

FROM RESEARCH TO INDUSTRY

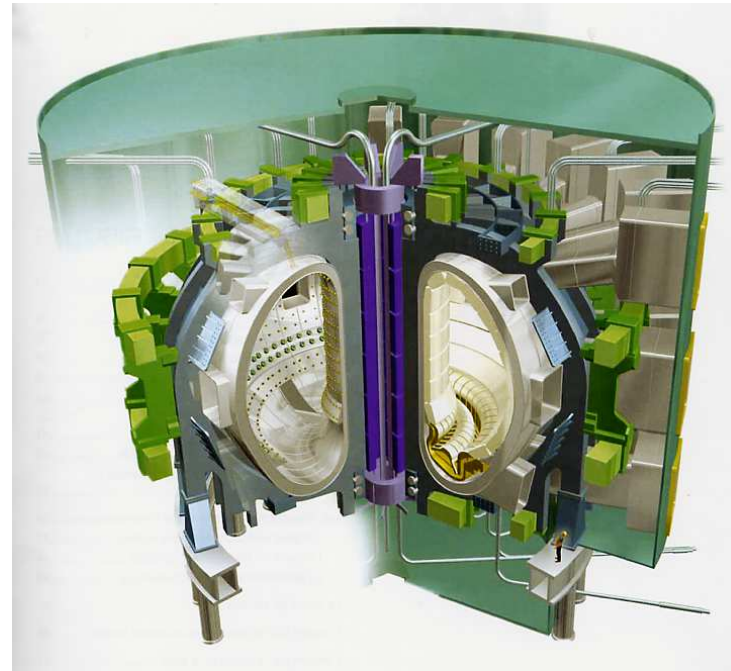
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THERMO-HYDRAULIC ANALYSES ASSOCIATED TO CEA DESIGN PROPOSAL FOR DEMO TF CONDUCTOR

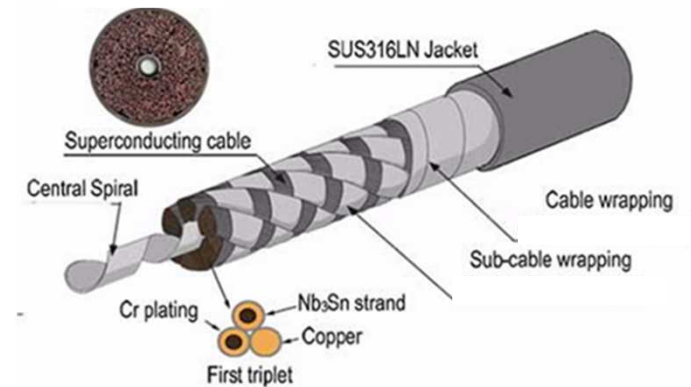


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Chats on Applied Superconductivity

Bologna, 14-16 September 2015

- Context
- CEA design proposal
 - ❖ Conductor features
 - ❖ Magnetic field
 - ❖ Nuclear heating
- Thermohydraulic tools
 - ❖ Cryosoft THEA code
 - ❖ Simplified thermohydraulic code
- Thermohydraulic analysis
 - ❖ Main hydraulic parameters
 - ❖ Burn scenario
 - ✓ Influence of cooling channel in casing
 - ✓ ΔT margin
 - ❖ Quench scenario
 - ✓ Quench initiation, detection and protection
 - ✓ Hot spot assessment
- Conclusion and outlook



Sketch of the CICC' conductor, Courtesy of ITER

EUROfusion supports programs to develop concepts for the fusion power demonstrator plant DEMO. In this framework, design activities are led at CEA for providing a Toroidal Field (TF) winding pack compatible with DEMO plant requirements.

The CEA proposal design is mainly based on the ITER experience regarding design and qualification tests.

Main criteria

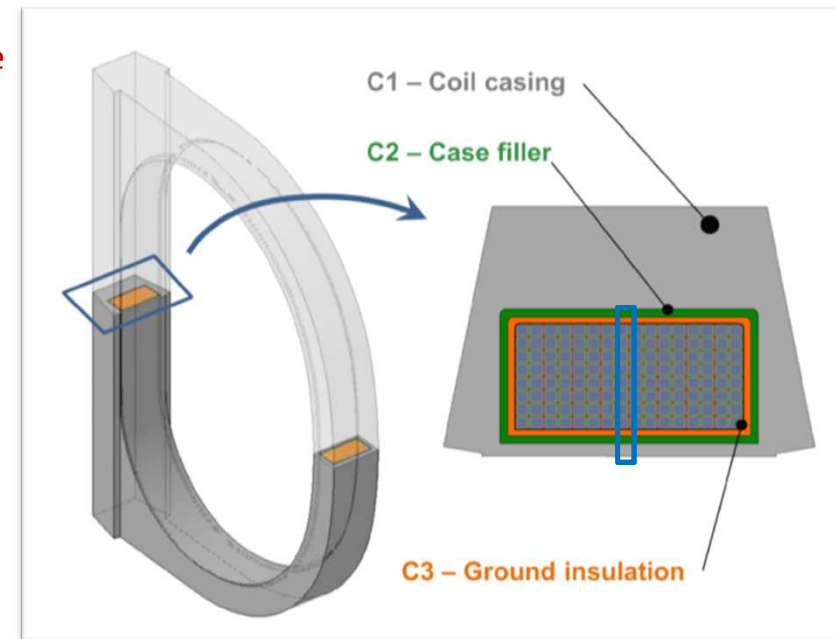
$$\Delta T_{CS} = T_{CS} - T_{op} > 1.5 \text{ K}$$

$$T_{\text{Hot Spot}} < 150 \text{ K}$$

From the specifications of the winding pack given by Process system code.

The future DEMO magnets proposed by CEA are likely to feature cable-in-conduit conductors (CICC) (the most successful from the stability point of view) cooled by forced flow of supercritical helium.

