

***Polarized Fusion and its Implications:  
the Potential for Direct in situ Measurements of  
Fuel Polarization Survival in a Tokamak Plasma***

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*(for the Jlab-GA SPF Collaboration)*

*(Peking University, Beijing – October 21, 2014)*

# *potential for running a Tokamak with polarized fuel*

- ***fusion fuels:***  $D + t \Rightarrow \alpha + n$ ;  $D + {}^3\text{He} \Rightarrow \alpha + p$ 
  - ↔ *both dominated by  $J=3/2$  resonance just above threshold*
  - ↔ *ion temperatures  $< 100$  KeV  $\Rightarrow$  s-waves dominate*
  - ↔ *D ( $s=1$ ) and t ( $s=1/2$ ) preferentially fuse when spins are aligned*
- ***polarized fuels:*** *up to 50% enhancement in the cross section*
  - ↔ *obvious benefits: boost to "ignition"; increased power,  $Q(\text{eff})$*
  - ↔ *iff polarizations survive in the plasma (for  $\sim$  few sec) ???*
- **History**
  - Kulsrud, Furth, Valeo & Goldhaber, Phys Rev Lett **49** (82) 1248
  - Lodder, Phys. Lett. A98 (83) 179
  - Greenside, Budny & Post, J Vac Sci & Technology A2 (84)
  - Coppi, Cowley, Kulsrud, Detragiache & Pegoraro, Phys Fluids **29** (86) 4060
  - Kulsrud, Valeo & Cowley, Nucl Fusion **26** (86) 1443
  - Cowley, Kulsrud, Valeo, E.J. Phys. Fluids 29 (86) 1443
  - ...

# *JLab-GA SPF (Spin-Polarized-Fusion) Collaboration*

- **Jefferson Lab**

HDice:

A.M. Sandorfi, A. Deur, T. Kageya, M. Lowry, X. Wei

*HD*

- University of Virginia:

X. Zheng, W.A. Tobias

*<sup>3</sup>He*

- **General Atomics/Fusion Energy Research**

GA-DIII-D:

G. Jackson, N. Eidietis, A. Hyatt, M. Lanctot, D. Pace, S. Smith, H. St-John  
*plasma, orbit & transport simulation*

GA-ICF Pellet Division:

M. Farrell, M. Hoppe, A. Nikroo

*ICF pellets for HD, <sup>3</sup>He*

- Oak Ridge National Lab:

L. Baylor

*cryo-injection guns*

- UC-Irvine:

W.W. Heidbrink

*fast particle detection*

# Potential Power gain from polarization in a large Tokamak

- $d+t \rightarrow \alpha+n$  transport Modeling – Sterling Smith, GA

- Transport equation:

$$\frac{d(Temp)}{d(time)} + \nabla \cdot \Gamma(heat\ flux) = S(source)$$

$$= reaction\ rate \times \alpha\ heating \times spin\ enhancement$$

$$= N_D N_t \langle \sigma V(T) \rangle \times (E_\alpha = 3.5\ MeV) \times \mathbf{F}$$

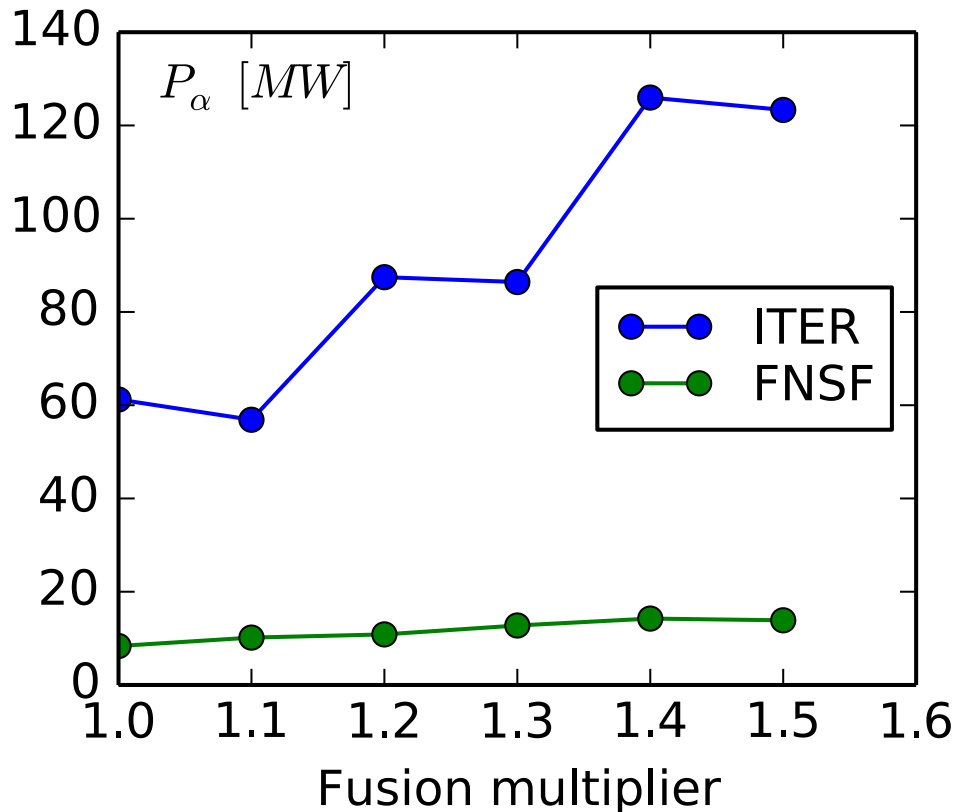


- Flux:  $\Gamma = \Gamma_{neoClassical} + \Gamma_{turbulent}$  , *polarization factor*  $\in [1, 1.5]$

- simulation codes: **NEO** **TGLF** **TGYRO**

- steady-state flux matched to source:  $\Gamma_{neoC} + \Gamma_{turb} = \int S dV$   
 $f(N, \partial N, T, \partial T)$

# ITER and FNSF total fusion power output increases with fusion multiplier



Alpha heating vs fusion multiplier;  
multiply by 5 to get total fusion power

- **The ITER gain is greater than the fusion factor**
  - 60 MW to 120 MW for 50% increase in fusion factor
- **FNSF gain is also greater, but not by as much**
  - 8 MW to 14 MW

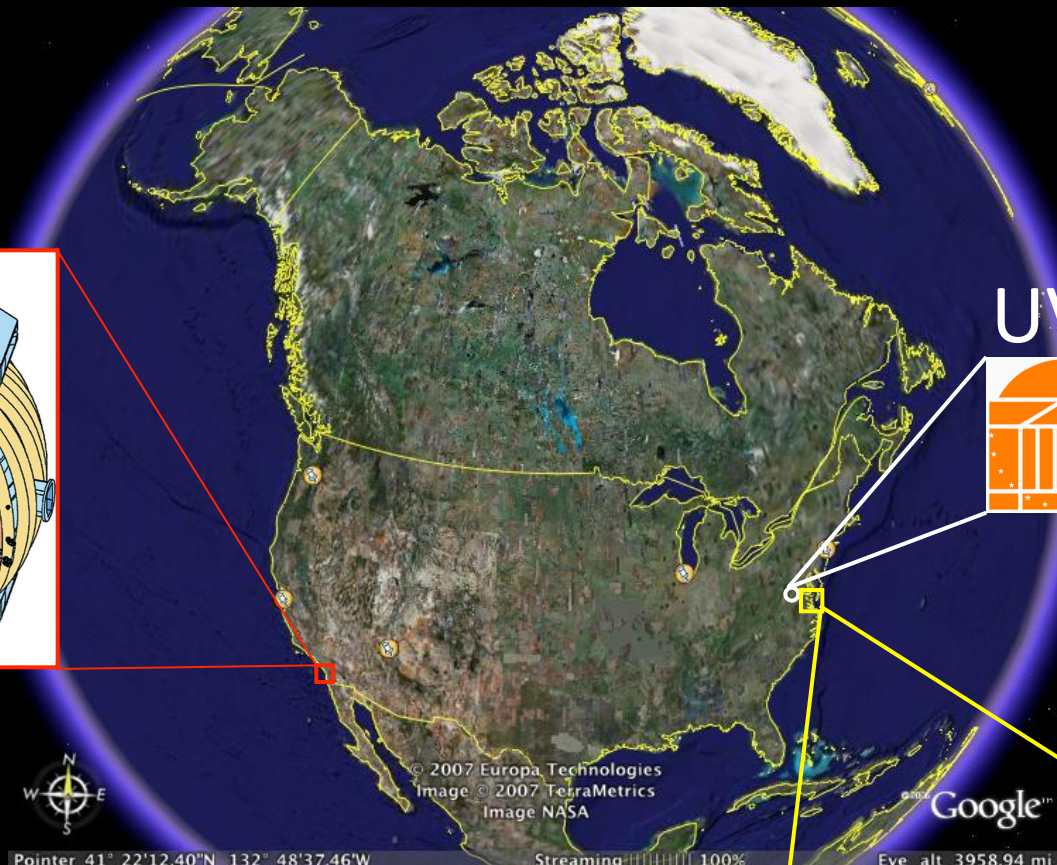
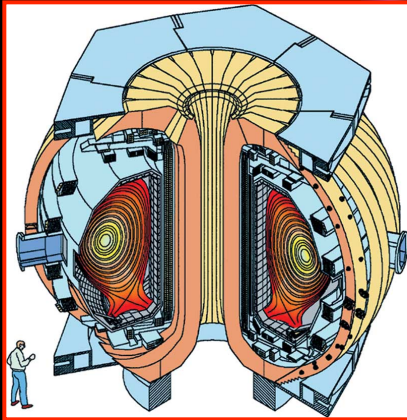
• net ITER gain is a factor of 2 in power

## *the path to polarized fuelling*

- over past ~3 decades NP and HEP have developed polarized materials, but with very different goals:  $10^6 - 10^8$  s lifetimes in few mole samples
- Fusion energy would need only 1-10 s lifetimes, but K-moles/day
  - ⇒ *simply scaling up NP/HEP production techniques is not appropriate*
  - ⇒ *new R&D would be required*
  - ⇒ *MUST demonstrate polarization survival before such an investment*

# Polarization survival test in the DIII-D Tokamak

**DIII-D**



UVa



Pointer 41° 22'12.40"N 132° 48'37.46"W

© 2007 Europa Technologies  
Image © 2007 TerraMetrics  
Image NASA

Streaming 100%

Google™

Eye alt 3958.94 mi

Strategy: use existing NP techniques and equipment to create polarization life-times sufficient to produce fuel for a test at DIII-D, which mitigates costs.



