

TRAINING COURSE ON NEUTRON DOSIMETRY, RADIOBIOLOGY AND INSTRUMENTATION:

Passive detectors: activation detectors and superheated emulsions

Stefano AGOSTEO, *POLIMI*

Wed. 24/6/2015, 17:00 – 18:00 pm



JABLOTRON

mi.am



POLITECNICO
DI MILANO

FAU



UNIVERSITY OF
HOUSTON

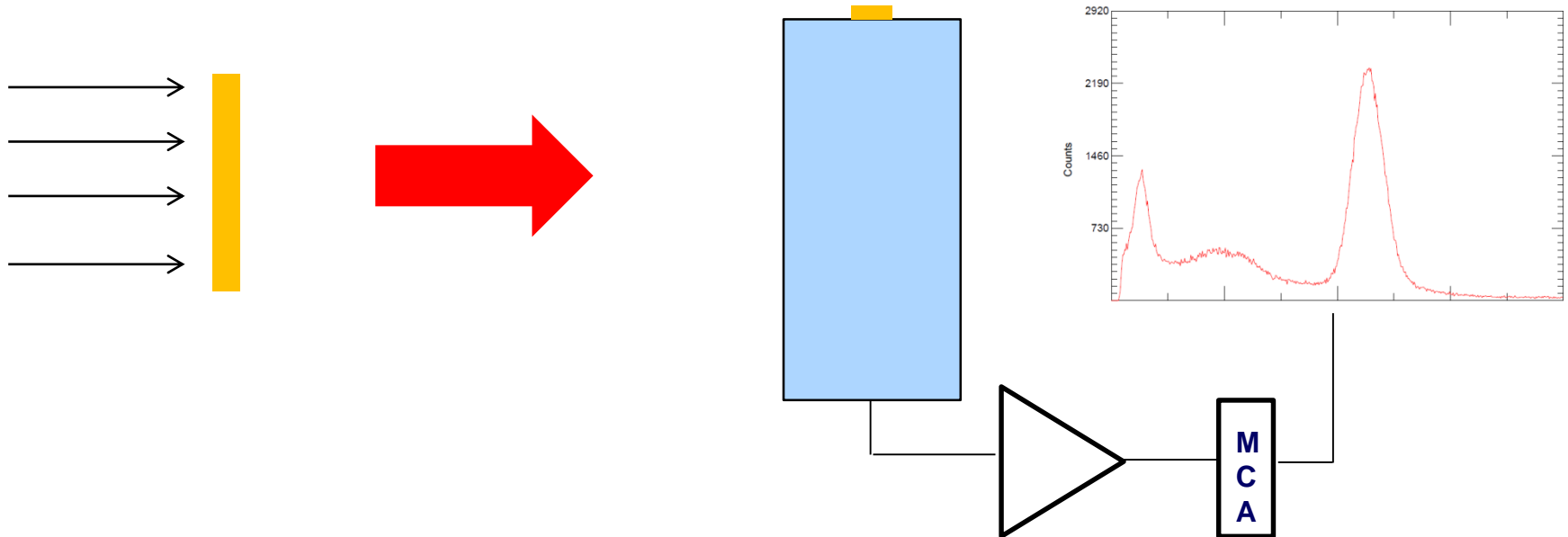
UOIT

UNIVERSITY OF
WOLLONGONG

20 Nov

ACTIVATION DETECTORS: introduction

- The method consists in measuring the induced activity of a target exposed to a neutron field and relating it to the neutron fluence rate.





ACTIVATION DETECTORS: basic principles

- The reaction rate (s^{-1}) is related to the neutron fluence rate by:

$$R = N_{TOT} \int_0^{\infty} \sigma_E \Phi_E dE$$

- where:

$$N_{TOT} = \frac{N_{AV}}{AW} W$$

- is the total number of nuclei inside the target, N_{AV} is the Avogadro number, AW the atomic weight and W the target weight.



ACTIVATION DETECTORS: thermal and epithermal neutrons

- The following simplified method assumes that,
 - for thermal neutrons

$$\sigma_E = \frac{\sigma_0 V_0}{v}$$

- where v is the neutron velocity, v_0 the neutron velocity @ 0.025 eV (2200 m s⁻¹), σ_0 is the neutron cross section @ 0.025 eV;

- and, for epithermal neutrons:

$$\Phi_E = \frac{\Phi_{epi}}{E} \quad (1/E \text{ slowing-down behaviour})$$

- where Φ_{epi} is the epithermal fluence rate per unit $\ln(E)$

ACTIVATION DETECTORS: thermal and epithermal neutrons

- the reaction rate R can be written as:

$$\begin{aligned}
 R &= N_{TOT} \int_0^{E_{max}} \sigma_E \Phi_E dE = N_{TOT} \left(\int_0^{0.5eV} \frac{\sigma_0 v_0}{v} n_{th,E} v dE + \int_{0.5eV}^{E_{max}} \sigma_E \frac{\Phi_{epi}}{E} dE \right) = \\
 &= N_{TOT} \left(\sigma_0 v_0 \int_0^{0.5eV} n_{th,E} dE + \Phi_{epi} \int_{0.5eV}^{E_{max}} \frac{\sigma_E}{E} dE \right) = \\
 &= N_{TOT} \sigma_0 n_{tot} v_0 + N_{TOT} RI \Phi_{epi} = N_{TOT} \sigma_0 \Phi_0 + N_{TOT} RI \Phi_{epi}
 \end{aligned}$$

- where:

$$RI = \int_{0.5eV}^{E_{max}} \frac{\sigma_E}{E} dE \quad \text{is the resonance integral (barn)}$$

- Φ_0 is the neutron fluence rate defined as the thermal neutron density times the 2200 m s⁻¹ neutron velocity.

