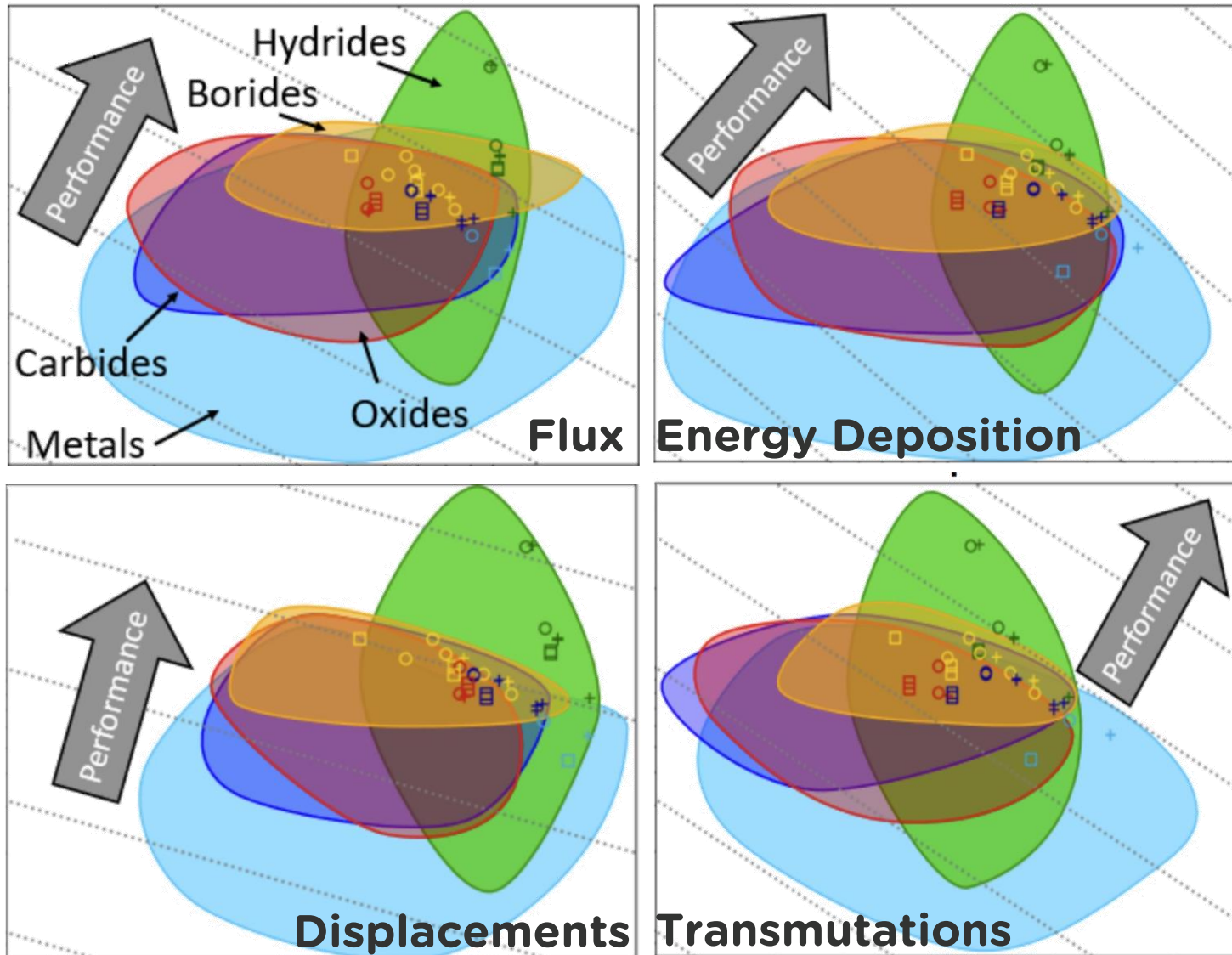
Atomic Number		17		35.45		Atomic Mass, u																													
Name		Cl		Chlorine		Halogen																													
Chemical Group Block																																			
1	1.0080							2	4.00260																										
3	7.0	4	9.012183							10	20.180																								
11	22.989...	12	24.305							18	39.9																								
19	39.0983	20	40.08	21	44.9559...	22	47.867	23	50.9415	24	51.996	25	54.93804	26	55.84	27	58.93319	28	58.693	29	63.55	30	65.4	31	69.723	32	72.63	33	74.92159	34	78.97	35	79.90	36	83.80
37	85.468	38	87.62	39	88.90584	40	91.22	41	92.90637	42	95.95	43	96.90636	44	101.1	45	102.9055	46	106.42	47	107.868	48	112.41	49	114.818	50	118.71	51	121.760	52	127.6	53	126.9045	54	131.29
55	132.90...	56	137.33	72	178.49	73	180.9479	74	183.84	75	186.207	76	190.2	77	192.22	78	195.08	79	196.96...	80	200.59	81	204.383	82	207	83	208.98...	84	208.98...	85	209.98...	86	222.01...		
87	223.01...	88	226.02...	104	267.1...	105	268.1...	106	269.1...	107	270.1...	108	269.1...	109	277.1...	110	282.1...	111	282.1...	112	286.1...	113	286.1...	114	290.1...	115	290.1...	116	293.2...	117	294.2...	118	295.2...		
57	138.9055	58	140.116	59	140.90...	60	144.24	61	144.91...	62	150.4	63	151.964	64	157.2	65	158.92...	66	162.500	67	164.93...	68	167.26	69	168.93...	70	173.05	71	174.9668						
89	227.02...	90	232.038	91	231.03...	92	238.0289	93	237.04...	94	244.06...	95	243.06...	96	247.07...	97	247.07...	98	251.07...	99	252.0830	100	257.0...	101	258.0...	102	259.1...	103	266.1...						