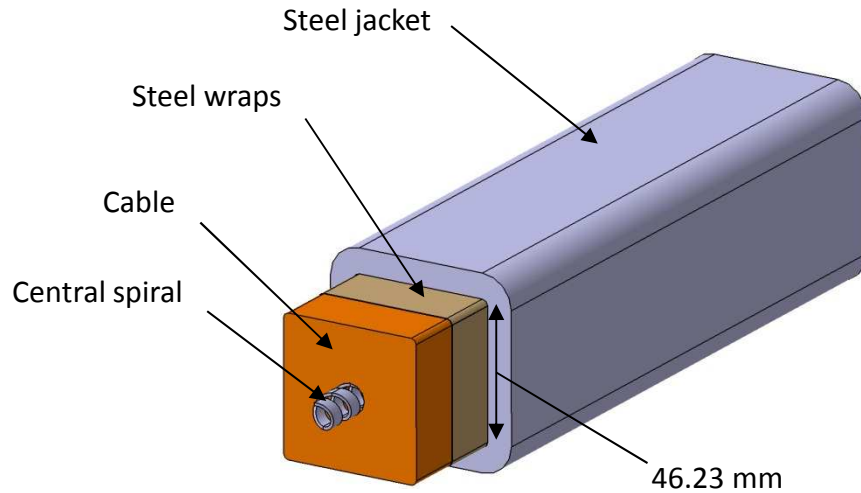
Thermo-hydraulic analyses contributes to assess the conceptual conductor design and focuses the study on the most critical pancake, the central where the maximum magnetic field is reached



View of TF coil and its associate winding pack cross section

For each of the 16 D-shaped winding (2x392 m long), 10 double-pancakes wound in 10 turns with hydraulic length of 392 m.

CEA CONDUCTOR: CABLE MAIN FEATURES



The conductor is a square-shaped Nb₃Sn double channel with a central spiral

All conductors will be cooled by supercritical helium that will flow in the central channel at high speed and at lower speed in the bundle region.

In the normal operation, the central channel increase the heat removal rate
During the quench, it prevents over high pressure raise.

SIMULATION of one conductor on median pancake that is the most critical one

TFEU4 OST Nb₃Sn strand

1392 SC + 294 pure Cu strands

Nominal current I_{op} = 95.5 kA

B_{eff} = 13.09 T

T_{cs} = 6.25 K

void fraction = 29 %

Double pancake design

→ hydraulic length = 392 m per pancake

Electrical model

$$(\epsilon_{\text{eff}}, n_{\text{eff}}) = (-0.66 \%, 6)$$

Annular bundle region

$$\Phi_{\text{strands}} = 1.024 \text{ mm}$$

$$\text{Area}_{\text{He}} = 594 \text{ mm}^2$$

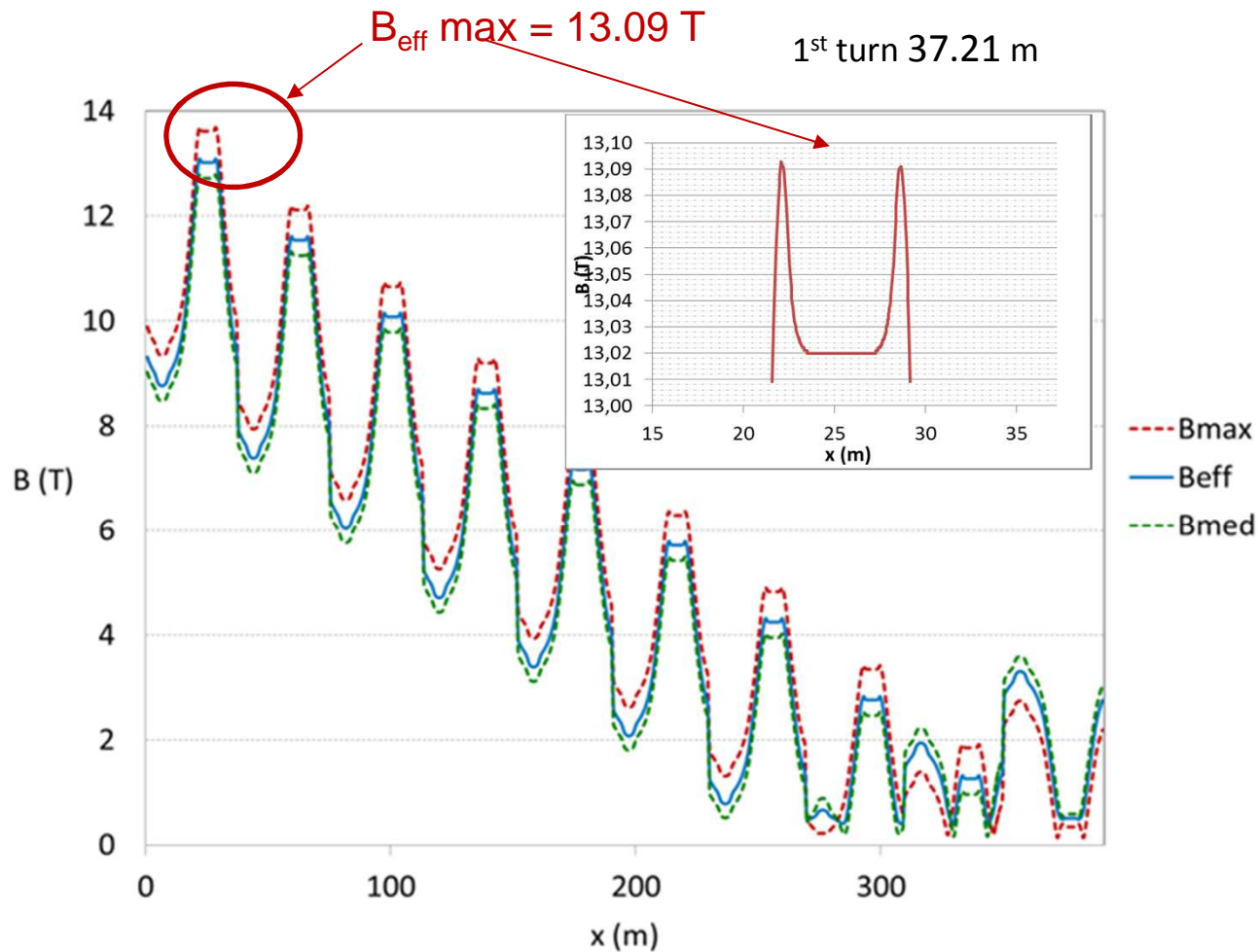
$$\Phi_h = 0.48 \text{ mm}$$

Central spiral

$$\Phi_{\text{in}}/\Phi_{\text{out}} = 8 \times 10$$



The conductor is subjected to the Effective Magnetic Field calculated with TRAPS code



$B(x) \longrightarrow T_{CS}(x)$

$T_{CS} \text{ min} = 6.25 \text{ K}$

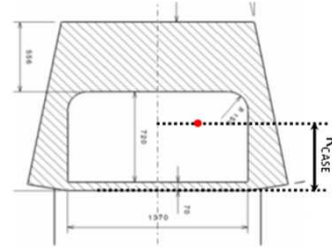
Courtesy of A. Torre et al.