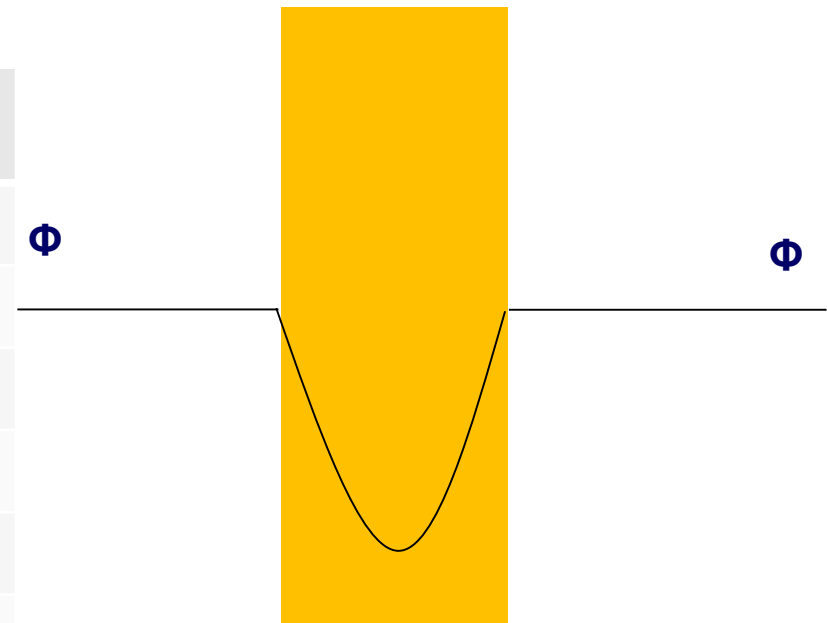
ACTIVATION DETECTORS: thermal and epithermal neutrons

- The expression in the previous slide is valid for an infinitely thin target:
- but an activation target shows a given thickness and the reaction rate expression must be corrected for the fluence rate depression factors, G_{th} and G_{epi}

$$R = G_{th} N_{TOT} \sigma_0 \Phi_0 + G_{epi} N_{TOT} R/\Phi_{epi}$$

- G_{th} and G_{epi} depend on the target material and thickness

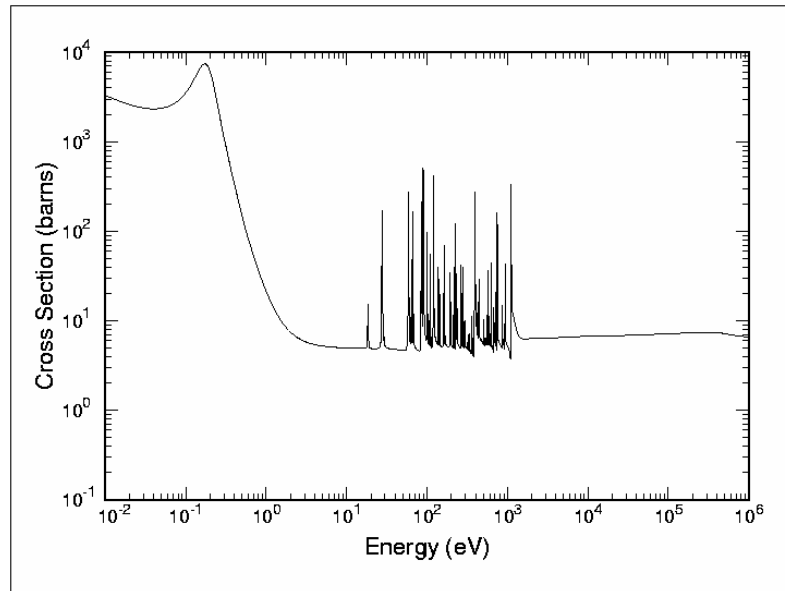
Target thickness (mg cm ⁻²)	Gold		Indium	
	G_{th}	G_{epi}	G_{th}	G_{epi}
5	0.995	0.763	0.987	0.649
7.5	0.994	0.698	0.981	0.573
10	0.992	0.645	0.976	0.519
20	0.985	0.521	0.956	0.400
40	0.969	0.410	0.924	0.294





ACTIVATION DETECTORS: thermal and epithermal neutrons

- The thermal neutron component can be discriminated by the epithermal one with a cadmium cover;
- cadmium cut-off @ 0.5 eV





ACTIVATION DETECTORS: target reaction rate and activity

- By neglecting:
 - neutron capture on already activated nuclei;
 - the target burn-up;
- The number of activated nuclei during irradiation is:

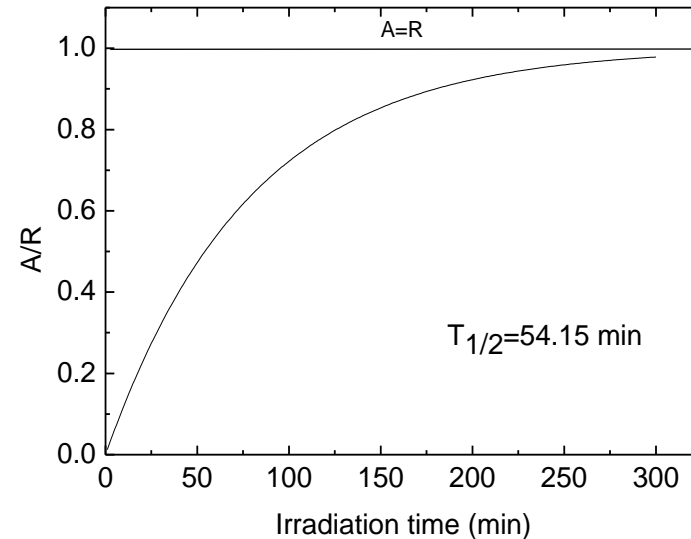
$$\frac{dN^*}{dt} = R - \lambda N^*$$

- At the end of irradiation (@ time t_{irr}):

$$N^*(t_{irr}) = \frac{R}{\lambda} (1 - e^{-\lambda t_{irr}})$$

- The induced activity at the end of irradiation is:

$$A(t_{irr}) = R(1 - e^{-\lambda t_{irr}})$$



ACTIVATION DETECTORS: target reaction rate and activity

- After a waiting time t_w (the time from the end of irradiation up to the beginning of counting):

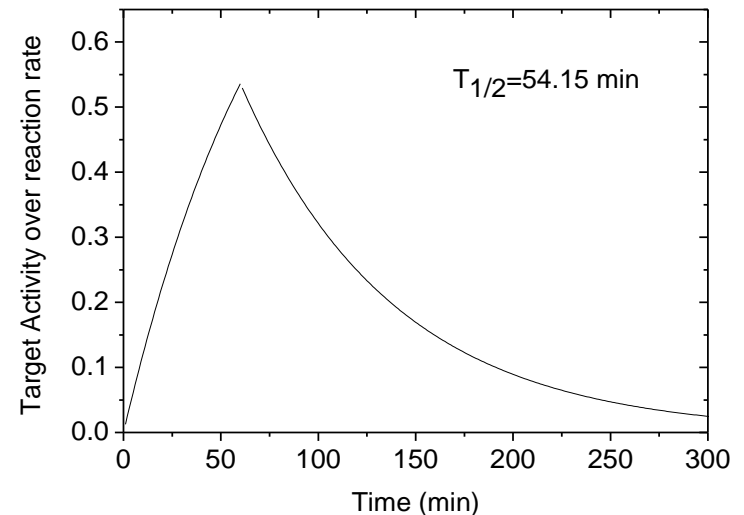
$$A(t_w) = A(t_{irr})e^{-\lambda t_w}$$

- The total counts acquired from t_w up to t_{meas} (i.e. counting time t_{meas}) are:

$$\begin{aligned}
 C_{tot}(t_{meas}) &= R\epsilon b(1 - e^{-\lambda t_{irr}})e^{-\lambda t_w} \int_{t_w}^{t_w+t_{meas}} e^{-\lambda t} dt = \\
 &= R\epsilon b(1 - e^{-\lambda t_{irr}})e^{-\lambda t_w} \frac{1 - e^{-\lambda t_{meas}}}{\lambda}
 \end{aligned}$$

- If $\lambda t_{meas} \ll 1$:

$$\frac{1 - e^{-\lambda t_{meas}}}{\lambda} = \frac{1}{\lambda} (1 - 1 + \lambda t_{meas}) = t_{meas}$$



ACTIVATION DETECTORS: target reaction rate and activity

- The reaction rate is assessed by measuring the saturation activity of the activated material (gamma rays with a NaI(Tl) or a Ge detector, β^- particles with a GM detector):

$$R = \frac{C_{tot}}{\varepsilon b (1 - e^{-\lambda t_{ir}}) e^{-\lambda t_w} \frac{1 - e^{-\lambda t_{meas}}}{\lambda}}$$

- Where b is the branching ratio and ε is the detector (peak) efficiency.
- If a bare and a cadmium covered target are used to separate the thermal and the epithermal components, the cadmium correction factor F_{Cd} should be used;
 - since cadmium is not completely transparent to epithermal neutrons.

$$C_{epi} = F_{Cd} C_{Cd}$$

- where C_{epi} are the counts due to epithermal neutrons to be subtracted from the counts from the bare target and C_{Cd} are the counts from the Cd-covered target.
- F_{Cd} depends on the thickness of the target material and of the Cd cover.