- Tungsten and hafnium have good scattering cross section and density.
- Carbon and hydrogen has good neutron moderation, and boron has excellent neutron absorption capabilities.
- Other elements are also good but come with economic/supply-chain/Material-stability issues:
  - Metallic Hydrogen
  - Re + Os based alloys
  - NiH<sub>2</sub>

National Center for Biotechnology Information (2023). Periodic Table of Elements. Retrieved June 12, 2023, from <https://pubchem.ncbi.nlm.nih.gov/periodic-table/>.



# Search For Advanced Material For Radiation Shielding

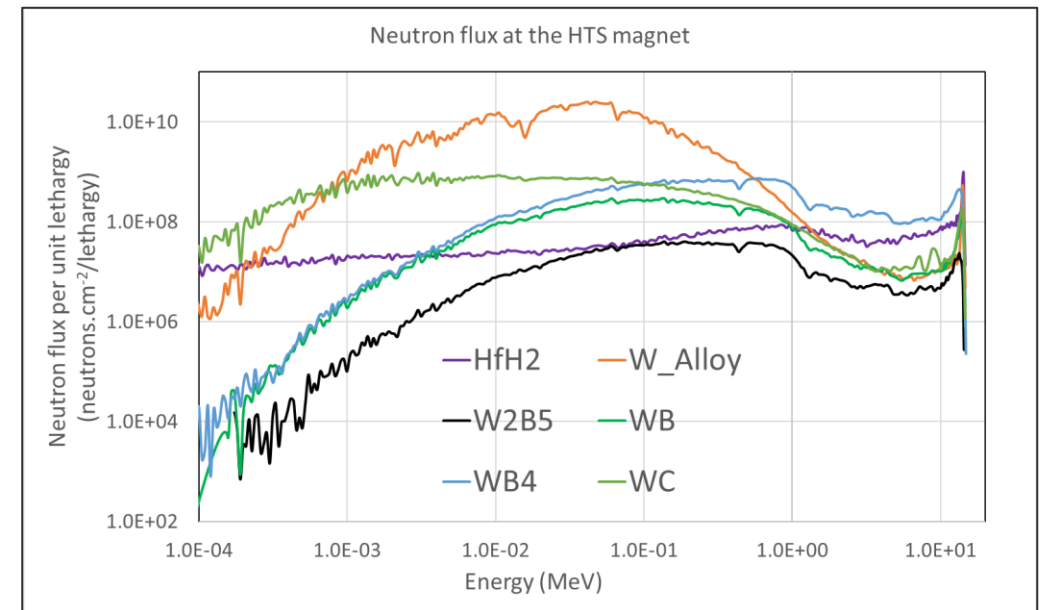
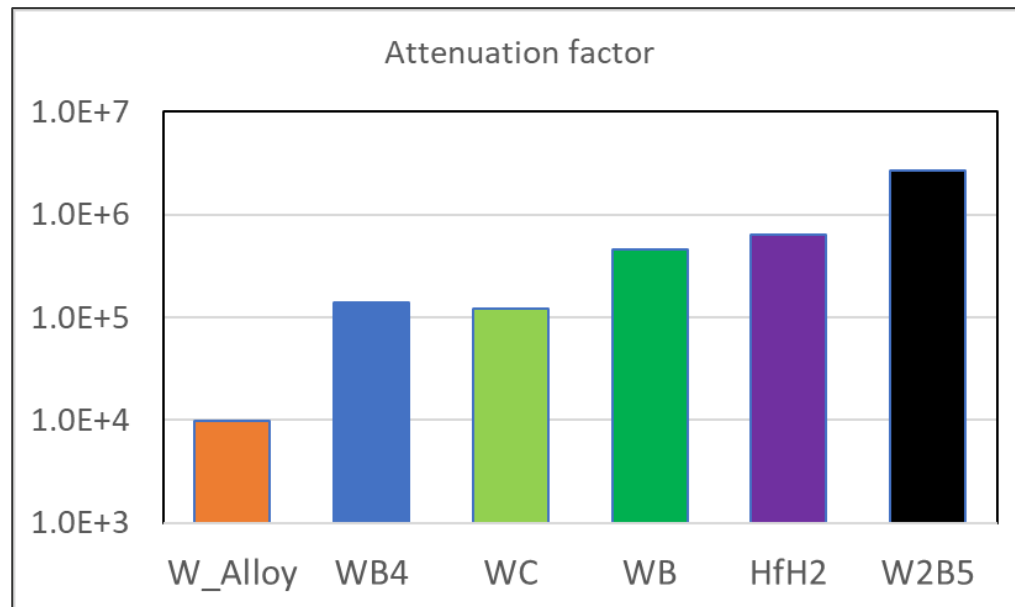


- The possibility of materials which contain one or more of the elements can increase drastically when scouting for shielding material candidates
- Material selection charts can assist in visually understanding the material types and their potential for a minimal thickness shield of REBCO
- Light blue shows metals, dark blue shows carbides, red shows oxides, green shows hydrides. Materials containing W (hollow circles), Hf (hollow squares) and Ta (pluses) are shown as points