**Jefferson Lab**

# Measurement strategy - overview

- test reaction:  $\vec{D} + {}^3\vec{He} \rightarrow \alpha + p$  {mirror reaction to  $D + t \rightarrow \alpha + n$ }
- use existing JLab facilities to create solid  $H\vec{D}$ ; ship to Tokamak
  - ➔ diffuse 200-400 atm  $HD$  into ICF shells; cool to solid; polarize  $H$  and  $D$ ;  $H \Leftrightarrow D$  spin transfer to maximize  $D$  spin; transport to DIII-D Tokamak; load into cryo-gun
- use existing UVa facilities to polarize  ${}^3\vec{He}$  gas ( $\sim 10$ -30 atm)
  - ➔ remove polarizing agents, diffuse into ICF shells; cool to seal; load into DIII-D cryo-gun
- generate  $H$  (or  ${}^4He$ ) plasma in the DIII-D Tokamak
  - ➔ inject polarized fuel into plasma, alternating alignment:
 

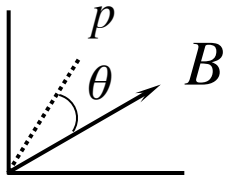
parallel:	$H\vec{D} \uparrow + {}^3\vec{He} \uparrow$	}	$\Leftrightarrow$ compare proton yields
anti-parallel:	$H\vec{D} \downarrow + {}^3\vec{He} \uparrow$		



# *A look at the experiment in a little detail*

- polarization-dependent angular distributions
- preparation of polarized HD and polarized  $^3\text{He}$
- ICF pellets and permeation of fuel material
- the DIII-D Tokamak and pellet injection
- Tokamak geometry and simulations of expected orbits in DIII-D
- expected reaction rates in a DIII-D test experiment
- secondary reactions
- particle detection: eg. Fast-Ion-Loss Detectors
- approximate time scale

# spin-dependent ${}^3\text{He}+D \rightarrow \alpha+p$ (or $t+D \rightarrow \alpha+n$ ) angular distributions



- angles relative to the magnetic field direction

- neglecting interference terms (good to  $\sim 2-3\%$ )

$$\frac{d\sigma}{d\Omega_{cm}} = \left( \frac{d\sigma}{d\Omega} \right)_0 \left\{ 1 - \frac{1}{2} P_D^V P_{3He} + \frac{1}{2} \left[ 3P_D^V P_{3He} \sin^2 \theta + \frac{1}{2} P_D^T (1 - 3\cos^2 \theta) \right] \right\}$$

→ angle integrated cross section :

$$\sigma_{cm} = \sigma_0 \left\{ 1 + \frac{1}{2} \vec{P}_D^V \cdot \vec{P}_{3He} \right\}, \quad \sigma_0 = \frac{4\pi}{(k_D^{cm})^2} \sum \frac{(2J_{5Li} + 1)}{(2I_D + 1)(2I_{3He} + 1)} \left| {}^{J\pi} T_{(2S_i+1)L_i, (2S_f+1)L_f} \right|^2$$



50% cross section gain for 100% parallel spin alignment

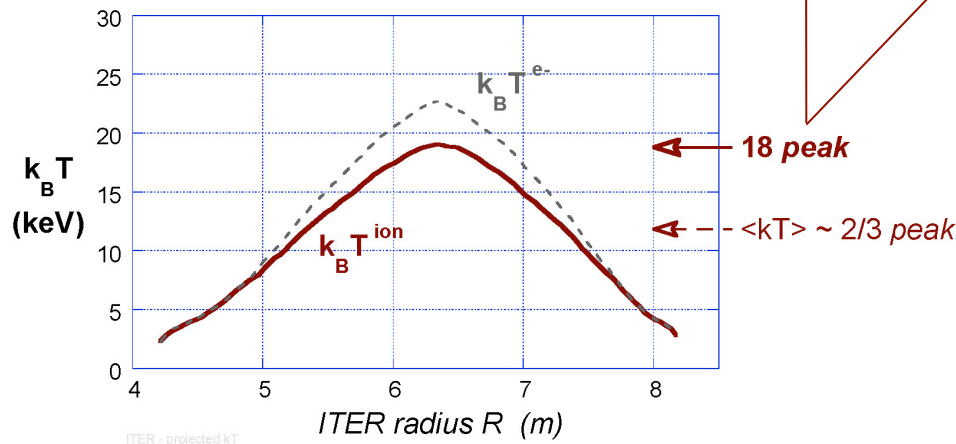
# reaction rate in a heated plasma

Cross sections are averaged over a Maxwellian Velocity distribution

$$\langle \sigma V \rangle = \frac{4c}{\sqrt{2\pi M_r} (k_B T)^{3/2}} \int e^{-\epsilon/k_B T} \epsilon \sigma(\epsilon) d\epsilon$$

eg: ITER

ITER Design Report, Plasma Physics and Controlled Fusion 44 (2002) 519



eg: DIII-D,

nominal

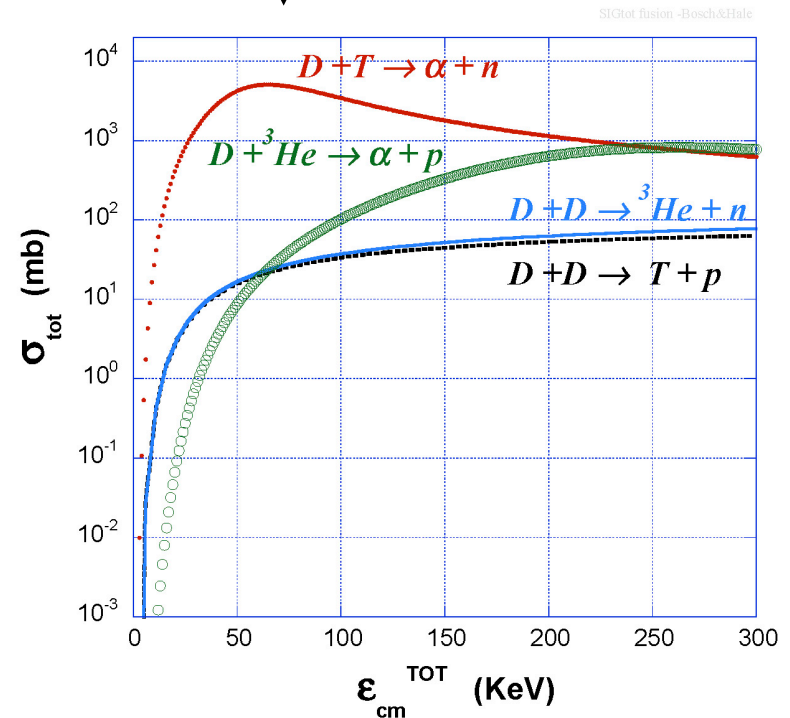
reduced density

kT(peak) ~ 5 keV

(9-13 keV)