Neutron Heat (NH) load distribution

380 W per coil

Neutrons heating contribution (W/m³)

$$P_{NH} = 50 \cdot e^{(-R_{CASE}/140)} \text{ W/m}^3$$



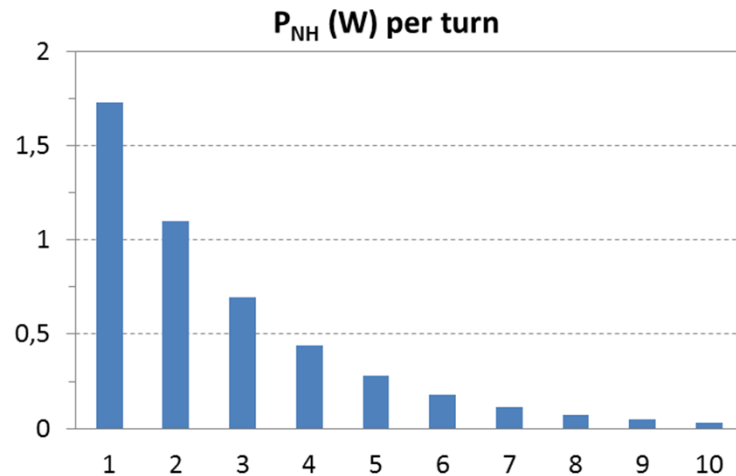
NH map in TF coil section across radial section

Conservative approach : P_{NH} constant in poloidal direction

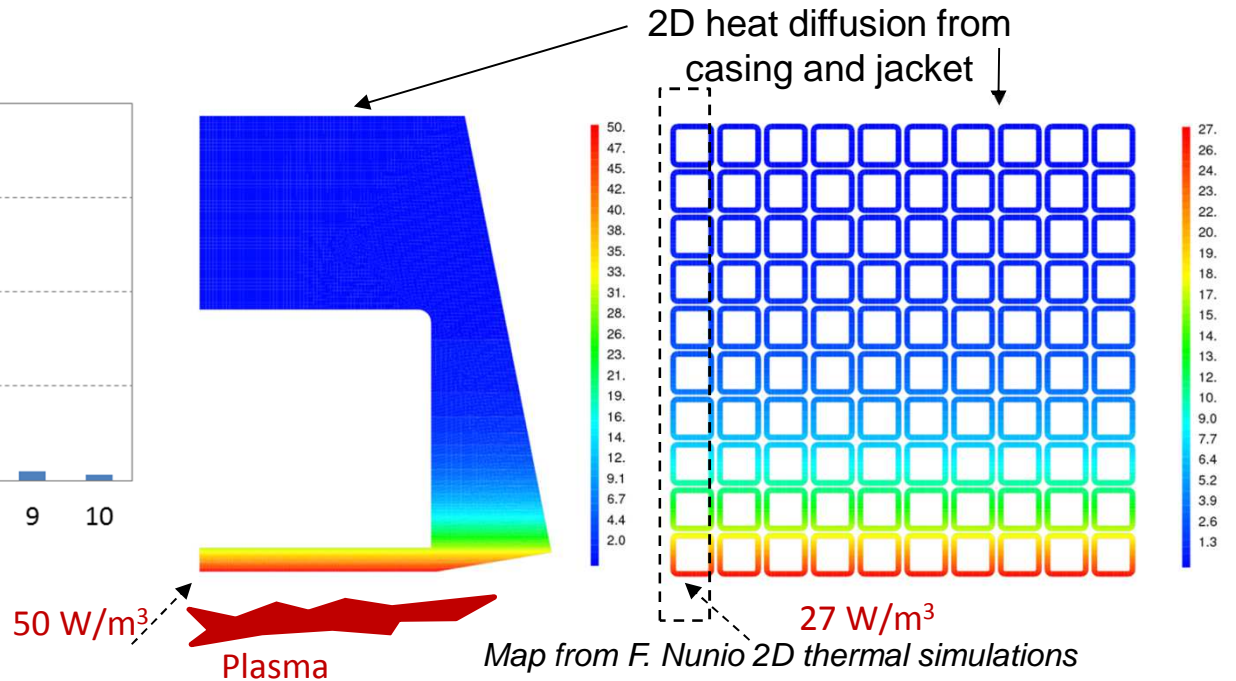
Nuclear power deposited directly in cable



Nuclear power deposited in casing

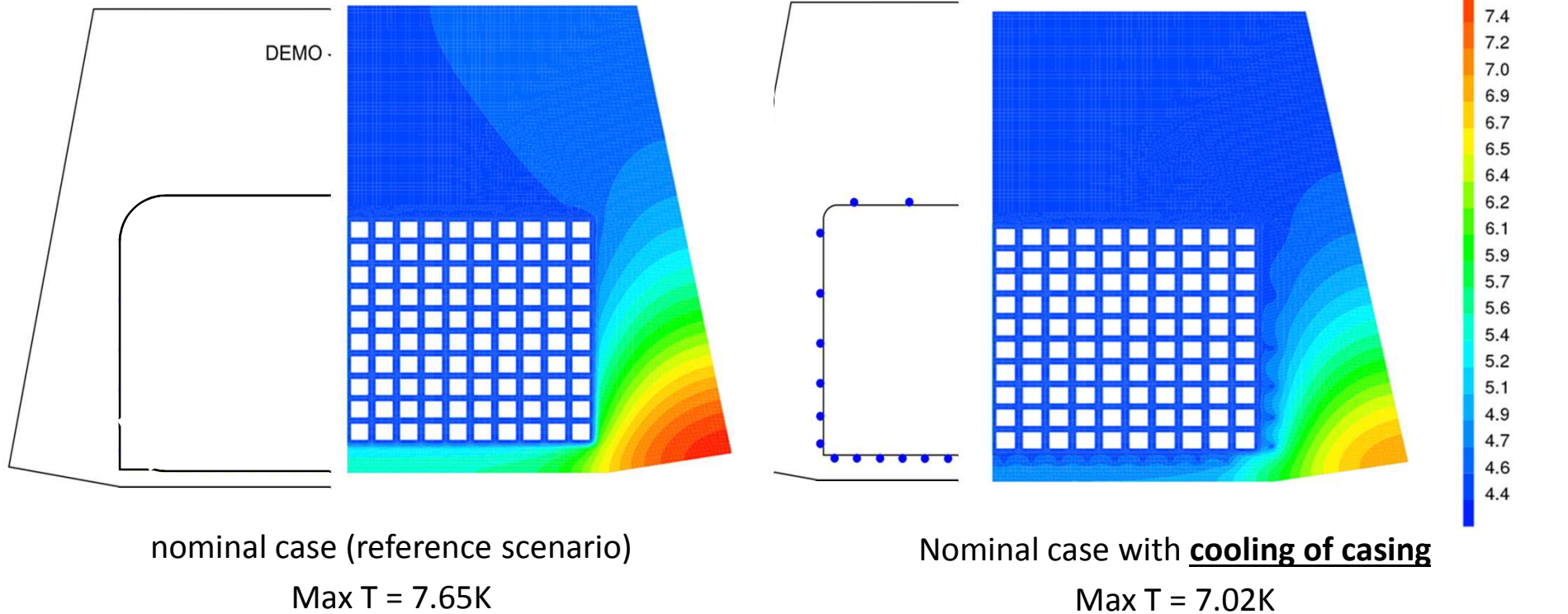


TF front case, 1st turn, 2nd turn ...
Gradient over TF, radial decay
Courtesy of L.Zani et al.



