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EUROPEAN ORGANIZATION FOR NUCLEAR RESEARCH

Proposal to the ISOLDE and Neutron Time-of-Flight Committee

The neutron capture cross section of $^{124}{\rm Sn}$ and its impact on neutrinoless double β decay searches

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Why do we need $^{124}Sn(n,\gamma)$ cross section data? Motivation A



Detector: Sn cryogenic superconducting bolometer (TIN.TIN -The India based TIN detector)

Nanal, Vandana.EPJ Web of Conferences, EDP Sciences 66 (2014)



Background assessment of Ονββ decay searches

- \Box ¹²⁴Sn is one of the promising $Ov\beta\beta$ candidates
- □ They measure the $O\nu\beta\beta$ decay peak which is at an energy equal to the Q-value of the reaction: 2292.7(4) keV
- □ Golden channel: decay rate <-> direct access to neutrino mass & CP violating Majorana phases (can <u>not</u> be probed by v oscillations experiments)
 - $0\nu\beta\beta$ is a second order weak interaction process and the event rates are very low $(T_{1/2} > 10^{17} \text{ y})^*$
- They are extremely sensitive to background signals which <u>can</u> <u>mimic</u> the signal of interest -> neutron-induced background in the Q-value region*

*Dawson, J., et al., Physical Review C 78.3 (2008): 035503 *Gupta, G., et al., Applied Radiation and Isotopes 158 (2020): 108923



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Detector : tin-loaded liquid scintillator for an active source-detector technique

Hwang, M. J., et al., Astroparticle Physics 31.6 (2009): 412-416.

Motivation A





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Background assessment of Ονββ decay experiments

ISSUE: γ rays following neutron capture on ¹²⁴Sn can mimic the $0\nu\beta\beta$ decay signal!!!

Literature* shows that, after activating a ¹²⁴Sn sample with a neutron thermal flux and measuring the delayed γ following neutron capture and subsequent β⁻ decays with a HPGe detector, a strong summing peak of 2288.2 keV has been seen*

 $Q_{0\nu\beta\beta} = 2292.7 \text{ keV}$

Also, a (worrying) 30% simulation vs. experiment difference was observed*

*Gupta, G., et al., Applied Radiation and Isotopes 158 (2020): 108923

Motivation A





Why do we need $^{124}Sn(n,\gamma)$ cross section data? Motivation B

Problem in reactor fuel depletion calculations for ¹²⁵Sb:

Annals of Nuclear Energy 161 (2021) 108441

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lournal homepage: www.elsevier.com/locate/anucene

Review

Monte Carlo neutronics benchmarks on nuclear fuel depletion: A review

Sean P. Martinson, Sunil S. Chirayath * Department of Nuclear Engineering. Texas A&M University, 3133 TAMU, College Station, TX 77843-3133, United State

ABSTRACT

Monte Carlo (MC) neutronics codes are used widely for academic and industrial needs. Several schemes of coupling MC neutronics code with isotope generation and depletion code exist, which are used for performing nuclear fuel depletion simulations. These simulations can estimate the inventory of isotopes in neutron irradiated nuclear reactor fuel. However, the accuracy of these simulations shall be validated through experiments. MC codes are seldom validated by isotopic benchmarks compared to criticality benchmarks. This work compiles and analyzes the fuel depletion benchmarks and validations used to analyze the performance of MC-based fuel depletion neutronics codes. Analyses of these benchmarks and validations showed that the computed concentrations of ¹³³Cs, ¹³⁵Cs, ¹³⁷Cs, ¹⁴⁸Nd, ²³⁹Pu, ²⁴⁰Pu, and ²⁴¹Pu in the irradiated fuel by the depletion codes agreed with the measured values within 10% error. However, the computed concentrations of ¹²⁵Sb, ²⁴²Cm, ²⁴³Cm, ²⁴⁴Cm, ²⁴⁵Cm, and ²⁴⁶Cm had errors more than 15% compared to the measured values. Ventina depletion code showed the most accurate predictions for the greatest number of isotope concentrations compared to ORIGEN2 and CINDER90.