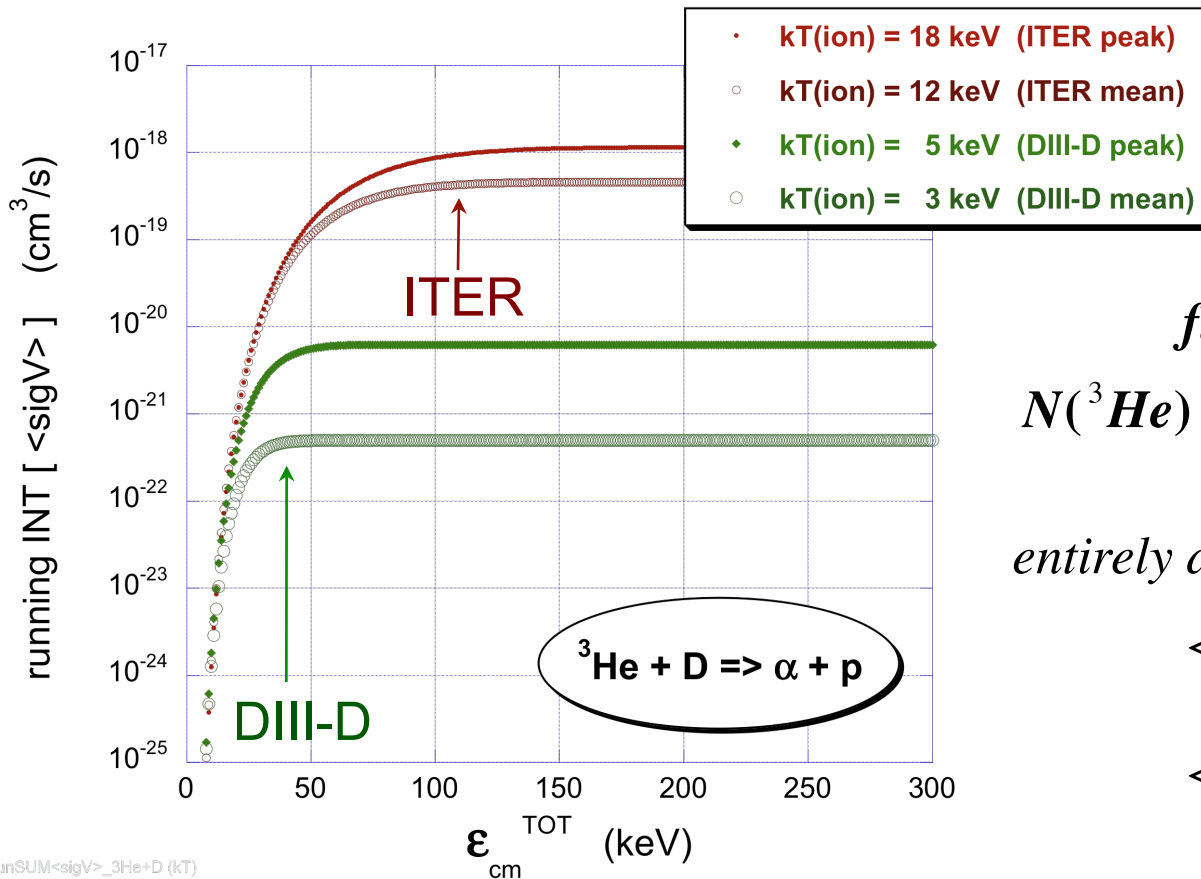
kT(mean) ~ 3 keV

(6-8 keV)



# relevant energy range

$$\text{running int} \langle \sigma V \rangle = \frac{4c}{\sqrt{2\pi M_r} (k_B T)^{3/2}} \int_0^{\epsilon_{cm}^{tot}} e^{-\epsilon/k_B T} \epsilon \sigma(\epsilon) d\epsilon$$



*fusion rate* ( $s^{-1}$ ):

$$N({}^3\text{He}) \cdot N(\text{D}) \cdot \langle \sigma V \rangle \cdot \text{Vol}(m^3)$$

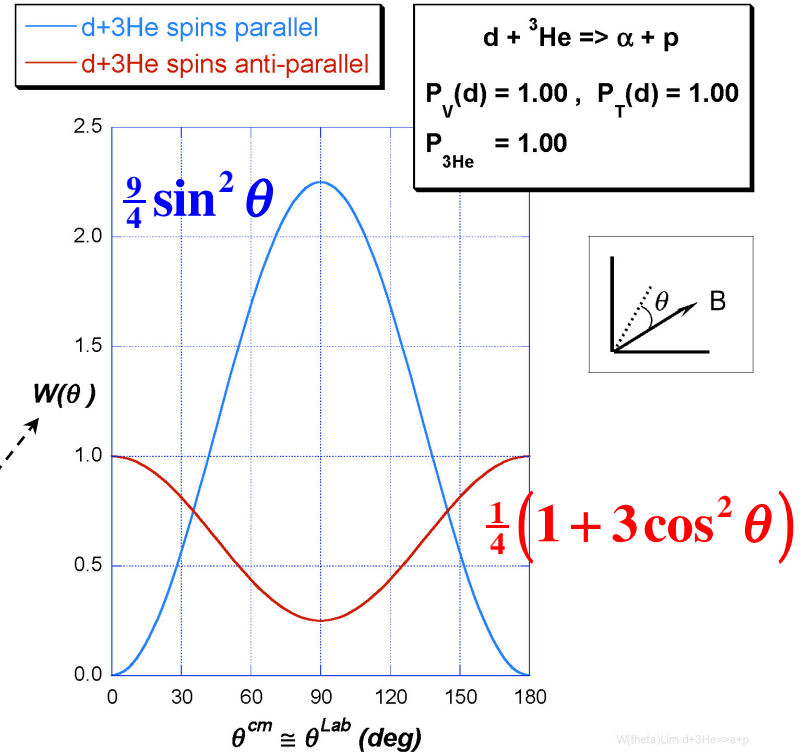
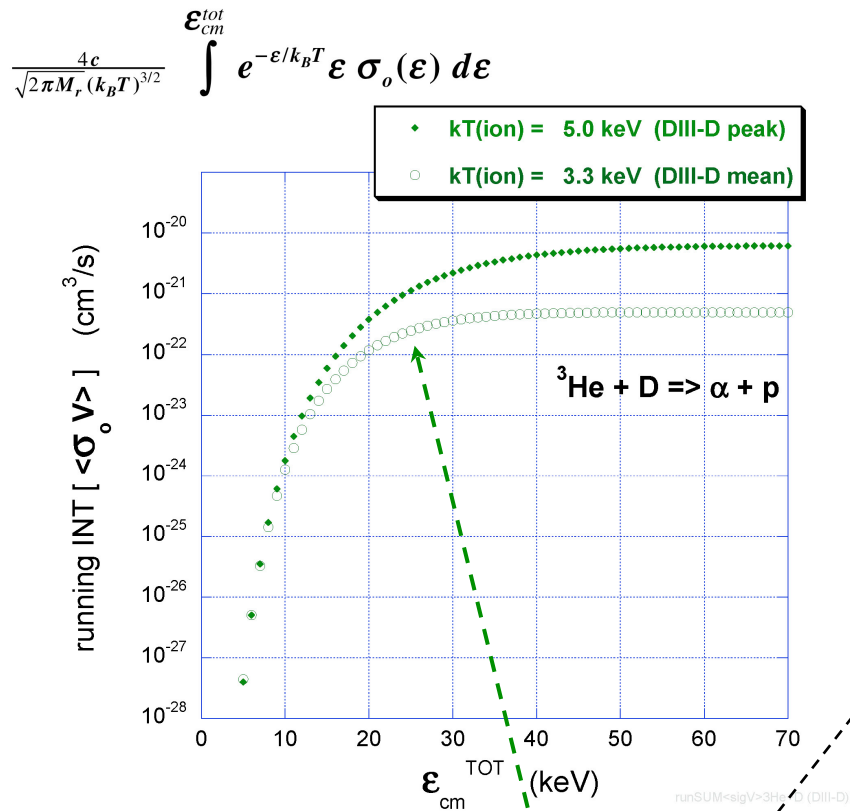
*entirely determined by energies*

< 120 keV (ITER)

< 40 keV (DIII-D)

runSUM<sigV>\_3He+D (kT)

# $d+{}^3\text{He} \rightarrow \alpha + p$ distributions wrt torus field



$$\langle d\sigma(\theta)V \rangle = \frac{1}{4\pi} \langle \sigma_o V \rangle \cdot W(\theta)$$

$$= \frac{1}{4\pi} \langle \sigma_o V \rangle \cdot \left\{ 1 - \frac{1}{2} P_d^V P_{3\text{He}} + \frac{1}{2} \left[ 3P_d^V P_{3\text{He}} \sin^2 \theta + \frac{1}{2} P_d^T (1 - 3 \cos^2 \theta) \right] \right\}$$

# expected $d+^3\text{He} \rightarrow \alpha+p$ distributions in a DIII-D test experiment

— d+3He spins parallel  
— d+3He spins anti-parallel

$d + ^3\text{He} \Rightarrow \alpha + p$   
 $P_V(d) = 0.40$ ,  $P_T(d) = 0.12$   
 $P_{^3\text{He}} = 0.70$