Thermal analysis (2D thermal model): Influence of cooling channels in casing

Thermal analysis of TF section with/without cooling channels of casing built up with Cast3M code



Output data: [Temperature](#), average heat load variation with time on all individual conductors

This cooling effect is included in the [thermohydraulic analysis](#) as an input data

Improvement:

$\Delta T = 0.63$ K

Cryosoft **THEA** is computer program for the **Thermal, Hydraulic and Electric Analysis** of superconducting cables that computes the evolution of **the temperature, coolant flow** and **current distribution** in a cable [1] both in steady state (**BURN**) scenario and transient (**QUENCH**) modes.

1D model:

- Thermal components as **strands and jacket**
- Hydraulic components: **compressible flow (P, T, v)** in a channel
 - Heat exchange with the wall
 - Mass, momentum and heat exchange with other channels
- Electrical components, current diffusion among resistive and inductive current carrying materials (II)

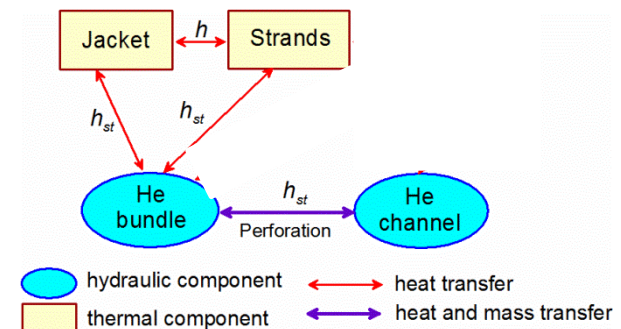
Implicit coupled analysis
User controlled configuration

Simplified 1D steady state model [2]:

- Assumed heat deposition profile along a cable
- Instantaneous transverse heat transfer between the cable components
- Steady state energy and momentum balance equations for compressible flow



mass flow rates and temperature profile along a cable



[1] L. Bottura et al. , A numerical model for simulation of quench in the ITER magnets, Journal of Computational Physics 125, 26-41, N°0077

[2] M. Lewandowska, K. Sedlak, Thermal-Hydraulic Analysis of LTS Cables for the DEMO TF Coil, IEEE Transactions on Applied Superconductivity 24 (2014) 4200305

Main thermal-hydraulic parameters:

- $T_{\text{inlet}} = 4.5 \text{ K}$
- $\Delta P = 6 - 5 = 1 \text{ bar}$
- Friction factor correlations
 - Annular bundle region:
 - Darcy-Forchheimer in bundle (conservative with void fraction = 29 %, geometry of the cable)
or
 - Katheder correlation
 - Central channel:
 - $f_{\text{EU spiral}} = 0.42 / \text{Re}^{0.1}$ (from measurements on 8/10 mm spiral)
- Heat transfer coefficients
 - $\text{Nu}_{\text{turbulent}} = 0.023 \text{ Re}^{0.8} \text{ Pr}^{0.4}$ (Dittus-Boelter correlation)
 - $\text{Nu}_{\text{laminar}} = 4$

New neutron heating load map presented previously

Normal (burn) and off-normal (quench) simulations:

Burn reference : Simulation of 2 h burn + 4 h dwell

Quench reference: Simulation of 60 s

The choice of a reliable correlation for the flow in the bundle region is not obvious:

One possibility is the Katheder correlation (fKath) that is based on the Katheder's experimental data base for the **void fraction ϕ from 0.37 to 0.47**.

One other is, the Darcy-Forchheimer correlation (**fDF**) based on the porous medium analogy model with an extensive database that included the results of pressure drop measurements in 23 CICC's with the **void fraction ranging from 0.25 to 0.365**

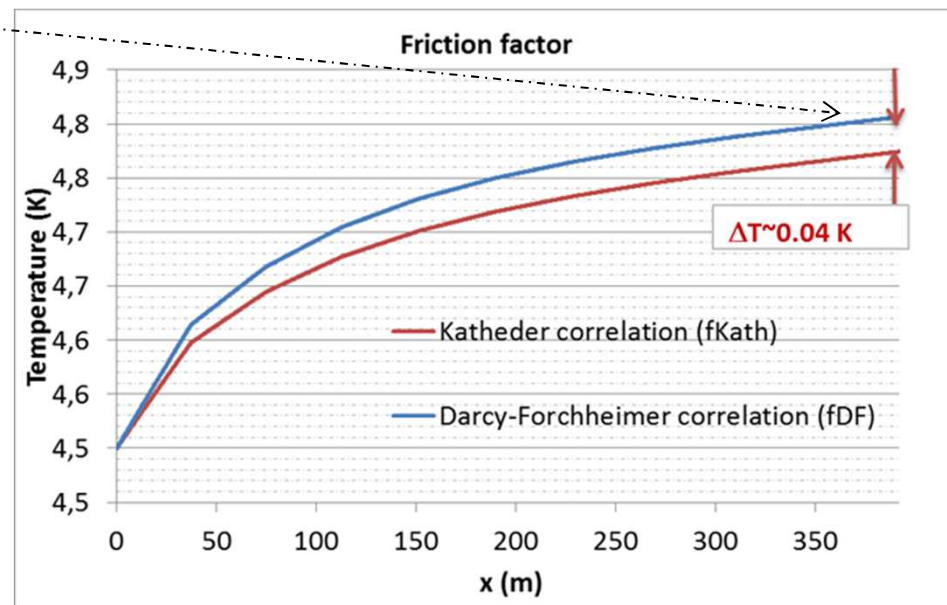
fKath correlation predicts much smaller friction factors than the fDF correlation which proposes a more conservative approach for the DEMO conductor.