# Our Advanced Shielding Materials Of Consideration

- Hydrides, carbides, and borides are shown to be good neutron shields for both, fast and slow neutrons.
- Tungsten boride's performance is superior when compared to the other advanced shield materials
- Neutron energies above 0.1 MeV are considered fast for the purposes of this presentation as these are considered the threshold to magnet degradation

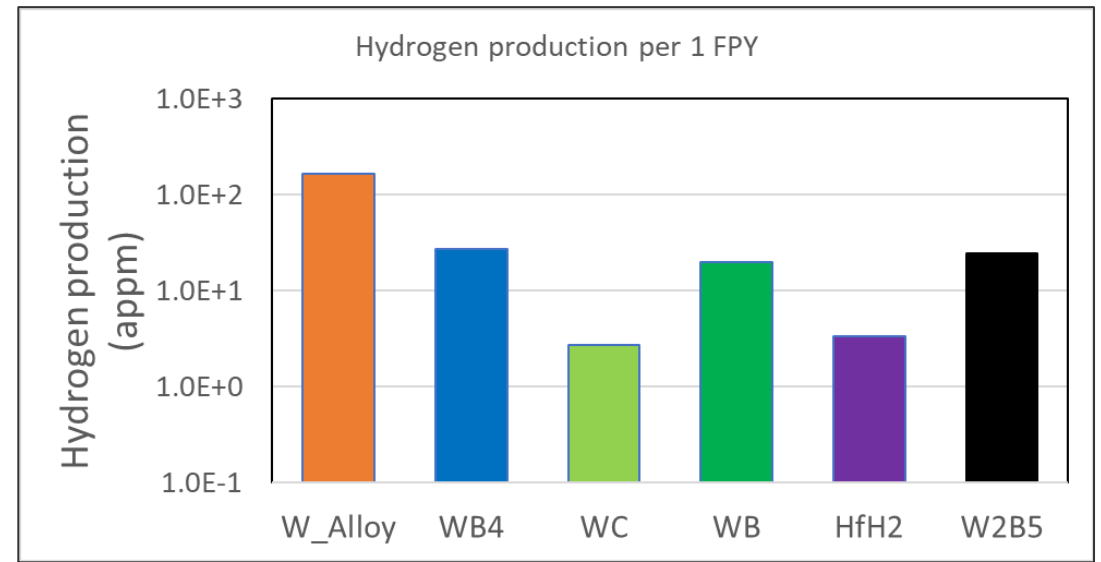
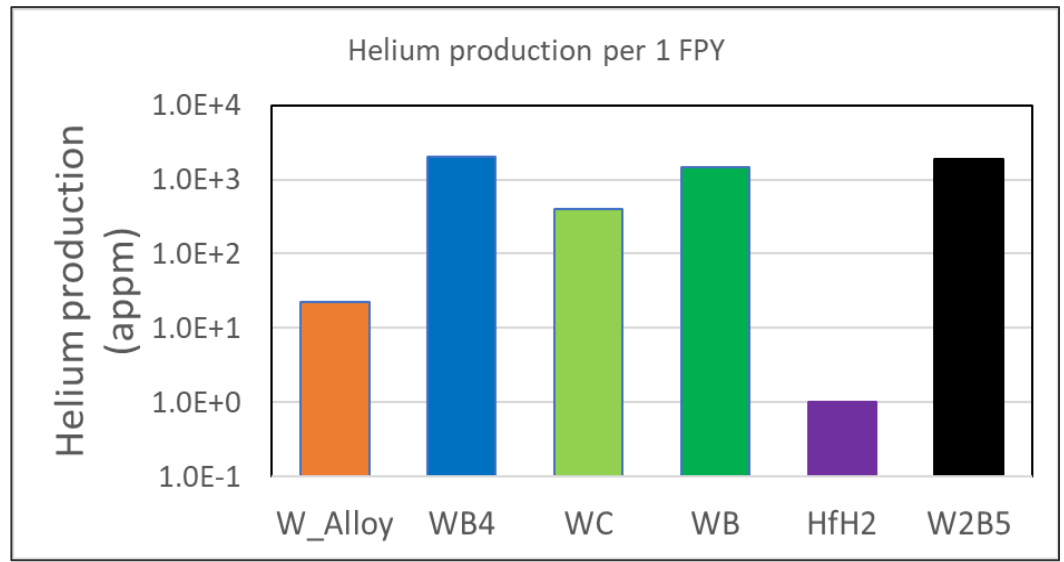


# Material Considerations In Advanced Shield Materials

Borides and Carbide produce high amounts of helium due to the alpha production reaction channel in Boron and carbon;



Tungsten alloy produces a large amount of hydrogen when compared against carbides and hydrides.





# Magnet Lifetime With These Advanced Materials

WC, WBs, and HfH2 come out as the preferred shielding solutions in monolithic form.