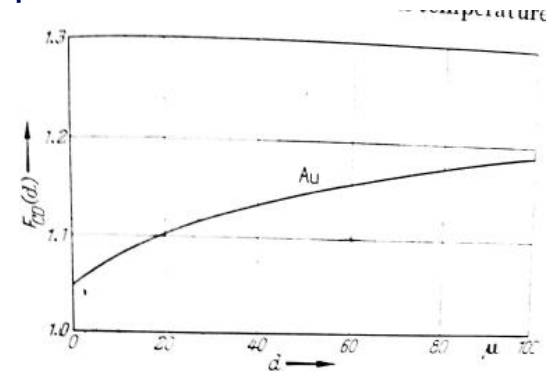


Fig. 12.2.2. The Cadmium correction factor for Gold foils in an isotropic neutron field as a function of Gold foil thickness. Cd filter thickness 1 mm

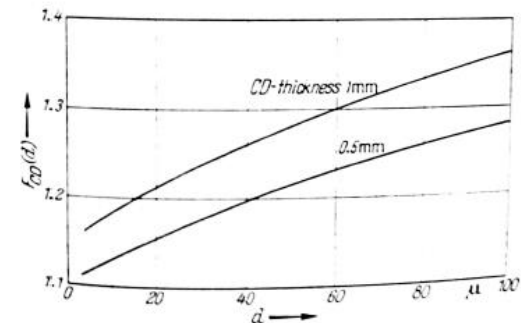


Fig. 12.2.3. The Cadmium correction factor for Indium foils in an isotropic neutron field as a function of Indium foil thickness. Cd filter thickness 0.5 and 1 mm

ACTIVATION DETECTORS: target reaction rate and activity

- The specific saturation activities should be subtracted for obtaining that due to thermal neutrons only:

$$R_{th} = \left(\frac{R_{bare}}{W_{bare}} - \frac{R_{epi}}{W_{Cd}} \right) W_{bare}$$

- where W_{bare} and W_{Cd} are the weights of the bare and Cd-covered target, respectively.
- Finally, for estimating Φ_0 and Φ_{epi} :

$$\Phi_0 = \frac{R_{th}}{G_{th} N_{tot} \sigma_0}$$

$$\Phi_{epi} = \frac{R_{epi}}{G_{epi} N_{tot} RI}$$

- It should be remembered that in the epithermal region:

$$\Phi = \int_{0.5eV}^{E_{max}} \frac{\Phi_{epi}}{E} dE = \Phi_{epi} \ln \left(\frac{E_{max}}{0.5eV} \right)$$



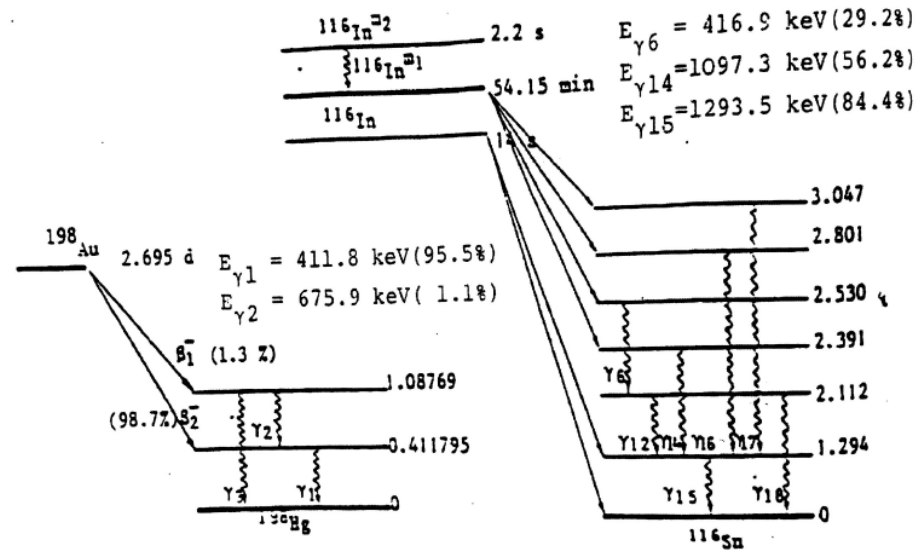
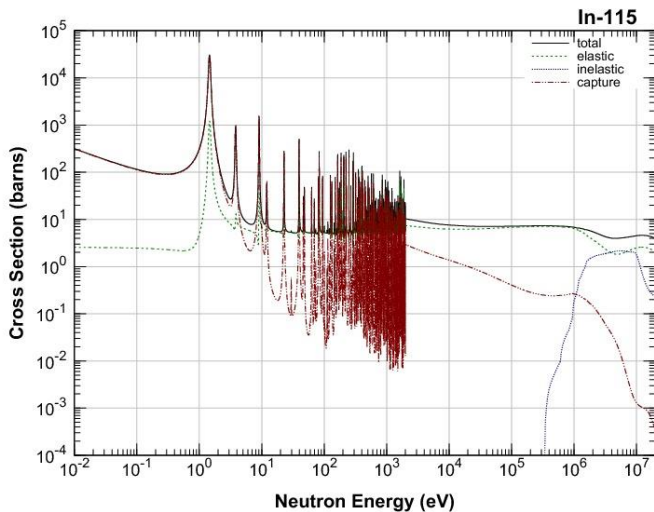
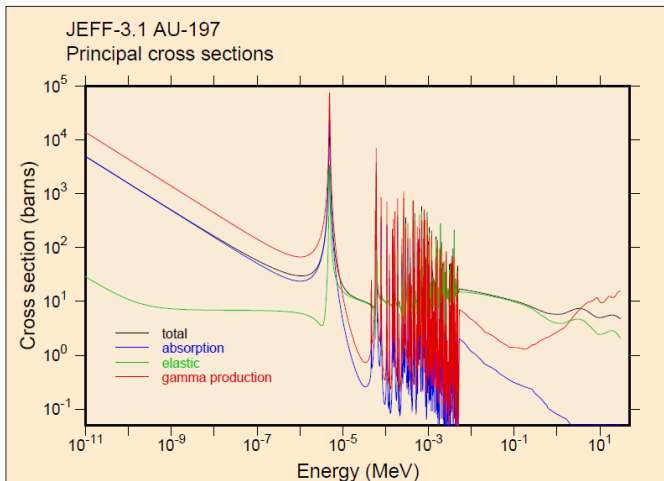
ACTIVATION DETECTORS: target materials for thermal and epithermal neutron detection