DEMO conductor
void fraction $\phi = 29\%$

$$\text{fKath} \quad f(\text{Re}, \phi) = \frac{1}{\phi^\alpha} \left[\frac{\beta}{\text{Re}^\gamma} + \delta \right]$$

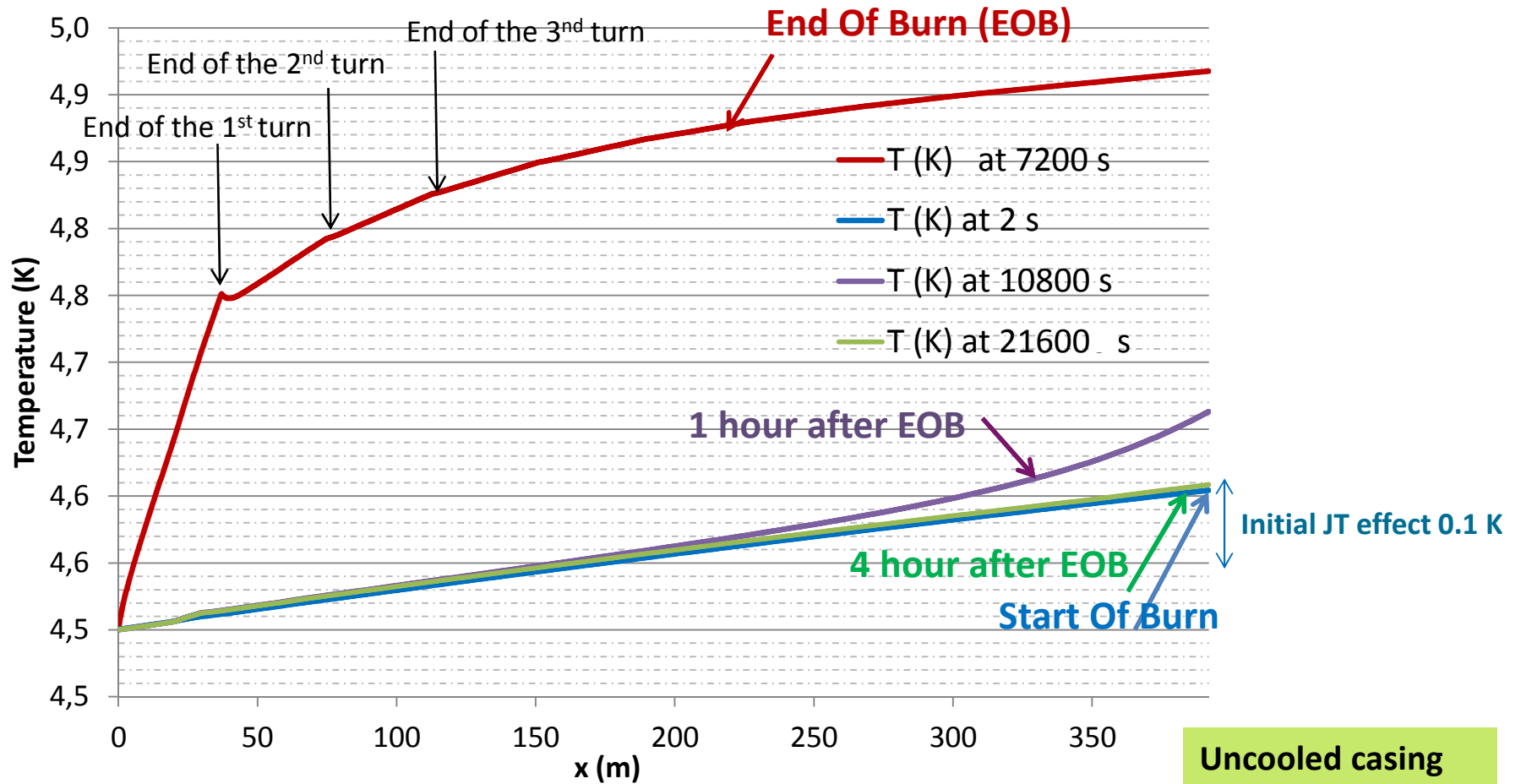
$$\text{fDF} \quad f = \frac{D_h^2 \phi}{2K} \frac{1}{\text{Re}} + \frac{D_h \phi^2}{2} \frac{C_F}{\sqrt{K}}$$

ϕ void fraction
Re Reynolds number
K permeability
 C_F drag factor
 D_h hydraulic diameter



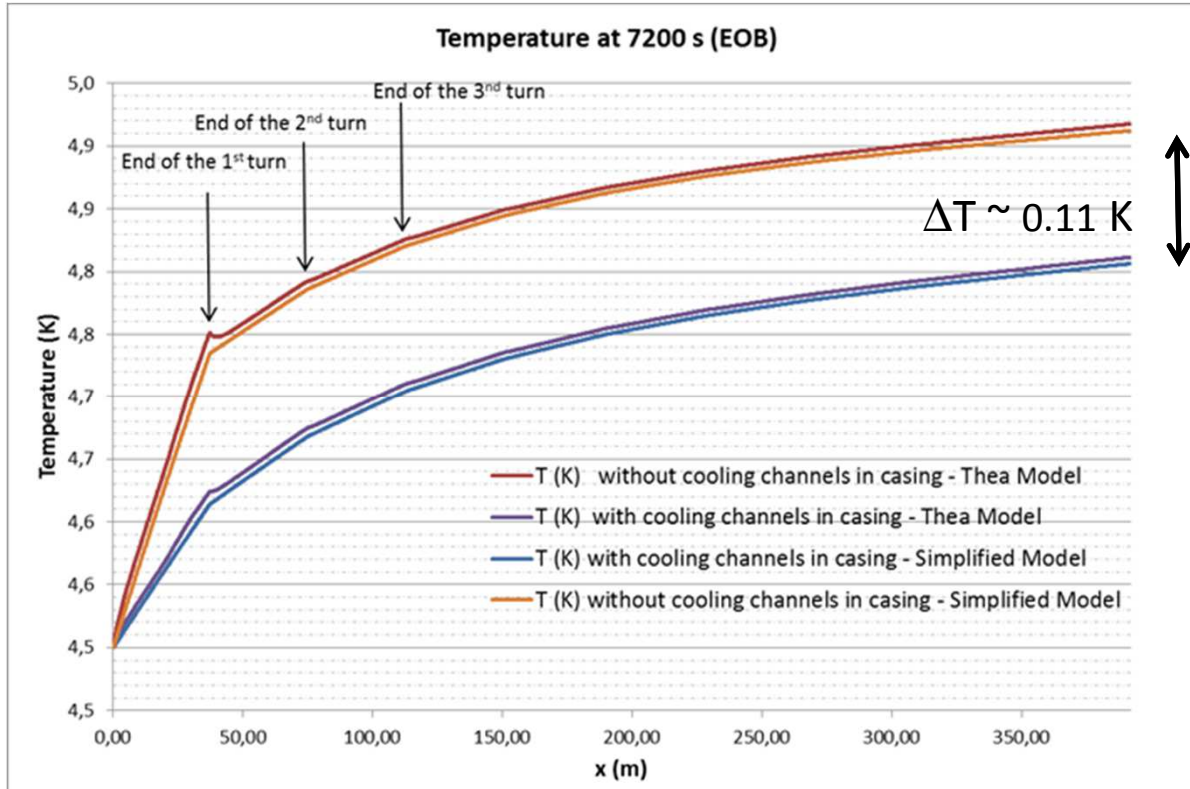
Mass flow (g/s) with f Kath ~ 28,4% greater	Bundle region		Central channel
	f Darcy-Forchheimer	f Katheder	
	5.57	7.77	5.85