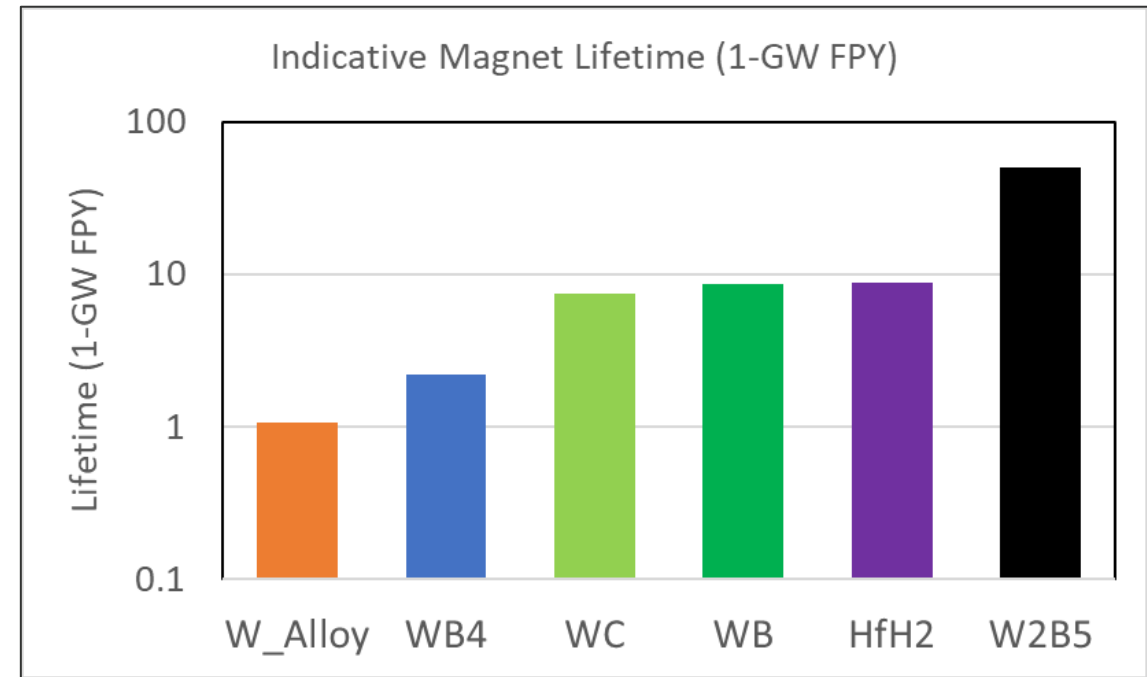
Some of these materials are at such low Technology Readiness Levels that significant work is needed for them to be realised.

Additional concerns of these materials are as follows:

Thermomechanical and structural compatibility of these materials at elevated temperatures

Thermomechanical and structural changes as a function of dose uptake.

Their compatibility with coolant materials



D X Fischer et al 2018 Supercond. Sci. Technol. 31 044006

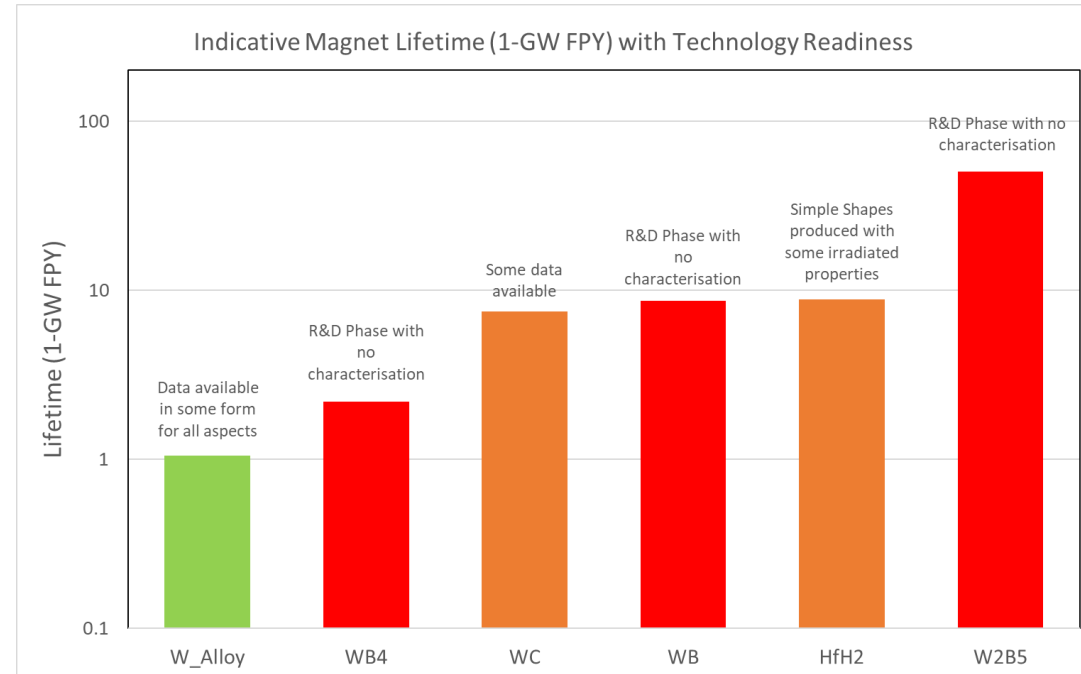


# Various Material Challenges To Overcome For These Materials

The performance of shielding materials is heavily dependent on the isotopic composition and density of the material, all of which can drastically change during the realisation process with our materials experts.