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NUREG/CR-6798 ORNL/TM-2001/259

Motivation **B**

Isotopic Analysis of High-Burnup PWR **Spent Fuel Samples From** the Takahama-3 Reactor

Section 3

Manuscript Completed: May 2002 Date Published: January 2003

Results

Table 12 Comparison	f analyses and	calculations	of Takahama-3 SF95-3
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	and the second second	Me	asured ^a g/MgU		
	SF95-3	SF95-3	SF95-3	C/E	C/E
Nuclide	Measured	SAS2H	HELIOS	SAS2H	HELIOS
234U	1.873E+02	2.401E+02	1.92E+02	1.28	1.02
²³⁵ U	1.326E+04	1.299E+04	1.36E+04	0.98	1.02
²³⁶ U	4.911E+03	4.904E+03	4.95E+03	1.00	1.01
²³⁸ U	9.338E+05	9.340E+05	9.34E+05	1.00	1.00
²³⁸ Pu	1.539E+02	1.491E+02	1.53E+02	0.97	1.00
²³⁹ Pu	6.194E+03	6.043E+03	6.54E+03	0.98	1.06
²⁴⁰ Pu	2.186E+03	2.219E+03	2.21E+03	1.02	1.01
²⁴¹ Pu	1.486E+03	1.413E+03	1.51E+03	0.95	1.01
²⁴² Pu	4.516E+02	4.571E+02	4.16E+02	1.01	0.92
²⁴¹ Am	3.310E+01	3.732E+01	3.93E+01	1.13	1.19
^{242m} Am	7.877E-01	8.130E-01	8.14E-01	1.03	1.03
243Am	8.047E+01	9.360E+01	7.98E+01	1.16	0.99
²⁴² Cm	1.964E+01	1.178E+01	1.64E+01	0.60	0.83
²⁴³ Cm	3.720E-01	3.006E-01	3.02E-01	0.81	0.81
244Cm	2.562E+01	2.507E+01	2.36E+01	0.98	0.92
245Cm	1.396E+00	8.783E-01	1.30E+00	0.63	0.93
²⁴⁶ Cm	1.049E-01	9.553E-02	8.97E-02	0.91	0.86
137Cs	1.347E+03	1.338E+03	1.31E+03	0.99	0.97
134Cs	1.404E+02	1.204E+02	1.06E+02	0.86	0.76

Burnup	35.42				
¹⁵⁰ Nd	1.896E+02	1.866E+02	1.87E+02	0.98	0.99
148Nd	3.979E+02	3.946E+02	3.97E+02	0.99	1.00
146Nd	7.340E+02	7.418E+02	7.34E+02	1.01	1.00
¹⁴⁵ Nd	7.392E+02	7.395E+02	7.35E+02	1.00	0.99
¹⁴⁴ Nd	9.347E+02	9.457E+02	9.29E+02	1.01	0.99
¹⁴³ Nd	9.299E+02	9.046E+02	9.10E+02	0.97	0.98
¹⁴² Nd	2.116E+01	1.802E+01	N/A	0.85	N/A
¹⁰⁶ Ru	1.360E+02	1.713E+02	1.67E+02	1.26	1.23
¹²⁵ Sb	3.733E+00	7.952E+00	1.01E+01	2.13	2.69
144Ce	4.560E+02	4.359E+02	4.22E+02	0.96	0.92
¹⁵⁴ Eu	2.525E+01	2.476E+01	2.76E+01	0.98	1.09
134Cs	1.404E+02	1.204E+02	1.06E+02	0.86	0.76
¹³⁷ Cs	1.347E+03	1.338E+03	1.31E+03	0.99	0.97
²⁴⁶ Cm	1.049E-01	9.553E-02	8.97E-02	0.91	0.86
²⁴⁵ Cm	1.396E+00	8.783E-01	1.30E+00	0.63	0.93
²⁴⁴ Cm	2.562E+01	2.507E+01	2.36E+01	0.98	0.92
²⁴³ Cm	3.720E01	3.006E-01	3.02E-01	0.81	0.81
²⁴² Cm	1.964E+01	1.178E+01	1.64E+01	0.60	0.83
²⁴³ Am	8.047E+01	9.360E+01	7.98E+01	1.16	0.99
^{242m} Am	7.877E-01	8.130E-01	8.14E-01	1.03	1.03
²⁴¹ Am	3.310E+01	3.732E+01	3.93E+01	1.13	1.19