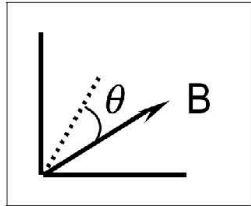
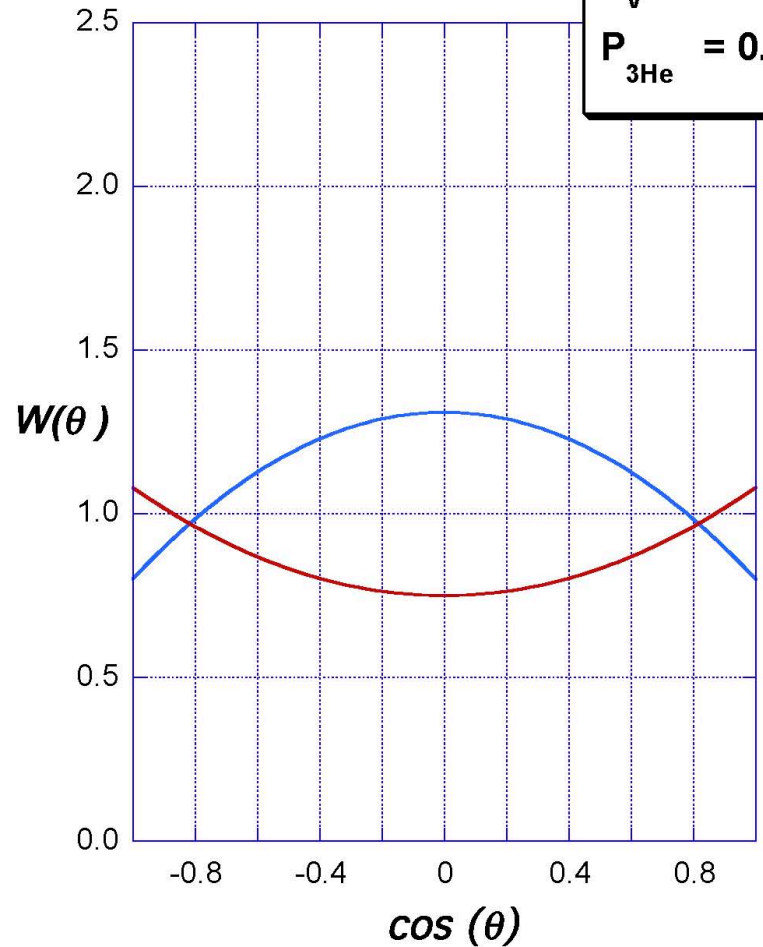
$$P_V(\vec{D}) = 0.40, P_T(\vec{D}) = 0.12$$

$$P(^3\vec{H}e) = 0.70$$

$$\langle \sigma^{par} V \rangle = \langle \sigma_o V \rangle \left\{ 1 + \frac{1}{2}(0.28) \right\}$$

$$\langle \sigma^{anti} V \rangle = \langle \sigma_o V \rangle \left\{ 1 - \frac{1}{2}(0.28) \right\}$$

$$\Rightarrow \frac{\langle \sigma^{par} V \rangle - \langle \sigma^{anti} V \rangle}{\langle \sigma_o V \rangle} = 0.28$$



W(cosAng)real d3He=>ap

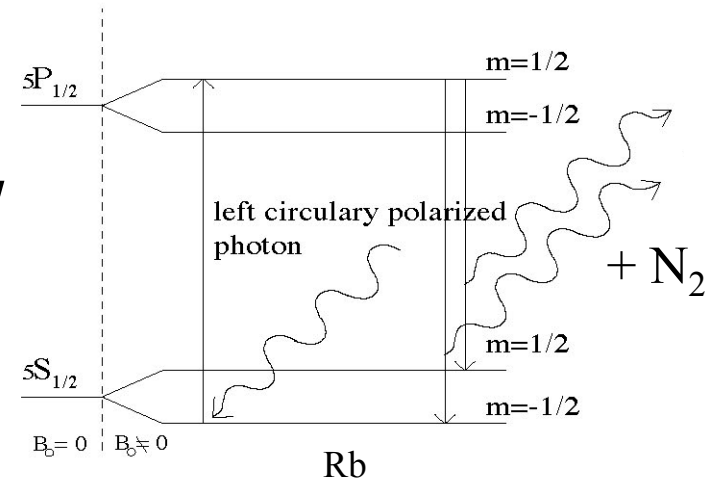
# Preparing $H\vec{D}$ fuel

⇒ talk by X. Wei  
Friday, Parallel-VII: S10

- **process:** HD, with impurities ( $10^{-4}$ ) of  $H_2$  and  $D_2$  condensed to solid
  - transferred to high field (15 tesla) and low temp (10-15 mK)
  - $H_2$  &  $D_2$  impurities metastable, polarize rapidly, transfer spin to HD
  - $H_2$  &  $D_2$  catalysts decay, leaving HD in frozen-spin state ( $T_1 \sim 10^8$  s)
- typical frozen-spin polarizations:  $P(\vec{H}) = 60\%$ ,  $P_V(\vec{D}) = 20\%$
- RF transitions are used to move spin from  $H$  to  $D \Rightarrow P_V(\vec{D}) = 40\%$
- large targets used in Nuclear Physics experiments at JLab and BNL
  - these polarizations are  $\sim \frac{2}{3}$  of theoretical maximum
  - there maybe some further gain in small ICF pellet sizes

## ${}^3\text{He}$ process - *spin exchange optical pumping*

- Rb vapor pumped with 795 nm laser -100W (in oven  $> 200$  C; with  $\sim 1\%$   $\text{N}_2$ ; uniform B)
- Rb transfers polarization to K by collisions
- K transfers spin to  ${}^3\text{He}$  by collisions

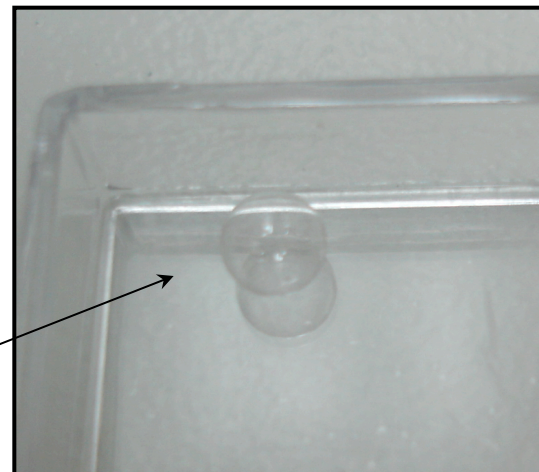


- typical polarizations in pumping chamber: 70% at 10 amagats ( $\sim 10$  atm) (some further gain may be possible with the right geometry cell)
- large volume targets used in Nuclear Physics exp at JLab, SLAC,...
- need for high-power laser  $\rightarrow$  must first polarize in glass cell,  
 $\rightarrow$  remove alkalis ( $\sim$  few ppm of  ${}^3\text{He}$ )  
 $\rightarrow$  then diffuse into ICF shell



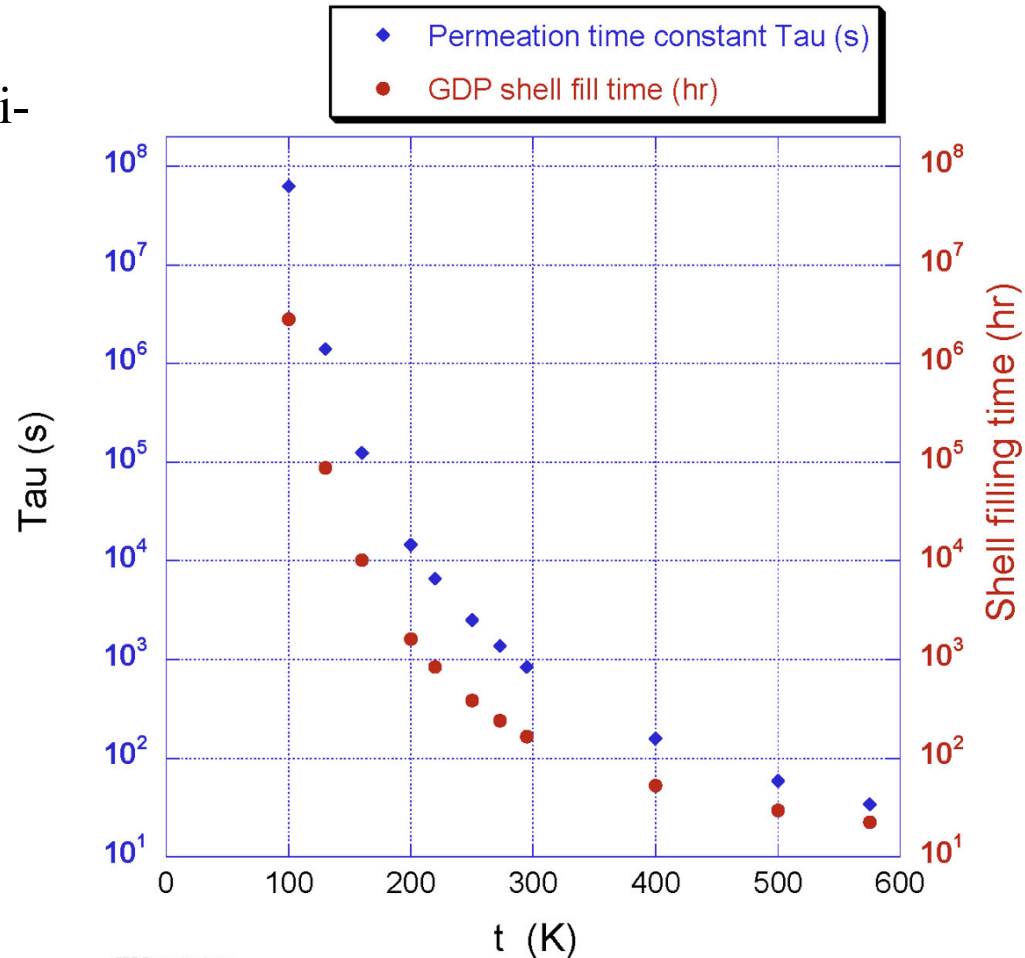
## *filling ICF shells*

- ICF shells:
  - fabricated by GA-ICF Pellet division
  - PAM (Poly-Alcohol repeating Monomer,  $C_9H_{10}$ ) mandrel is coated with a glow discharge polymer (GDP,  $CH_{1.3}$ ); heating to 600 K (326 C) dissociates PAM which diffuses out through GDP, leaving a shell behind.
- GDP shells filled in high-pressure chamber; pressure increased in steps to maintain small pressure differential across shell walls.
- Filling rate adjusted to remain below the shell buckling pressure:  
eg. typically  $\Delta P = 2/3 \times P_{buckle}$
- HD shells: 4 mm OD x 20  $\mu m$  wall
- $^3He$  shells: 4 mm (possibly 8 mm) OD x 20  $\mu m$  wall