These considerations are:

- Possible tolerances in specific shape production
- Supply chain to support the tonnes of material needed
- Non-irradiated characteristics such as thermal conductivity and hardness
- Irradiated characteristics such as thermal expansion and swelling
- Transmutation and maintenance considerations



# Impact Of Alloying Materials On The Radioactive Waste Production

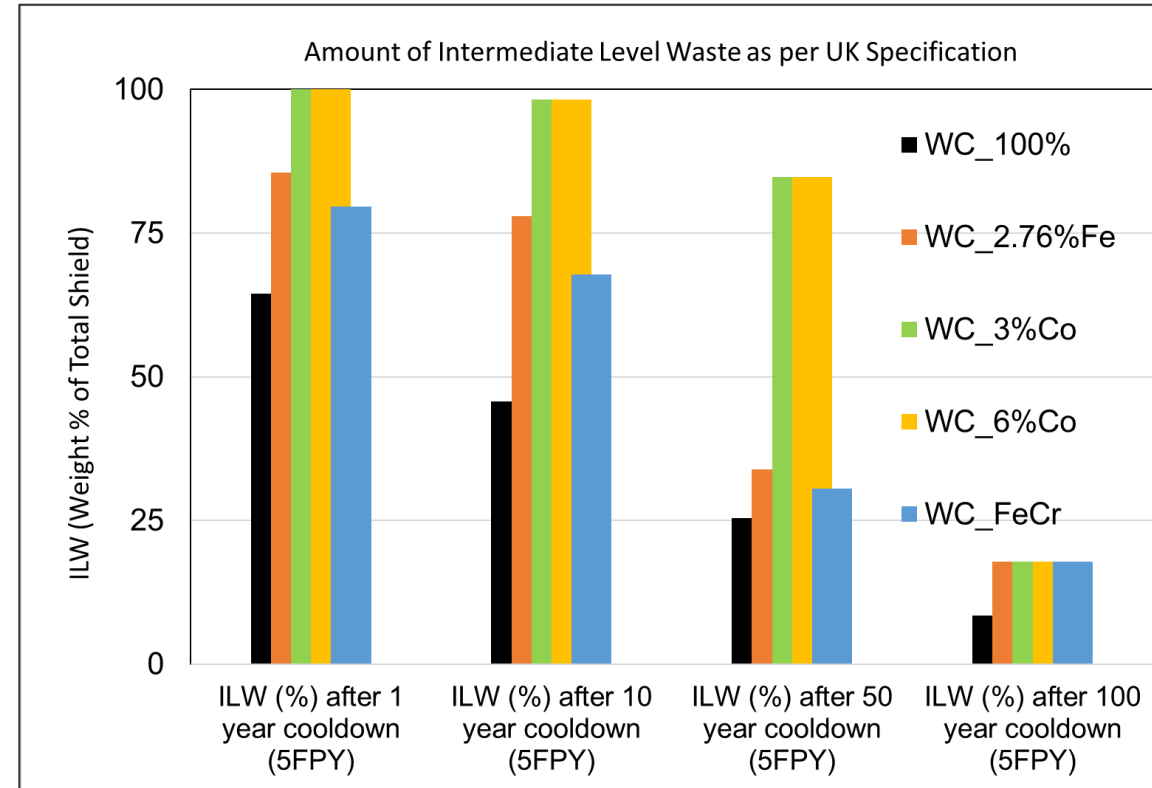
WC has a binder material to enhance its thermomechanical properties, but this causes additional concerns:

- Generation of radiological waste.
- Biological dose during maintenance.
- Degradation of binder following transmutation over time.

Radiological waste assessment of WC with different binders justifies avoiding Co as a binder.

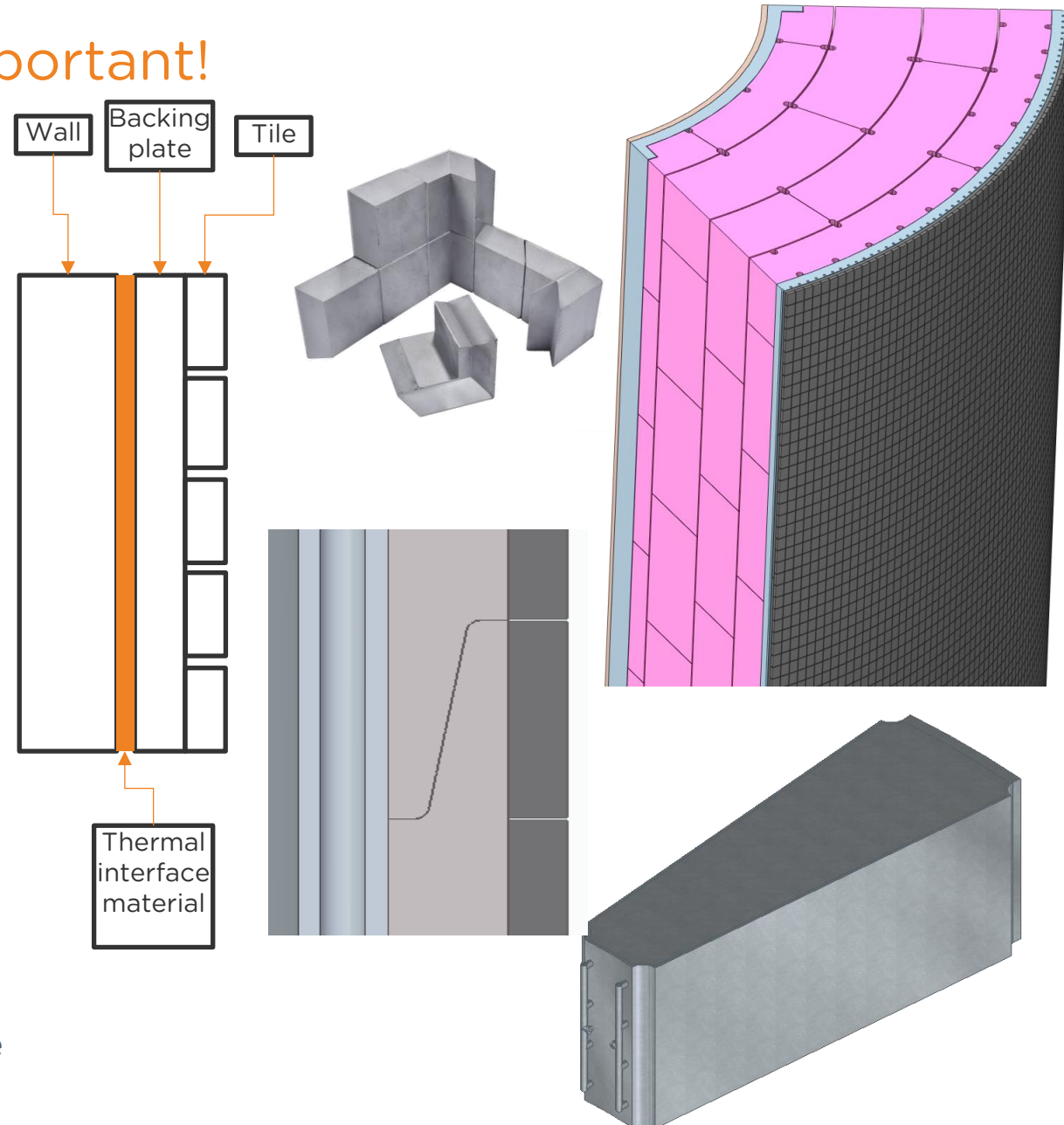
Binder-less Carbides would be preferable but at what cost?