Mass flow (g/s) f DF with cooling channels in casing	Bundle region		Central channel	
	THEA at the outlet of conductor	Simplified model	THEA at the outlet of conductor	Simplified model
	5.57	5.57	5.82	5.85



Conductor temperature as function of space and time

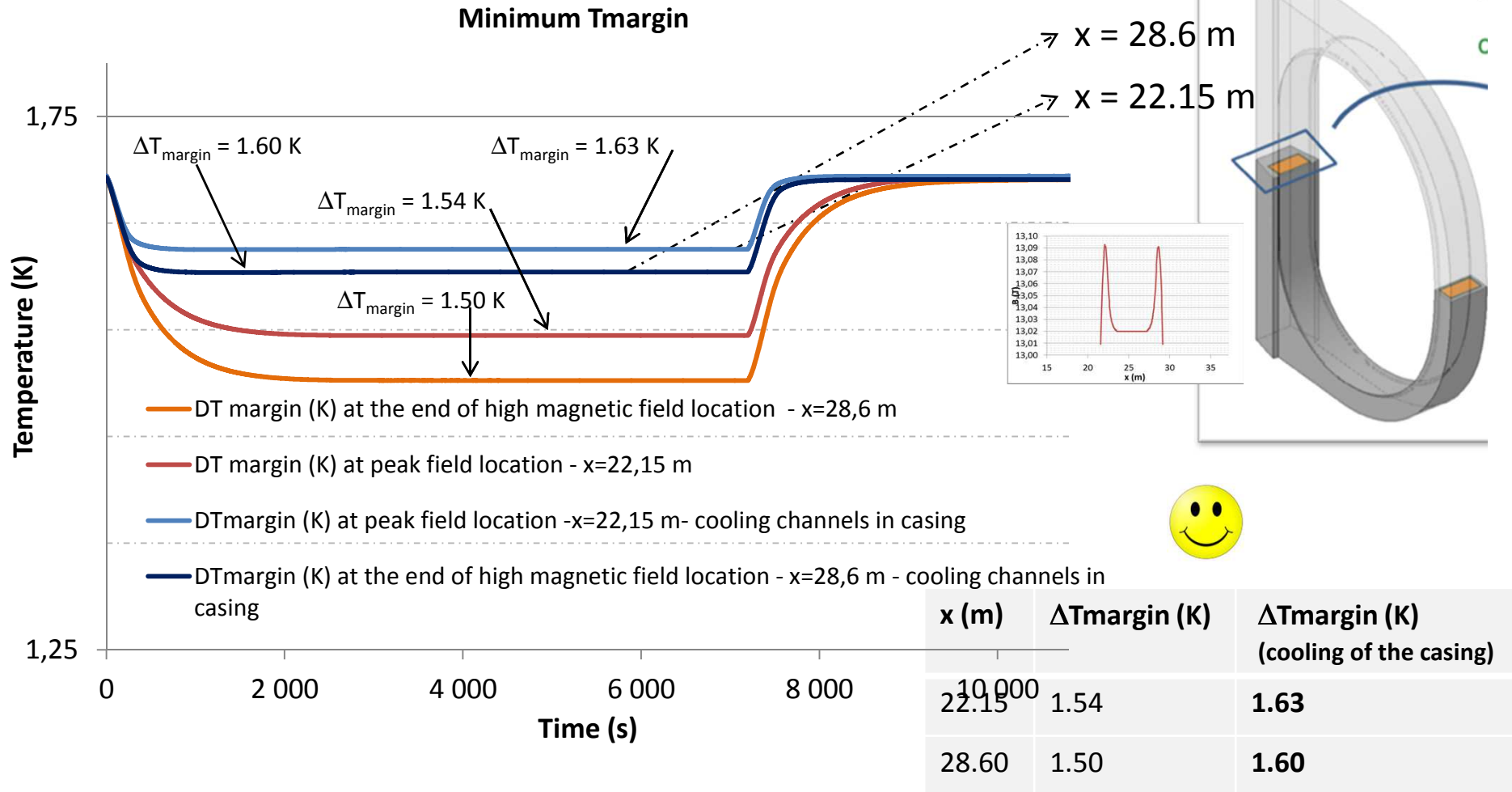
improvement due to the cooling channels in casing



THEA and Simplified model Friction factor fDF

Temperature at end of burn (K) (EOB)	Without cooling	With cooling	Cooling improvement
x=392 m			
Thea Model	4.918	4.811	0.11
Simplified Model	4.912	4.806	0.11

Temperature evolution over time



Related to the temperature ...