- Main activation reactions for thermal neutron detection:
 - ✓ $^{197}\text{Au}(n,\gamma)^{198}\text{Au}$: $T_{1/2}=2.69$ d, $\sigma(0.025$ eV)=98.5 b;
 - ✓ $^{115}\text{In}(n,\gamma)^{116\text{m}}\text{In}$: $T_{1/2}=54.15$ min, $\sigma(0.025$ eV)=157 b;
 - ✓ Other materials: Dy, Co, Cu, Ag.

	Gold	Indium
Half-life	2.695 d (^{198}Au)	54.15 min ($^{116\text{m}}\text{In}$)
σ_0 (0.025 eV)	98.8 b	157 b
RI	1560 b	2600 b



ACTIVATION DETECTORS: gold and indium foils



Au-198 and In-116m decay schemes with branching ratios (in brackets)



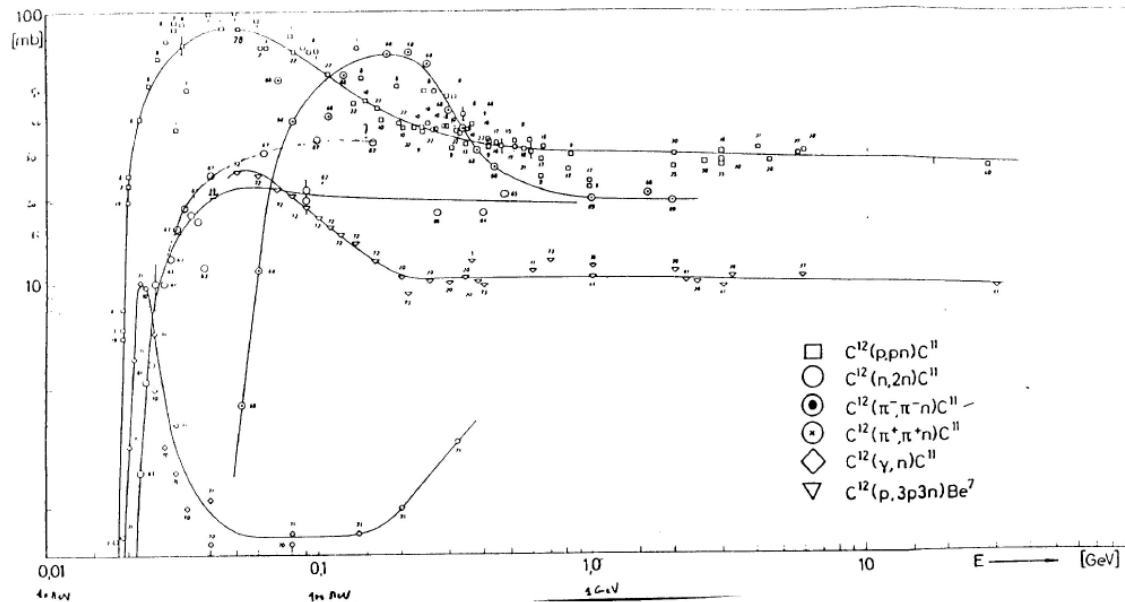
ACTIVATION DETECTORS: fast neutrons

- Several threshold reactions can be exploited, e.g.:
 - ✓ $^{58}\text{Ni}(n,p)^{58}\text{Co}$ $E_{\text{th}} = 1.9 \text{ MeV}$
 - ✓ $^{59}\text{Co}(n,\alpha)^{56}\text{Mn}$ $E_{\text{th}} = 5.2 \text{ MeV}$
 - ✓ $^{54}\text{Fe}(n,p)^{54}\text{Mn}$ $E_{\text{th}} = 2.2 \text{ MeV}$
 - ✓ $^{58}\text{Ni}(n,2n)^{57}\text{Ni}$ $E_{\text{th}} = 13.0 \text{ MeV}$
 - ✓ $^{115}\text{In}(n,n')^{115\text{m}}\text{In}$ $E_{\text{th}} = 0.339 \text{ MeV}$
 - ✓ $^{32}\text{S}(n,p)^{32}\text{P}$ $E_{\text{th}} = 2.0 \text{ MeV}$
 - ✓ $^{12}\text{C}(n,2n)^{11}\text{C}$ $E_{\text{th}} = 20 \text{ MeV}$
 - ✓ $^{27}\text{Al}(n, \alpha)^{24}\text{Na}$ $E_{\text{th}} = 4.9 \text{ MeV}$
 - ✓ $^{27}\text{Al}(n, p)^{27}\text{Mg}$ $E_{\text{th}} = 3.8 \text{ MeV}$
- The neutron spectrum can be reconstructed from the saturation activities assessed with a set of activation foils;
 - The reaction cross section against energy (the “detector response”) must be known for this purpose.