^a At discharge, except for ²³⁹Pu which includes contribution from ²³⁹Np precursor. ^b Burnup estimated using ¹⁴⁸Nd analysis.



Motivation **B**

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⊗ANS

Experimental and Computational Forensics Characterization of Weapons-Grade Plutonium Produced in a Fast Reactor Neutron Environment

The final step in this part of the investigation was to compare the quantities of the various fission products predicted from the MCNPX burnup simulation to those measured using gamma spectroscopy. Both the simulated and measured values were normalized to the mass of DUO_2 in order to account for differences in the simulated mass and the mass of the actual sample. These comparative data are shown in Table V. As can be seen in Table V, the difference between the simulated and measured values for most of the isotopes is equal to or less than 12%; however, the activity predicted for ¹²⁵Sb was over 50% larger than the measured activity. Upon further investigation, it was discovered that ¹²⁵Sb is a particularly troublesome nuclide to

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 $^{a}S/E = simulation/measurement.$

Comparison of Gamma Spectroscopy Measurements to Simulation

	Measured Activity (Ci/g DUO ₂)	Standard Deviation (Ci/g DUO ₂)	Simulation Activity (Ci/g DUO ₂)	S/E ^a
¹⁴⁴ Ce	9.60E-02	2.29E-03	8.43E-02	$\begin{array}{c} 0.88 \pm 0.09 \\ 1.10 \pm 0.11 \\ 0.94 \pm 0.10 \\ 1.11 \pm 0.13 \\ 1.51 \pm 0.16 \\ 0.92 \pm 0.10 \end{array}$
¹³⁴ Cs	2.01E-03	5.24E-05	2.21E-03	
¹³⁷ Cs	1.41E-02	2.32E-04	1.32E-02	
¹⁵⁴ Eu	1.80E-04	1.12E-05	2.00E-04	
¹²⁵ Sb	1.16E-03	4.63E-05	1.75E-03	
⁹⁵ Zr	6.53E-03	1.96E-04	6.00E-03	

Motivation **B**



Figure: Burnup chain adjacent to 125Sb



Neutron Environment

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Present status of $^{124}Sn(n,\gamma)$ cross section data



A. Kimura et al., EPJ Web of Conferences 146, 11031 (2017) in ANNRI at MLF/J-PARC (ND2016 proceedings)



Cross Section (barns)

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Elastic to capture ratio -> very unfavorable!!!

Neutron energy resolution is not paramount -> AVERAGE CROSS SECTION (above 20 keV)



Fig. 15. Comparison between the evaluated neutron flux in EAR2 (blue) and in EAR1 (red). The increase at the new measuring station is on average a factor 40.

 Table: Average distance in-between resonances

 against energy resolution at EAR1 versus EAR2

Neutron Energy range	No. of resonances (ENDF/B- VIII.0)	Avg. distance between Capture Resonances	Energy resolution in EAR1 *	Energy resolution in EAR2 *	
50 eV-10 keV	7	1200 eV	10 eV	200 eV	
10-100 keV	67	1300 eV	300 eV	3000 eV	
100-200 keV	68	1500 eV	800 eV	-	
200-314 keV	47	2400 eV	1200 eV	-	

Goal: resolve resonances up to 15-20 keV

*Guerrero, C., et al. The European Physical Journal A 49 (2013): 1-15.