## eg. HD Shell filling times

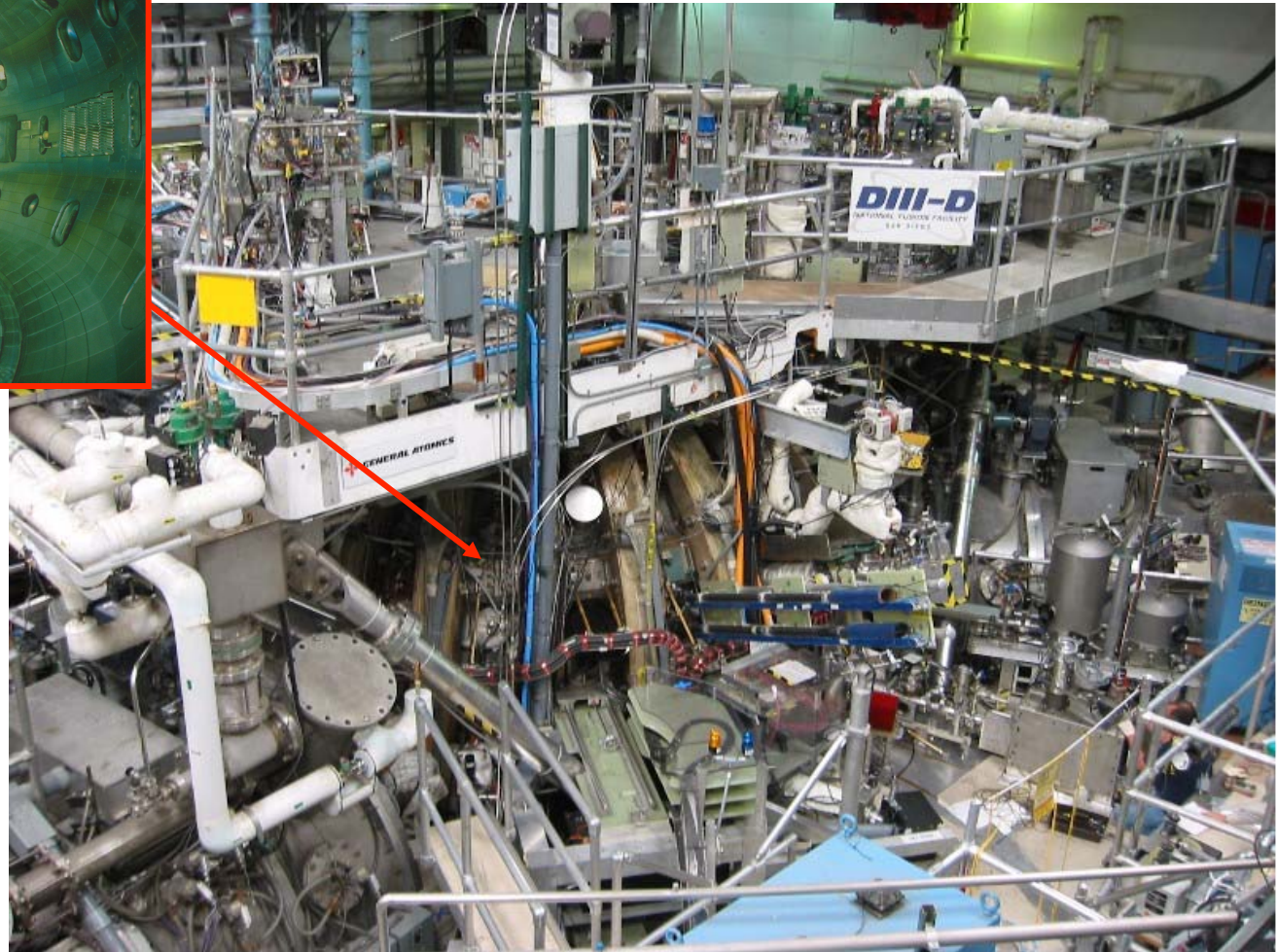
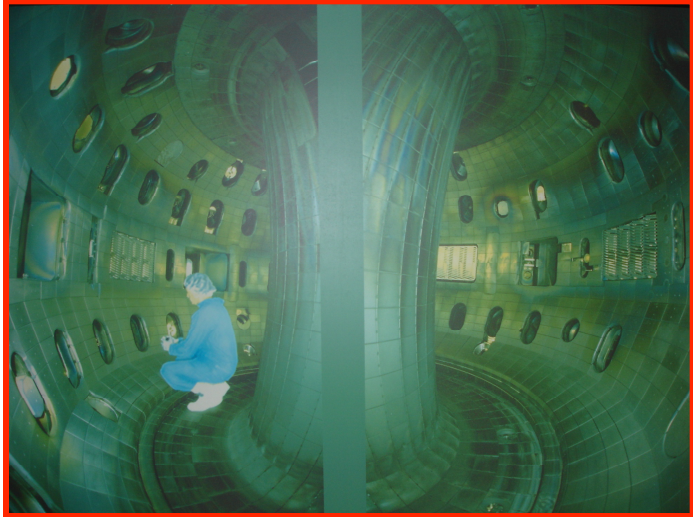
- eg. 4 mm OD x 20  $\mu\text{m}$  wall GDP shells require **23 hr** to fill to 0.3 millimoles with HD at 575 K
- pressure drops as chamber is cooled cryogenically:
  - at 100 K, pressure = 73 atm, permeation time  $\sim$  1 year
  - pellet is sealed !!!**
  - at 18 K, pressure =  $\frac{1}{4}$  atm, exterior gas pumped away & replaced with He for cooling



GDP Permeation times

# *DIII-D Tokamak at GA, San Diego*

2.1 tesla torus (normal-conducting coils)

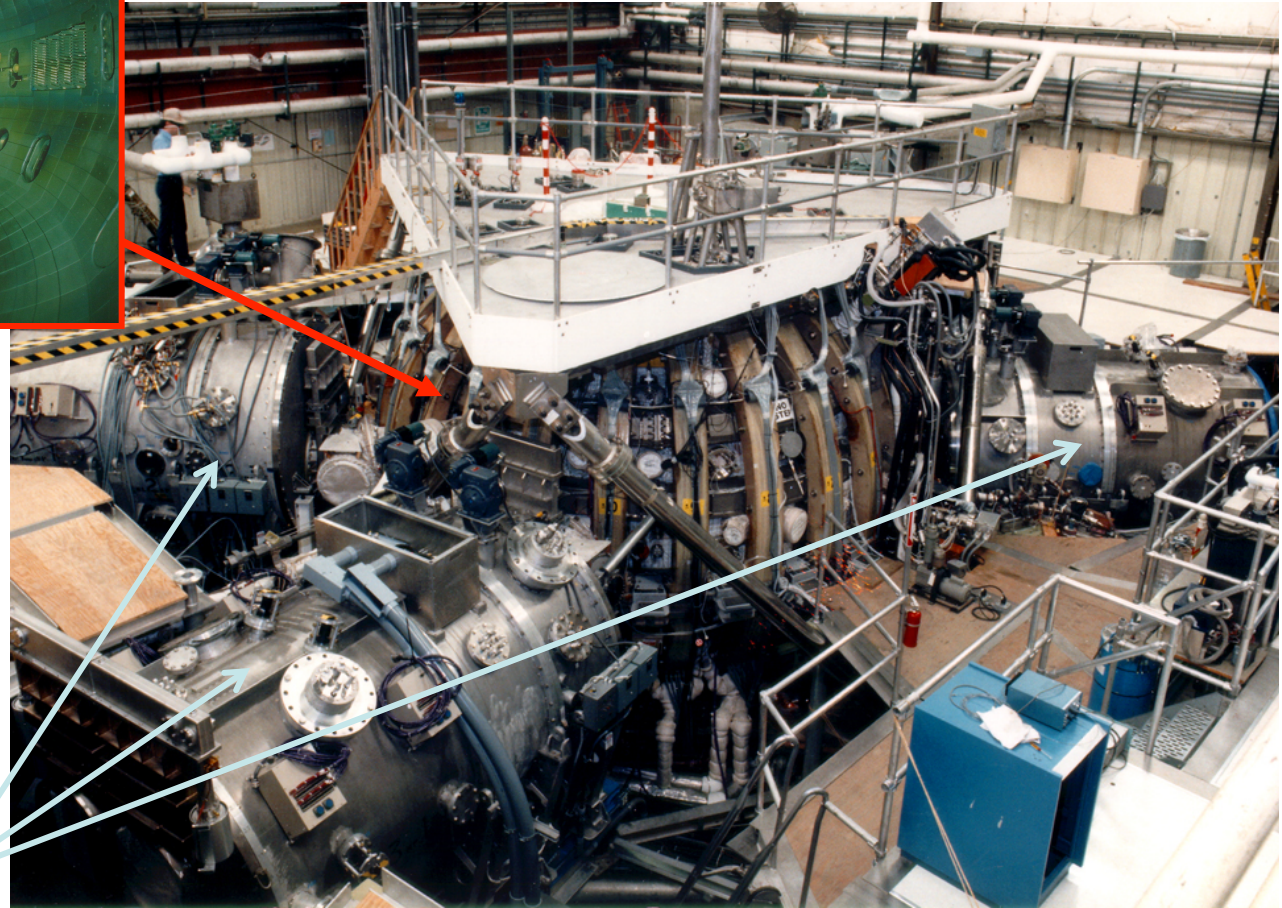
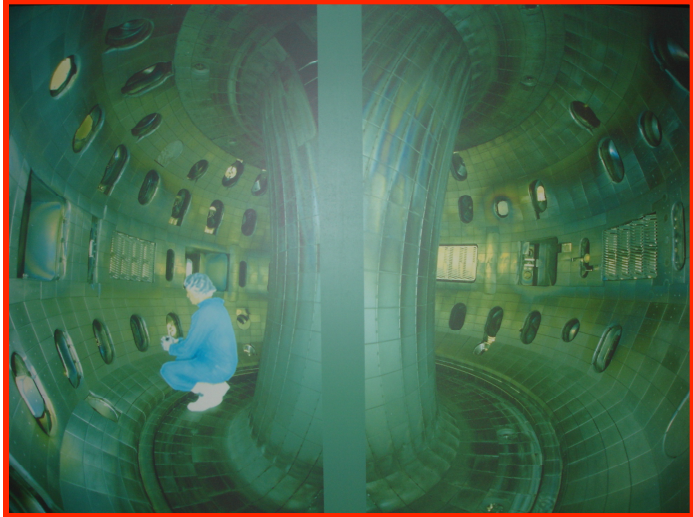