ACTIVATION DETECTORS: high-energy hadrons

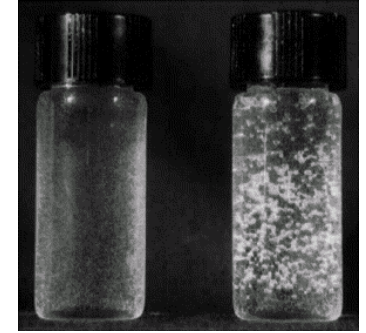
- The hadron fluence above about 20 MeV can be assessed through the activation of ^{11}C ($\sigma\text{-sec} \approx 20$ mb, slightly dependent on hadron energy), i.e. for neutrons through the reaction:
 - ✓ $^{12}\text{C}(n,2n)^{11}\text{C}$ $E_{\text{th}} = 20$ MeV $T_{1/2} = 20.5$ min
 - A plastic scintillator is exposed to the hadron field and
 - the ^{11}C activity is measured by coupling the scintillator to a PM and by counting the positrons emitted by ^{11}C decay.





SUPERHEATED EMULSIONS

- “**Superheated emulsion**” is the name adopted by ISO and ICRU for detectors based on a **superheated liquid suspended in a gel**, also known as bubble detectors or superheated drop detectors.
 - ✓ The suspended droplets consist of an over-expanded halocarbon and/or hydrocarbon which **vaporizes** upon exposure to **the high-LET recoils** from neutron interactions.
 - ✓ The superheated emulsion is contained in a vial and acts as a continuously sensitive, miniature bubble chamber.
 - ✓ The **total number of bubbles** evolved from the radiation-induced nucleation of drops gives an **integrated measure** of the total neutron exposure.

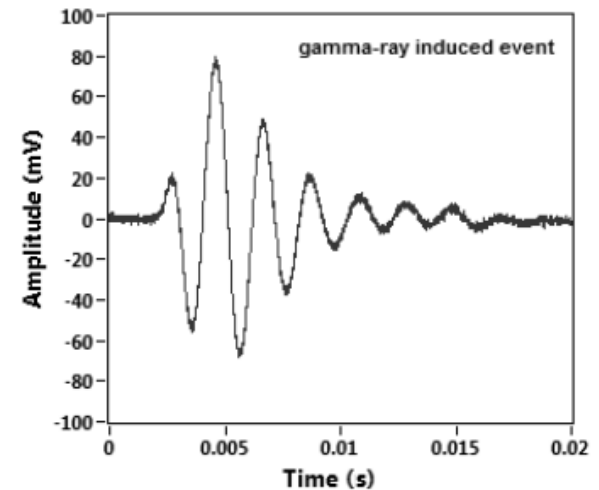
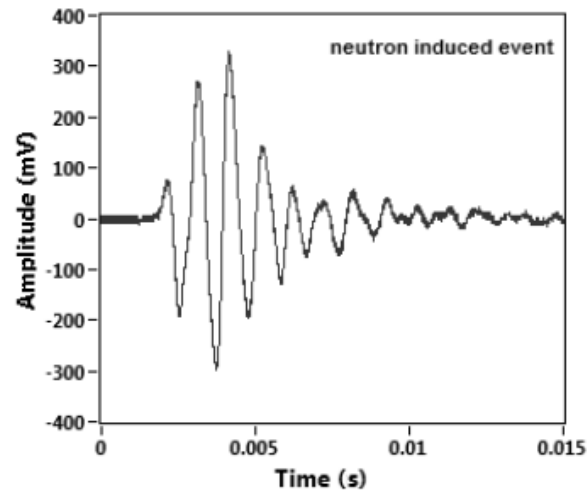


Courtesy of F. d'Errico, Yale Univ. and DMNP Pisa Univ.



SUPERHEATED EMULSIONS

- ✓ Bubbles can be counted either optically (by eye) or through an acoustic transducer transforming the micro-explosion following bubble formation into an electronic signal.