*Lerendegui-Marco, J., et al. The European Physical Journal A 52 (2016): 1-10.



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Enriched sample -> paramount!







Enriched ¹²⁴Sn



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Sample(s) Details

- 3 disks (1 g each)
- 97.9% enrichment
- 10 mm diameter
- 1.8 mm thickness

- 25 g (natural Tin rod)
- 99.999% Purity
- 13 mm diameter







Available @ IFIN-HH Target lab

Available @ Nuclear Physics Institute Czech Academy of Sciences (I. Tomandl, F. Marek)

Resolution function and multiple scattering -> SAMMY-based calculations

In EAR2 : 1.0 g sample (0.00618 at/b)

In EAR2 : 0.1 g sample (0.000618 at/b)



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How will we measure?

Setup of 9 sTED detectors

- Measure from thermal to highest reachable energy by these detectors in EAR2
- Use an enriched sample: 97.9% of ¹²⁴Sn
- Thin-thick approach:
 - Up to 15-20 keV-> thick (1.0 g)
 - Average xs above 20 keV-> thick (3.0 g)
 - First resonance -> thin (0.1 g)

Ancillary: also irradiate ^{nat}Sn, ¹⁹⁷Au, ^{nat}C, ^{nat}Pb samples + Empty

Experimental Details [EAR2]



The 9 sTED's setup used in EAR2 for the $^{209}\mbox{Bi}(n,\gamma)$ campaign



Counts estimation:

> 1.0 g sample, 0.00618 at/b (10 mm-diameter)

Proton Request [EAR2]

> 0.1 g sample, 0.000618 at/b (10 mm-diameter)



 $\frac{\text{Total background}}{\text{Total Counts}} = \text{empty+in-beam } \gamma + \text{elastic}$ $\frac{\text{Total Counts}}{\text{Total Counts}} = 124 \text{Sn}(n,g) - 0.1/1g + \text{total background}$



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Proton Request [EAR2]





Proton Request [EAR2]

Counts estimation:

> 3.0 g sample, 0.01856 at/b (10 mm-diameter)



Eff-4.5%, JEFF-3.3 [9 sTEDs]



Eff-13.5%, JEFF-3.3 [27 sTEDs]





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Proton Request [EAR2]

For ¹²⁴Sn (1 g sample) -> <u>6.5x10¹⁷ protons</u>

For ¹²⁴Sn (0.1 g sample) -> <u>1.1x10¹⁷ protons</u>

For ¹²⁴Sn (3 g sample) -> <u>4.8x10¹⁷ protons</u>

For ^{nat}Sn -> <u>1.1x10¹⁷ protons</u>

Ancillary: normalisation (197Au) + background estimation (natPb, natC, Empty) -> 6.5x1017 protons

In total: 2.0x10¹⁸ protons



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Motivation:

A: Neutrinoless double β decay searches
 B: Nuclear fuel depletion calculations: ¹²⁵Sb problem

Status of data:

✓ No ToF neutron capture data exist to map out first resonances below 10 keV, questionable resonances...

Experiment goals:

✓ To provide for the first time reliable, low uncertainty neutron capture ToF data from thermal to 15-20 keV -> resonance parameters for the most intense resonances

✓ Possibly average cross section above 20 keV

Impact:

 To better quantify the neutron-induced background for neutrinoless double β decay (Ονββ) searches

✓ Optimistically, to at least partially clarify the differences between various evaluations -> improve on ¹²⁵Sb problem

first resonances below Proton request Sample 124Sn (1 g) 124Sn (0.1 g) 124Sn (3 0 g) 124Sn (3 0 g)

1.1x10 ¹⁷
4.8x10 ¹⁷
1.1x10 ¹⁷



Summary

Protons

6.5x10¹⁷

Thanks

Do you have any questions?

https://www.nipne.ro/proiecte/pn3/ntof / https://www.nipne.ro/

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