- 2.1 tesla max
  - B ramp up, 3 s
  - flat top ~ 10 s
  - ramp down, 7 s
- 15 min btw shots
- 80 keV neutral-beam Injectors for heating

# DIII-D Tokamak at GA, San Diego

2.1 tesla torus (normal-conducting coils)

archive photo – during installation, ca ~ 1985

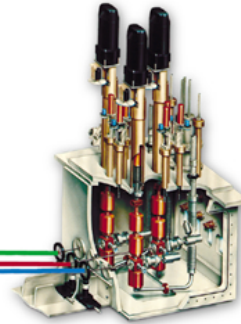
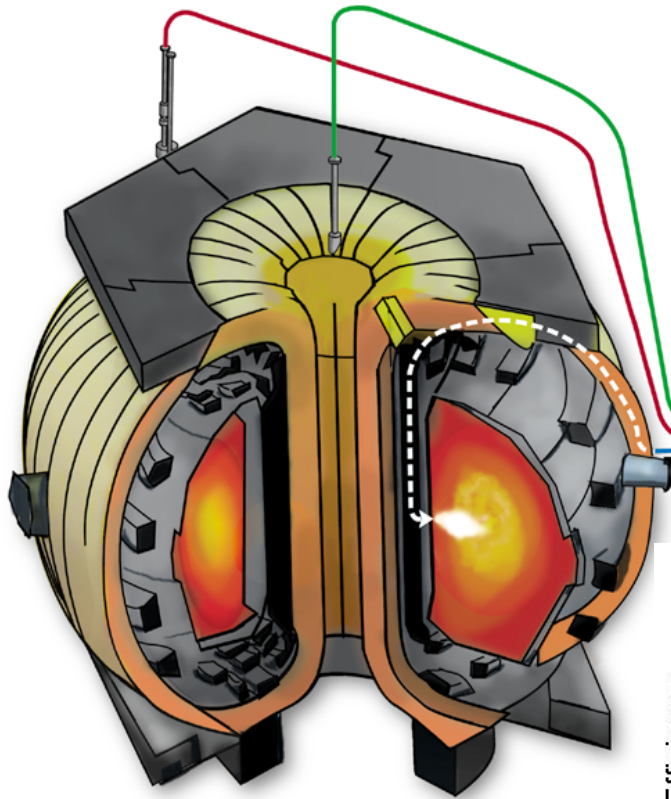


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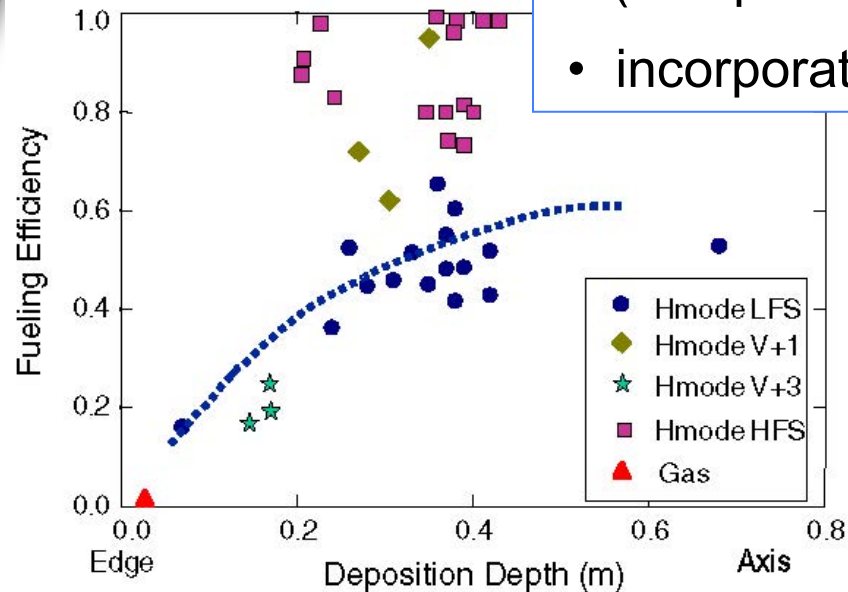
# pellet injection into DIII-D

- ORNL cryp-pellet injectors operating at 4 K  
- pellets propelled by H<sub>2</sub> gas ~1000 m/s



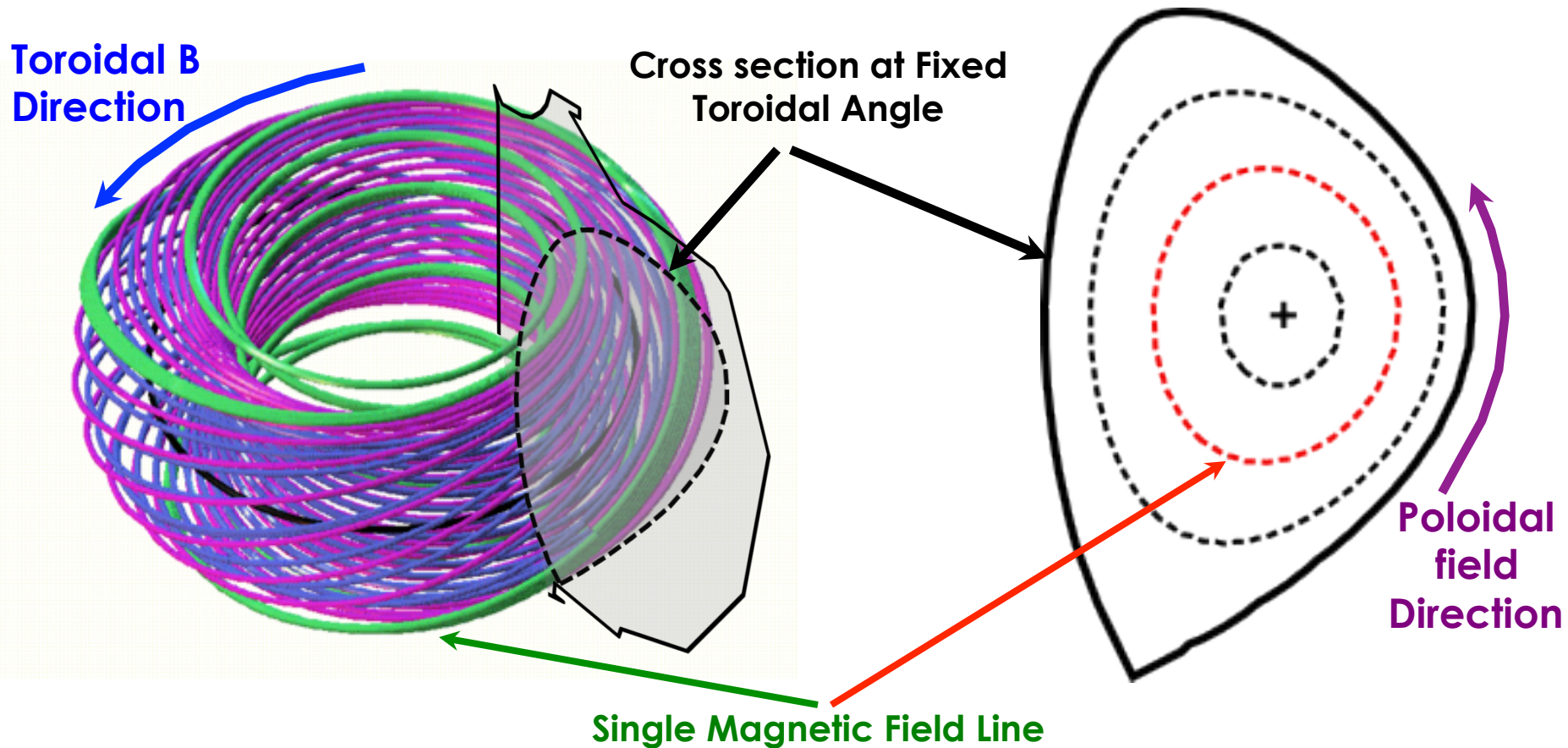
## Modifications:

- provide B field along full length of guide tubes
- adapt to HD cartridges (and possibly <sup>3</sup>He)
- incorporate SQUIDS



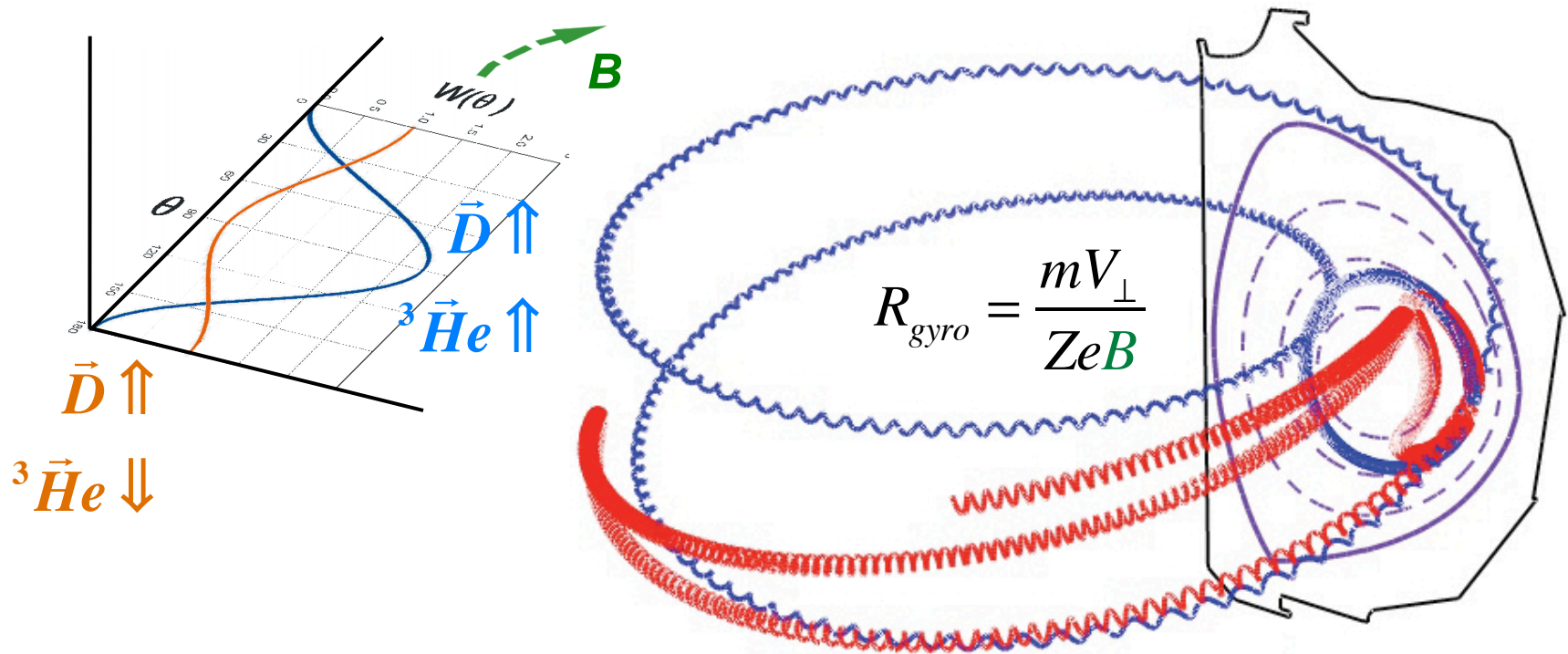
# Magnetic Geometry of a Tokamak – a Series of Nested Flux Surfaces

- confinement via gyrotron oscillation around closed magnetic field lines
- Axis-symmetry: equivalent cross-sections at different toroidal positions
- Flux surfaces represent the region of closed magnetic field lines



# Spin-Alignment and Orbit Losses

- parallel spins  $\rightarrow$  large  $V_{\perp}$   $\rightarrow$  large gyroradii  $\rightarrow$  hit the wall in a few orbits
- anti-parallel spins  $\rightarrow$  large  $V_{\parallel}$   $\rightarrow$  small gyroradii  $\rightarrow$  better confined

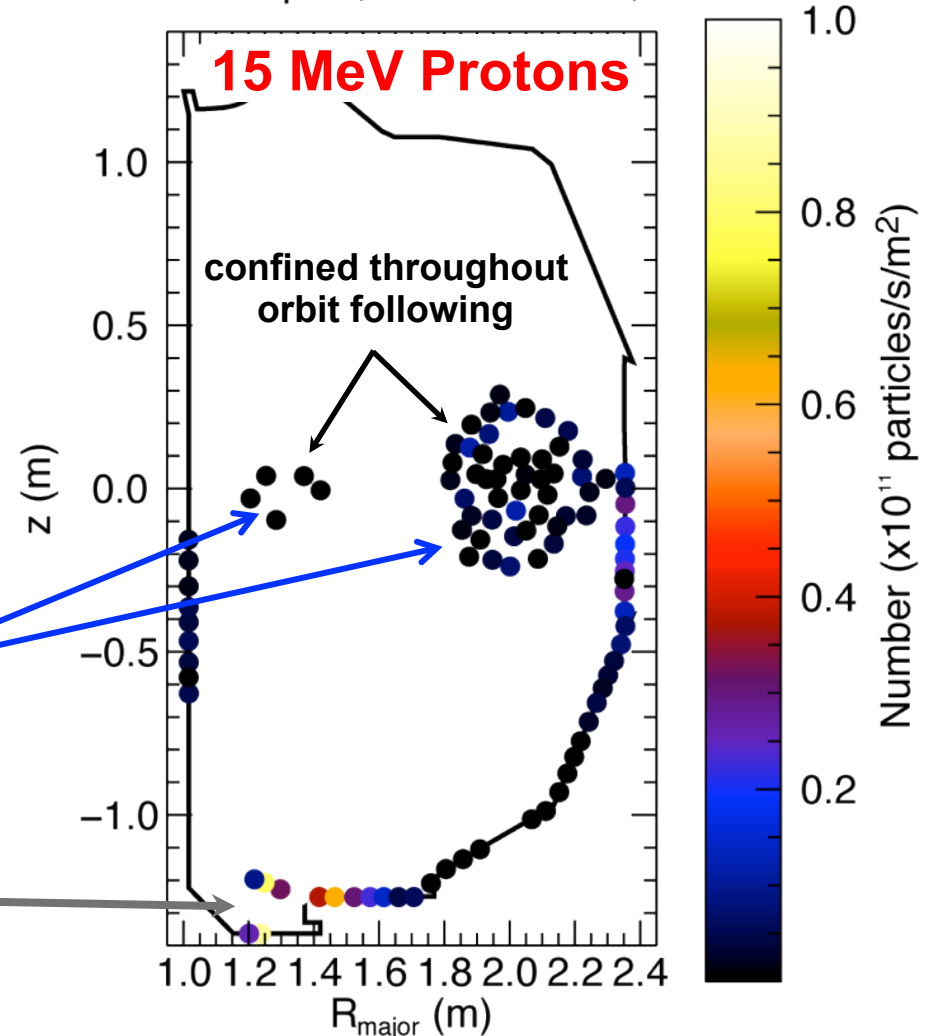


$\alpha$  and  $p$  loss-locations on Tokamak wall depend on initial polarizations

# tracking fusion products through DIII-D

- Orbit calculation follows a large (but limited) number of physical steps along the ion trajectory, until a wall strike is detected
- Most trajectories intersect the Tokamak wall within a few orbits
- For  $\alpha+p$  born parallel to the local field ( $\theta_{cm} \approx 0$ ), some are confined, depending on origin within the plasma
- Divertor region is location of largest loss population for parallel birth

Parallel Spins, Parallel Births, 099



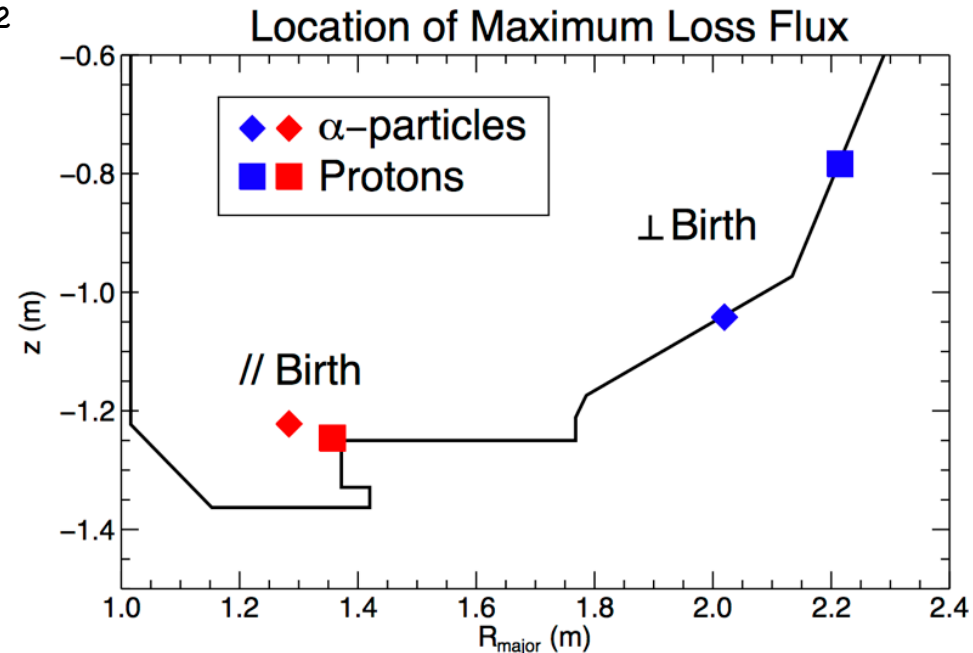


# Peak Loss Fluxes Occur at Different wall locations for $\parallel$ ( $\theta_{cm} \approx 0$ ) birth and $\perp$ ( $\theta_{cm} \approx 90$ ) birth fusion products