Work is needed on the ALARP justifications for activated shielding materials.

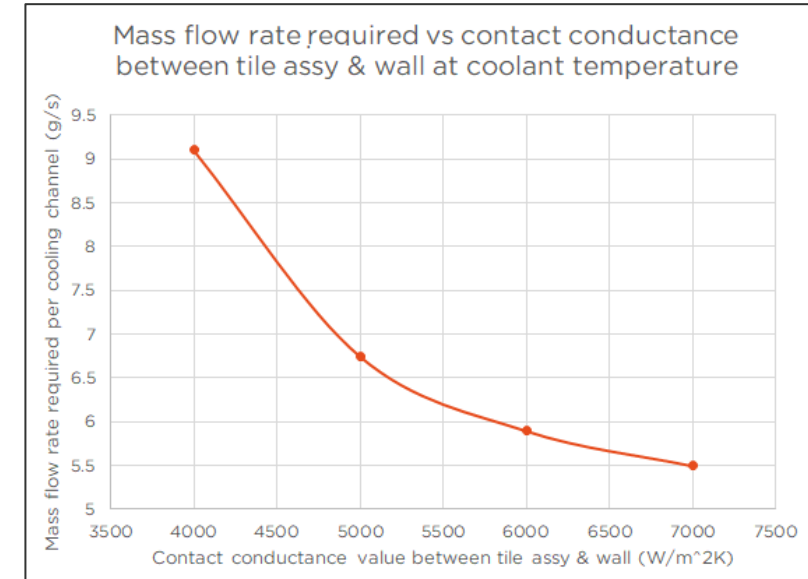
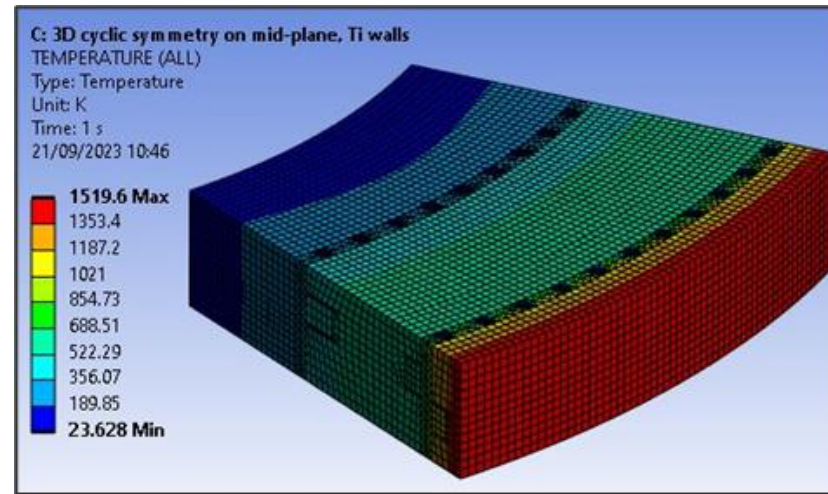
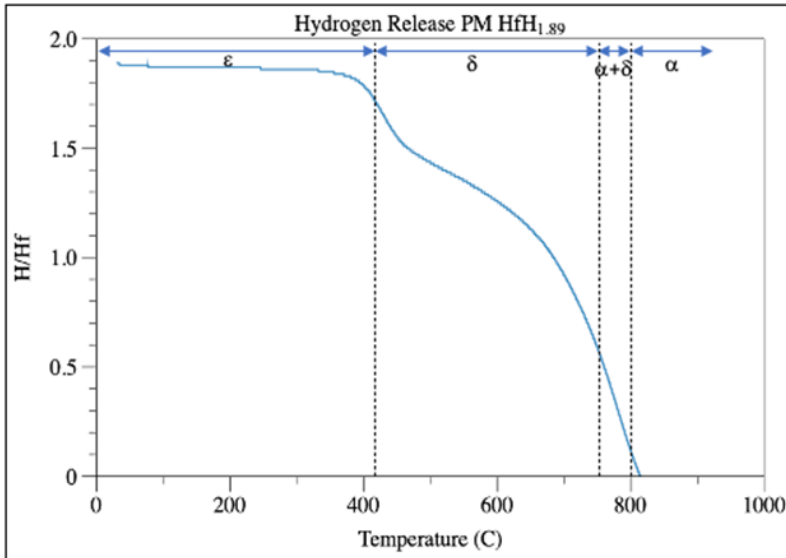