- ❑ Steady state simulation (burn scenario) -> good agreement between THEA and simplified model $\Delta T \sim 0.008 \text{ K}$
- ❑ Friction factor -> f Darcy-Forchheimr correlation is a conservative approach $\Delta T \sim 0.04 \text{ K}$
- ❑ Cooled casing
 - ✓ Temperature gain on casing $\Delta T \sim 0.63 \text{ K}$
 - ✓ And at the end of hydraulic length (x=392 m) $\Delta T \sim 0.11 \text{ K}$
- ❑ Gain on ΔT_{margin} (with/without casing cooling) $\Delta T \sim 0.03 \text{ K}$



Max. Tmargin = **1.63 K** while,
under the most pessimistic scenario
Min. Tmargin = **1.50 K**

Objective:

Analysis of quench behavior along the conductor and hydraulic circuit in one of the most realistic scenarios:

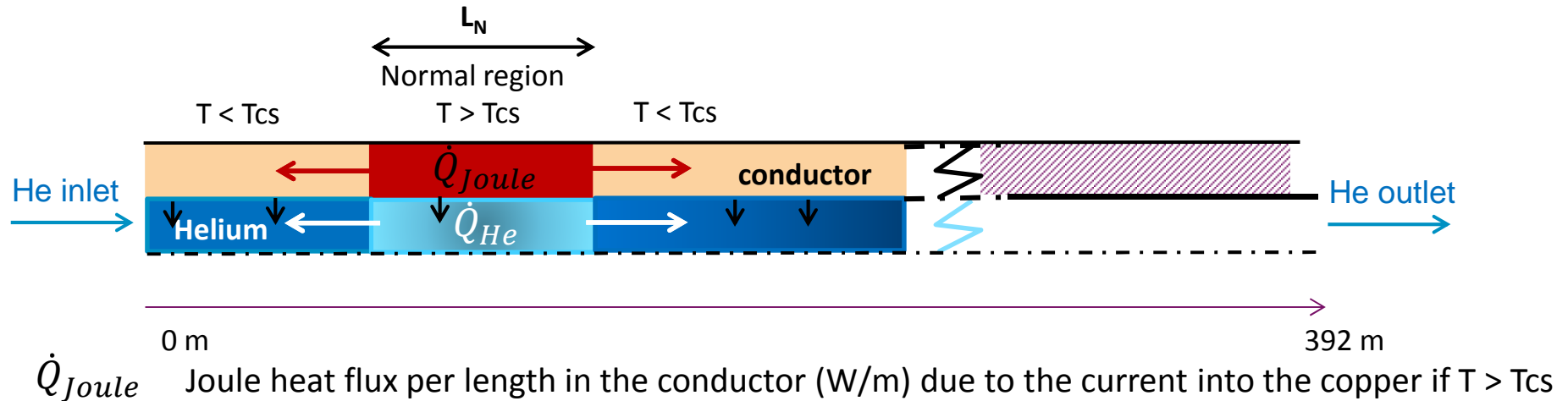
- ✓ Local heat deposition at the peak field where the margin in temperature is minimal and the internal layer exposition on plasma maximal

This scenario assess the hot spot temperature in conservative way.

Main numerical quench sequences:

- Quench initiation -> Research of the MQE
- Quench detection -> Voltage threshold, holding time
- Quench propagation -> Fast Discharge Method

Main heat tranfert between conductor and Helium



$\dot{Q}_{He} = hP(T - T_{He})$ Thermal power exchange by convection between the conductor and helium

Temperature front between the normal and superconducting region:

V_n : velocity of the normal zone

V_{He} : velocity of spelling induced by the warmed helium due to \dot{Q}_{He}

Quench is initiated locally at the peak field region by a minimum quench energy
This scenario is conservative for T hot spot assessment:

Minimum Quench Energy Power:

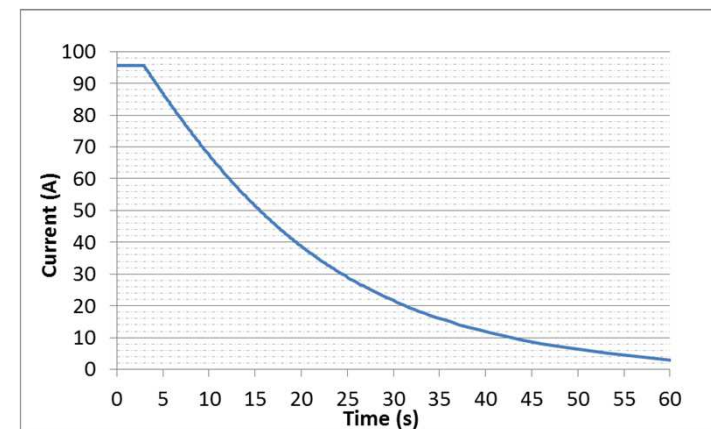
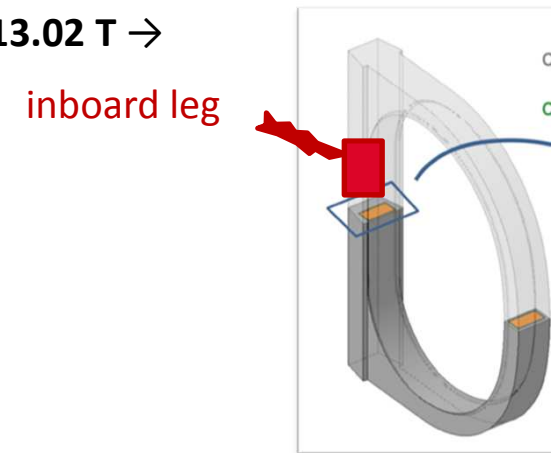
Perturbation of **100 ms** applied over **1 m** length (from 25 to 26 m) at **B = 13.02 T** →
MQE computed at 2700 W/m (12.07 mJ/cc of SC strand)

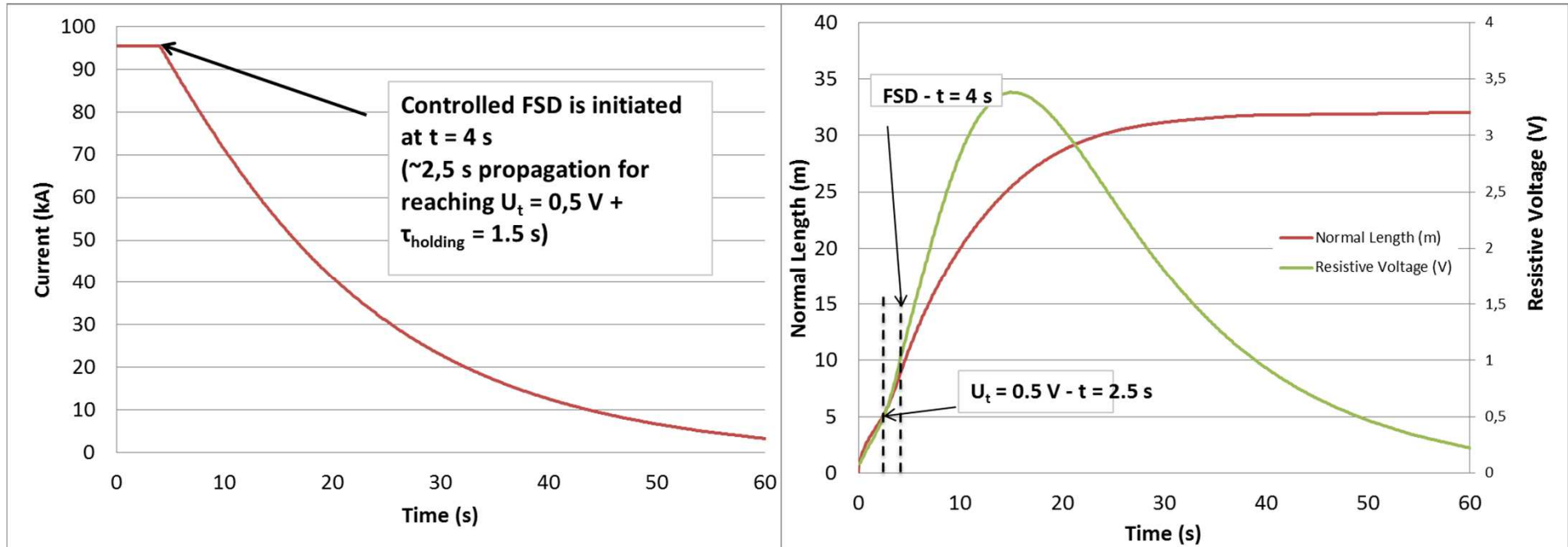
The quench detection is based on a resistive voltage threshold