- Typical detector area  $\approx 4 \times 4 \text{ cm}^2$
- Expected fluxes from tracking:  
 $\sim 10^9$  particles reach detector

eg. flux (in units of 10 pA):

$\perp$ Birth	Anti-parallel Spin	Parallel Spin
$\alpha$ -particles	<b>1.6</b>	<b>2.6</b>
Protons	<b>1.6</b>	<b>2.5</b>

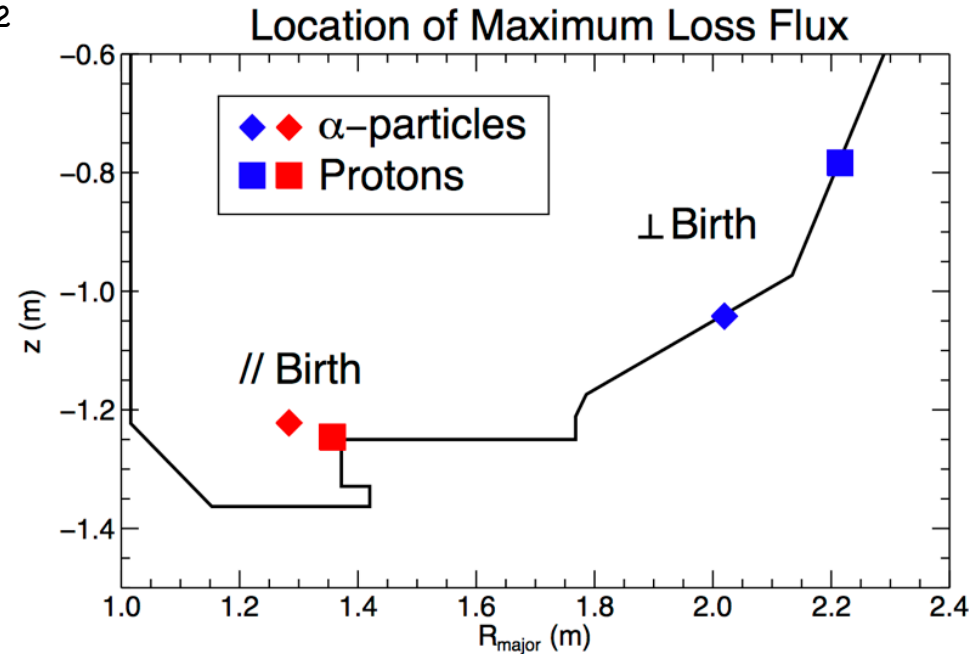


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 $\sim 10^9$  particles reach detector

eg. flux (in units of 10 pA):

$\perp$ Birth	Anti-parallel Spin	Parallel Spin
$\alpha$ -particles	1.6	2.6
Protons	1.6	2.5



- 28% difference  $(\sigma_{\uparrow\uparrow} - \sigma_{\uparrow\downarrow})/\sigma_0 \Rightarrow$  **44%** difference in proton signals, due to polarized angular distributions and how products propagate in  $B_{\text{Torus}}$
- tracking studies are underway to optimize signal and detector locations

## *secondary reactions*

- use H-plasma heated with H neutral beams

- simulations follow secondary reactions to estimate background yields:

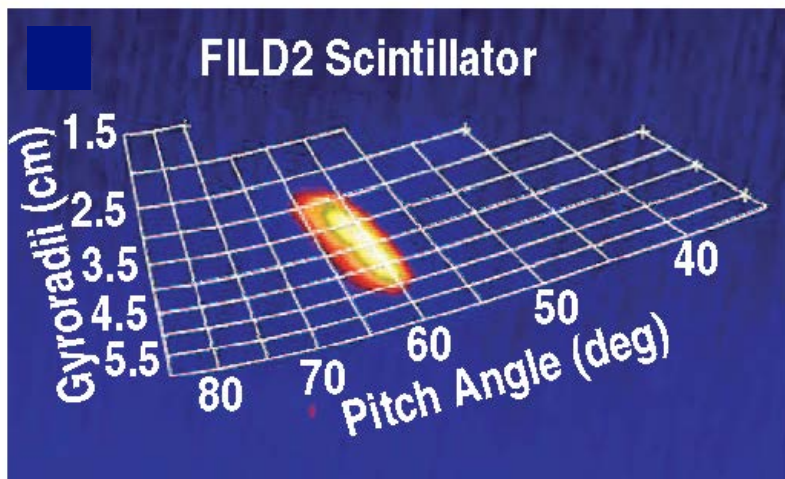


- 15 MeV protons from  ${}^3\text{He} + \text{D} \Rightarrow \alpha + \text{p}$  provide a unique signature that is easily separated

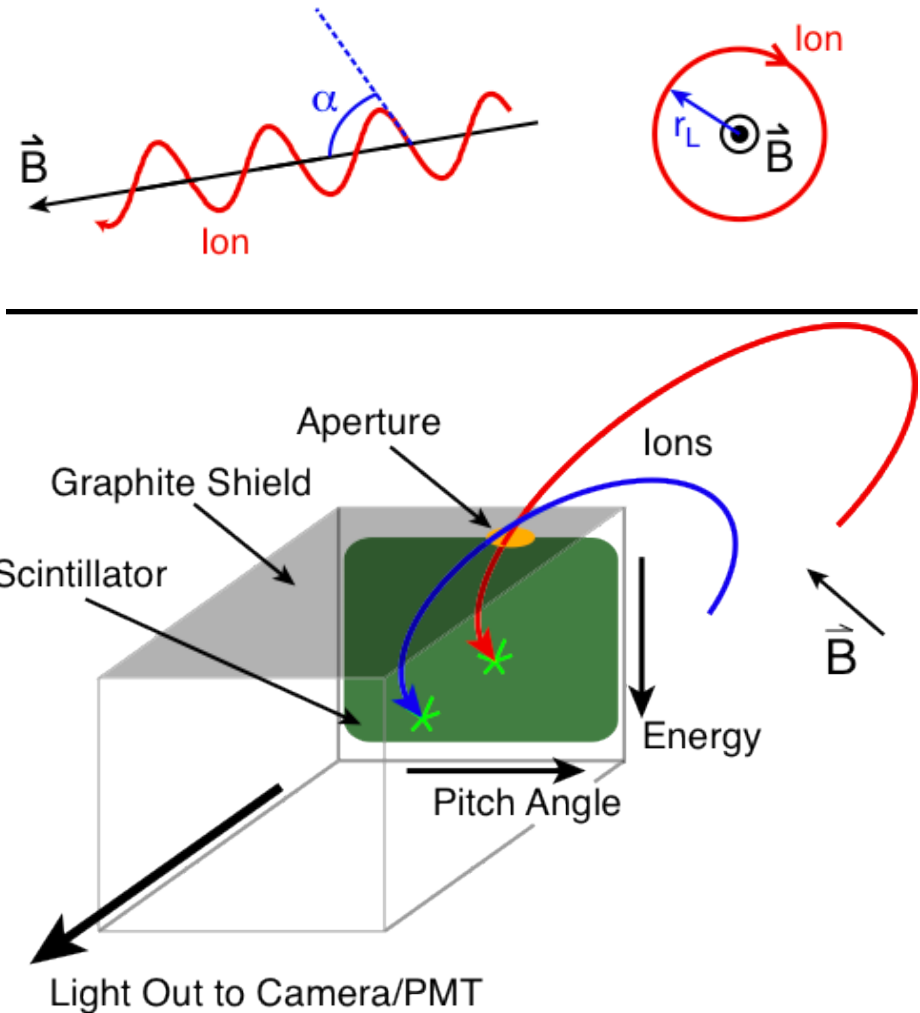
# Fast Ion Loss Detector (FILD) Measures the Energy and Pitch Angle of Energetic Ions that Reach its Position on the Outer Wall

light emission pattern :

- ↳ Gyro-radius  $\Leftrightarrow$  ion energy
- ↳ pitch (polar) angle



Chen et al, Rev Sci Inst 83 (2012) 10D707



## *Possible approximate Timeline*

### I. R&D (internally funded) to address open issues in preparation for a proposal to US DOE

- plasma requirements, injection, fusion product detection
  - **2013-2014 (2015) IR&D from General Atomics - funded ✓**
- HD: effects of ICF shell diffusion; max D polarization (largely riding on the tail of ongoing NP target development)
- $^3\text{He}$ : polarization after diffusion; max pressure
  - **some R&D required – tbd**



### II. Possible Funding request for a full experiment

- **eg. 2015-2016, proposal to US DOE (FES + NP)**
- **experiments at DIII-D ~ 2019**