# Thermomechanical Properties Are Important!

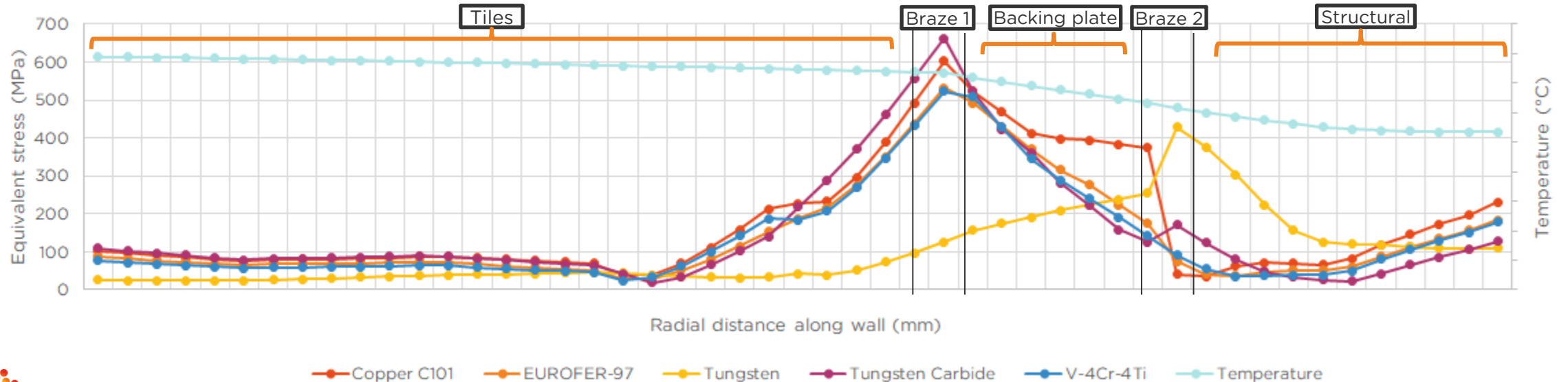
- The radiation shield will experience a volumetric heating load in the region of 10s of MW/m<sup>3</sup>.
- A high performing shield candidate will result in an exponential reduction in volumetric heating through its depth
- Cooling solutions will be required to ensure shielding materials remain within their operating window, but cooling pipes will degrade the shielding performance!
- Material Irradiation response will change as a function of temperature changing material properties in multiple ways
- High thermal conductivity will elevate some burden of cooling structures but the thermophysical properties need to be understood holistically. This will then produce an overall shielding solution.
- Properties like thermal expansion, coolant compatibility, strength will result in gaps reducing the overall volume of shielding material available



# Thermomechanical Properties Are Important!

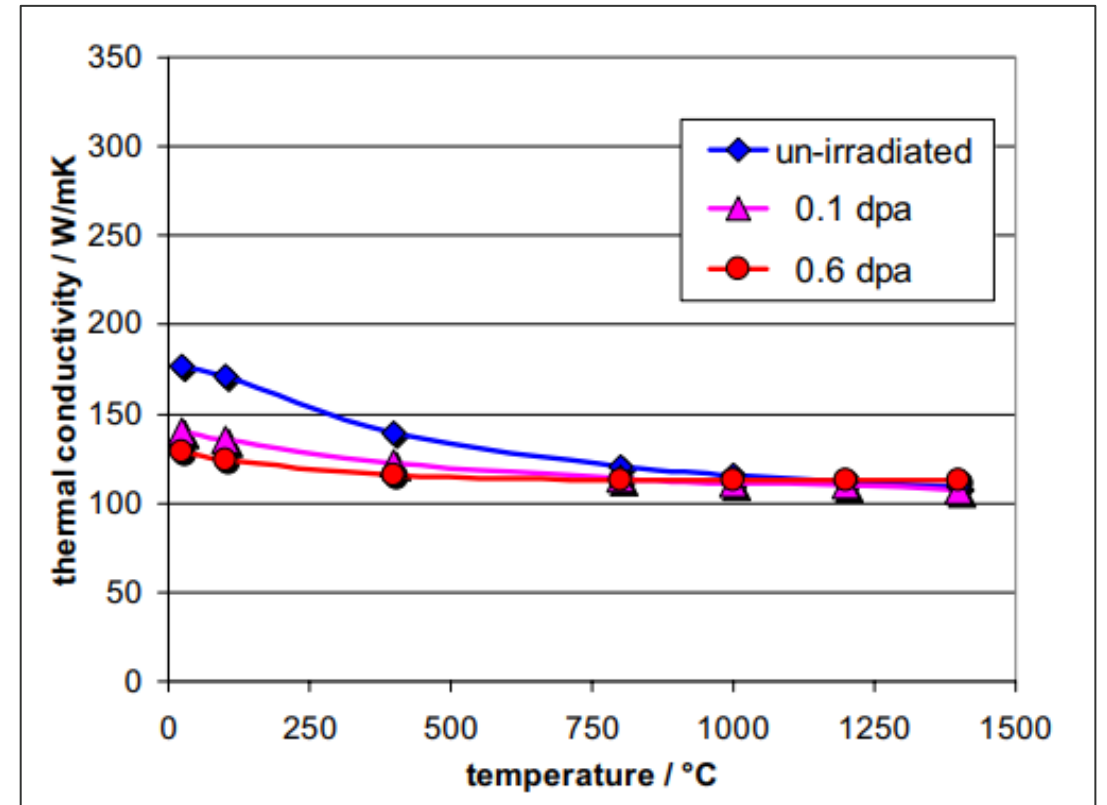


Backing plate material vs radial equivalent stress



# Thermomechanical Properties Are Important!

- To account for volumetric heating and structural loads, the shield material must have:
  - High strength to weight ratio
  - Good thermal conductivity
  - Good fatigue and creep rupture properties
  - Small modulus of elasticity
  - High electrical resistivity and heat capacity
  - Low coefficient of thermal expansion
  - A good compatibility with the coolant
  - High workability and good weldability
  - Commercial availability
  - Magnetic compatibility