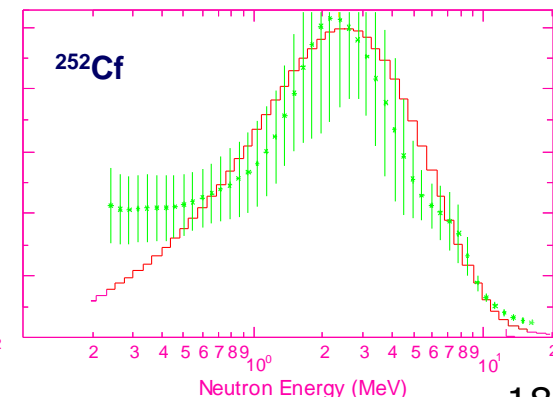
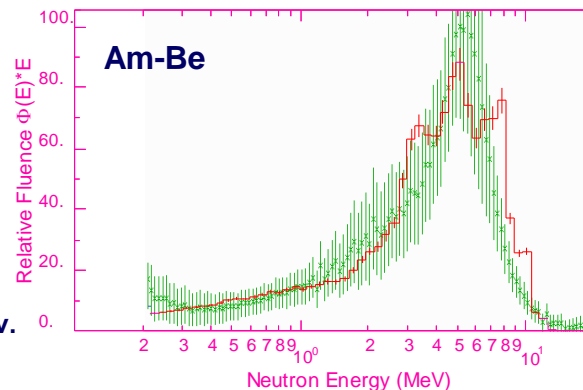
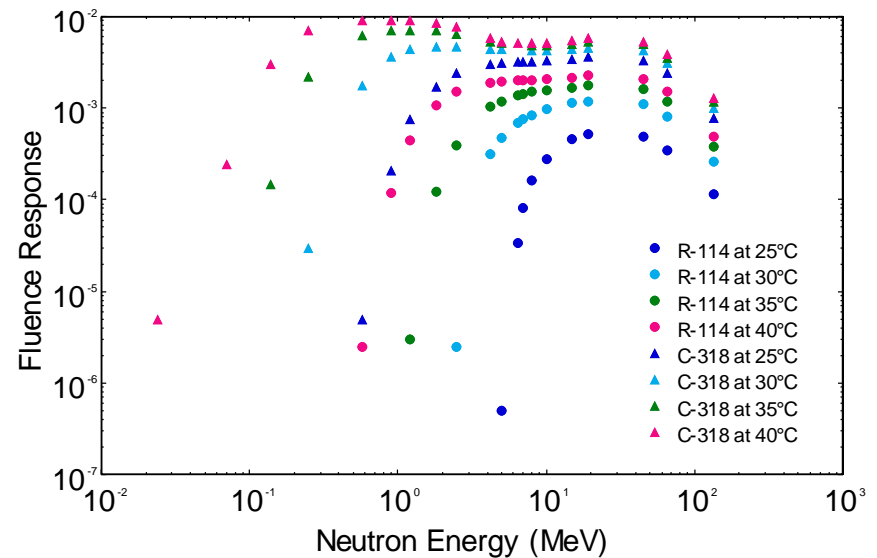


P.K. Mondal et al. Nucl.Instrum.Meth. A729 (2013) 182-187



SUPERHEATED EMULSIONS

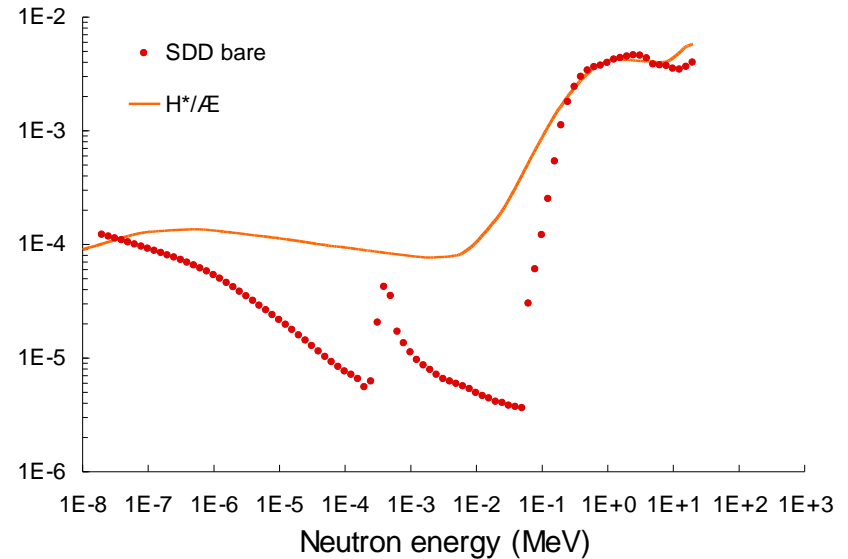
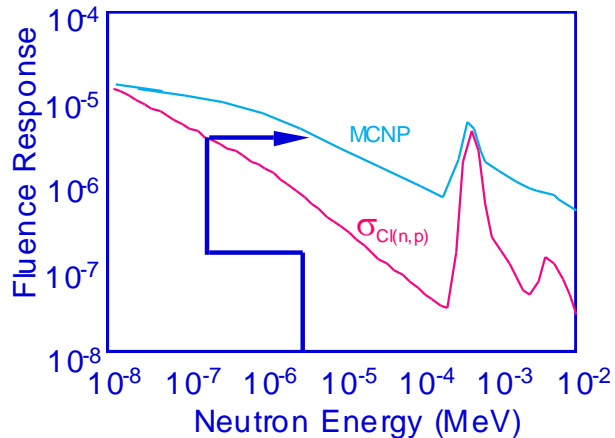
- Superheated emulsions are currently used either as **personal and environmental dosimeters** or as **neutron spectrometers**.
 - ✓ **Neutron spectrometry** is performed by exploiting the different response to neutron energy against temperature or pressure of the superheated liquid.
 - ✓ **Dosimeters**: one of their advantages is the possibility of determining an average ambient dose equivalent rate in a pulsed neutron field.
 - ✓ They are **completely insensitive to low-LET radiation**, X and γ rays as well as muons, which is a clear advantage when measuring the neutron component in mixed fields.



All figures in this slide:
Courtesy of F. d'Errico, Yale Univ. and DMNP Pisa Univ.

SUPERHEATED EMULSIONS

- The $H^*(10)$ response is underestimated for epithermal neutrons (up to about 100 keV) and is fairly accurate in the neutron energy interval from 100 keV up to about 10 MeV.



- Low energy response: MCNP calculations
- Fast neutron response: PTB calibrations

All figures in this slide:
Courtesy of F. d'Errico, Yale Univ. and DMNP Pisa Univ.

SUPERHEATED EMULSIONS

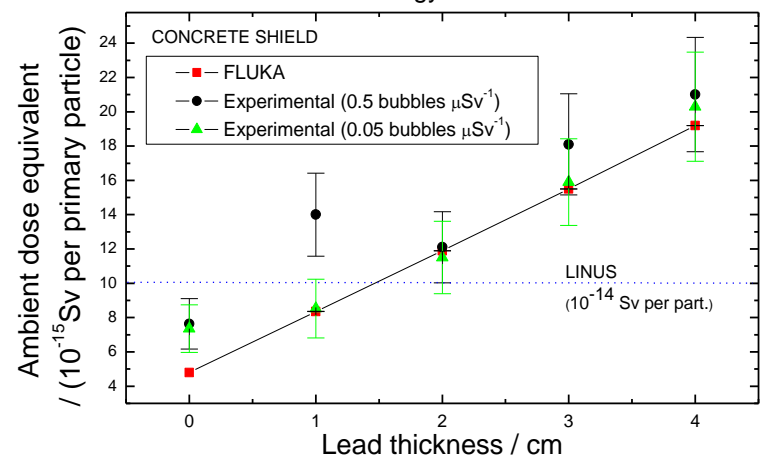
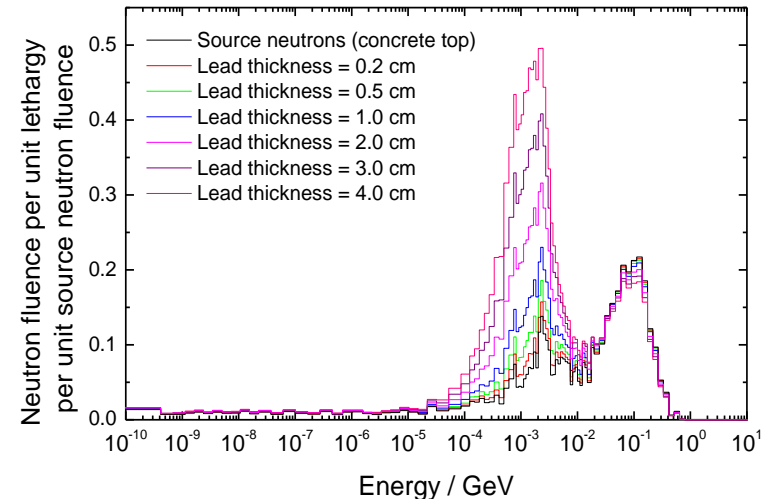
- The response to higher energies was measured by irradiating bubble detectors with quasi-monoenergetic neutrons in the energy interval 46-133 MeV. The results showed a significant underestimate of the $H^*(10)$ (d'Errico et al. RPD 100 (2002) 529-532).
- Measurements were also performed in the mixed field of high-energy radiation available at CERF. An underestimation of about 40% with respect to the reference ambient dose equivalent was observed in that experiment (Mitaroff et al., RPD 102 (2002) 7-22).
- Measurements in high-energy neutron fields generated by various types of hadron beams performed at CERN showed that bubble detectors underestimate the $H^*(10)$ by a factor 0.4-0.7 depending on the neutron spectrum (Agosteo et al. Health Phys. 75 (1998) 619-629).

Detector	Ambient dose equivalent rate ($\mu\text{Sv h}^{-1}$)			
	front NA44	side NA44	dump NA45	NA45
LINUS _{sph,UMi}	21.2±0.2	22±1	227±22	108±10
bubble detectors	19±4	13±1.5	210±44	78±6



SUPERHEATED EMULSIONS

- The possibility of extending the response of bubble detectors to HE neutrons was investigated by exposing the dosimeters inside lead converters of varying thickness at the CERF facility.
 - MC simulations showed that, as the thickness of the lead converter increases, a growing number of evaporation neutrons are generated by the high-energy component of the neutron field, thus enhancing the detector sensitivity.
 - This behaviour was confirmed experimentally. The comparison with the reference $H^*(10)$ indicates that the required thickness of the lead converter is in the interval 1-1.5 cm.





SUPERHEATED EMULSIONS: OPTICAL BUBBLE COUNTING

- The application of large volume detector chambers for the three-dimensional dosimetry of brachytherapy implants lead to study novel position-sensitive systems for assessing the bubble spatial distribution.
 - ✓ **Optical tomography** was proposed by d'Errico et al, 2008 for this purpose.
- The satisfactory results obtained with this technique lead to apply **scattered light for bubble counting of superheated emulsions for individual dosimetry** (d'Errico et al, 2008).
- The dosimeter is placed in a light-shielded enclosure and **illuminated from the bottom by LEDs** (light-emitting diodes). The light scattered by the bubbles is detected by photodiodes positioned along the detector wall.
- A **very good linearity** of the response (photodiode voltage against number of bubbles) of this system was observed.
- The **uniformity in size of the drops** suspended in the gel was found to be of primary importance for a smooth behaviour of the system. This feature is guaranteed by the manufacturing technique for the superheated emulsions which is capable of providing drops with size **in the range 50-150 μm with a dispersion lower than 10%**.