M. Scheerer et al Physica Scripta T91, 98 (2001)

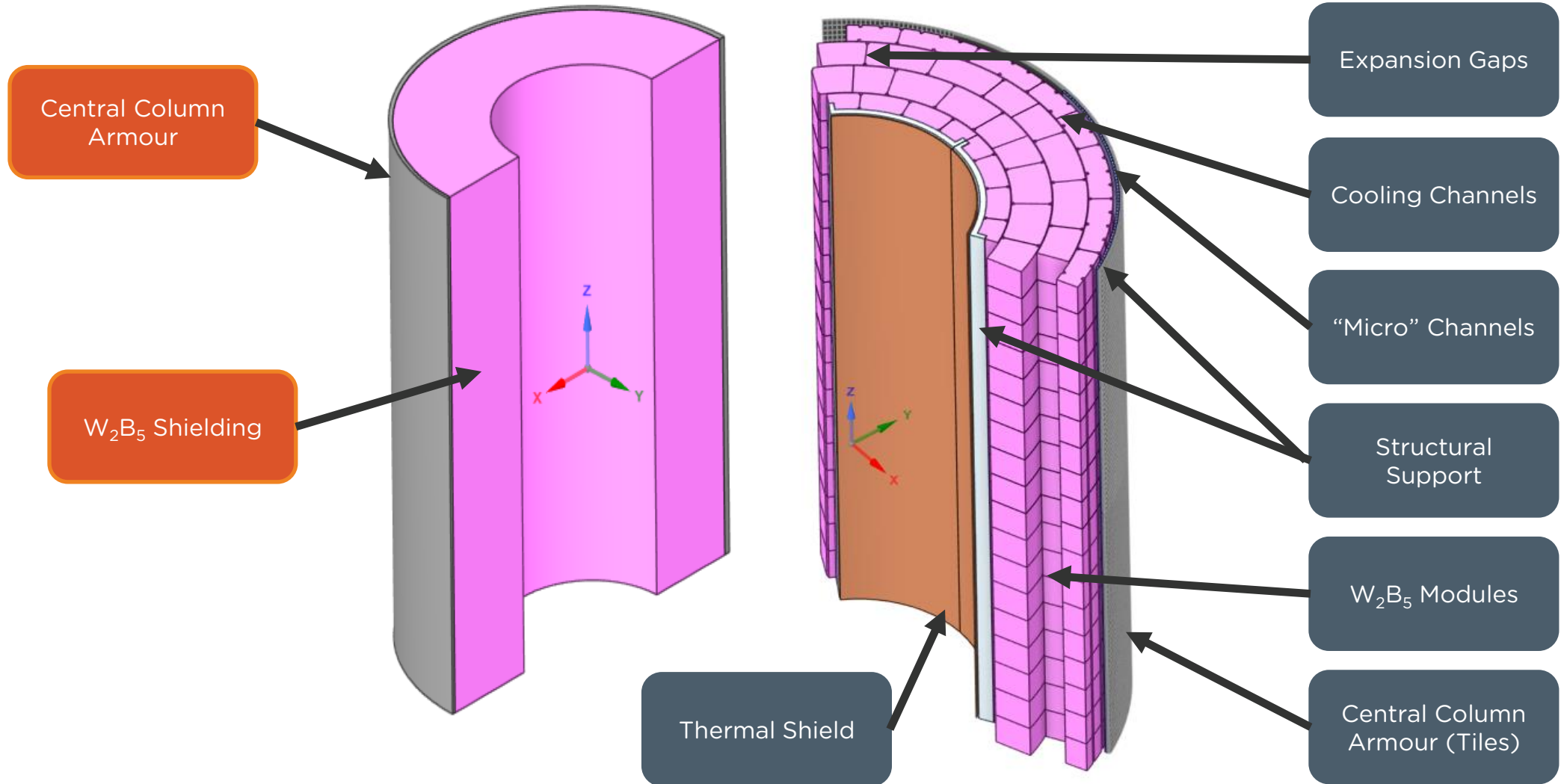
Thermal conductivity of Tungsten  
before and after irradiation



# Thermomechanical Properties Are Important!

Neutronics Pre concept Model  
Magnet Life = 80 FPY [1-GW<sub>f</sub>]

Multiphysics Concept Model  
Magnet Life = 12 FPY [1-GW<sub>f</sub>]





Thank you

CONTACT US

NAME



[Gurdeep.kamal@tokamakenergy.com](mailto:Gurdeep.kamal@tokamakenergy.com)

[www.tokamakenergy.com](http://www.tokamakenergy.com)



Tokamak Energy Ltd  
173 Brook Drive  
Milton Park  
Oxfordshire OX14 4SD  
United Kingdom