- ✓ Voltage threshold $U_t = 0.5 \text{ V}$
- ✓ Delay time $\tau_{\text{delay}} = 1.5 \text{ s}$
- ✓ The Fast Safety Discharge (FSD)

is triggered after 4 s and the current decrease is controlled taking into account the heating of the dump resistor.

Normal zone is established by a 2x minimum energy input over limited length in the superconductor and propagate then.





Current evolution during detection and propagation phases

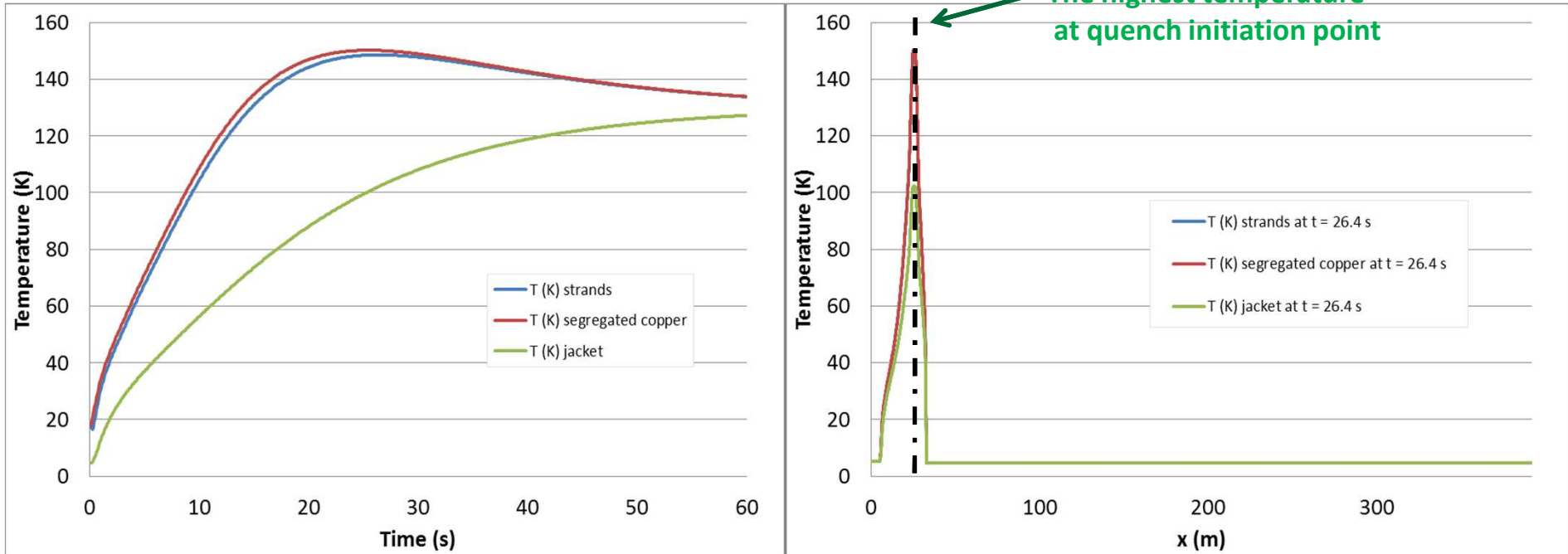


Normal zone starts to grow, it will continue to expand under combined actions of **Heat conduction** and **ohmic heating**

~5.22 m of quenched length at $t=2.5$ s for reaching $U_t = 0.5$ V

~8.56 m of quenched length at $t=4$ s for FSD

~32 m of quenched length at $t=60$ s

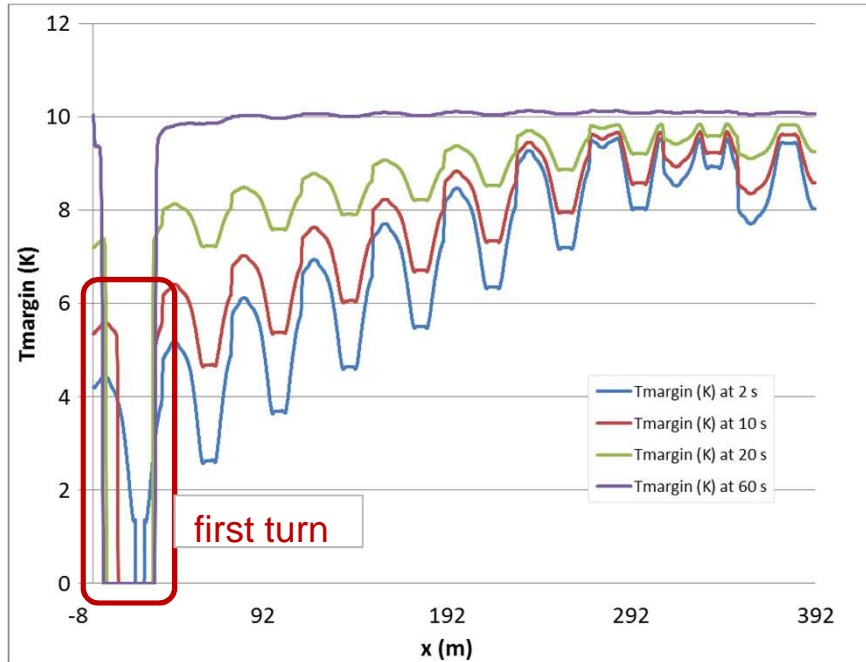


Temperature vs space and time

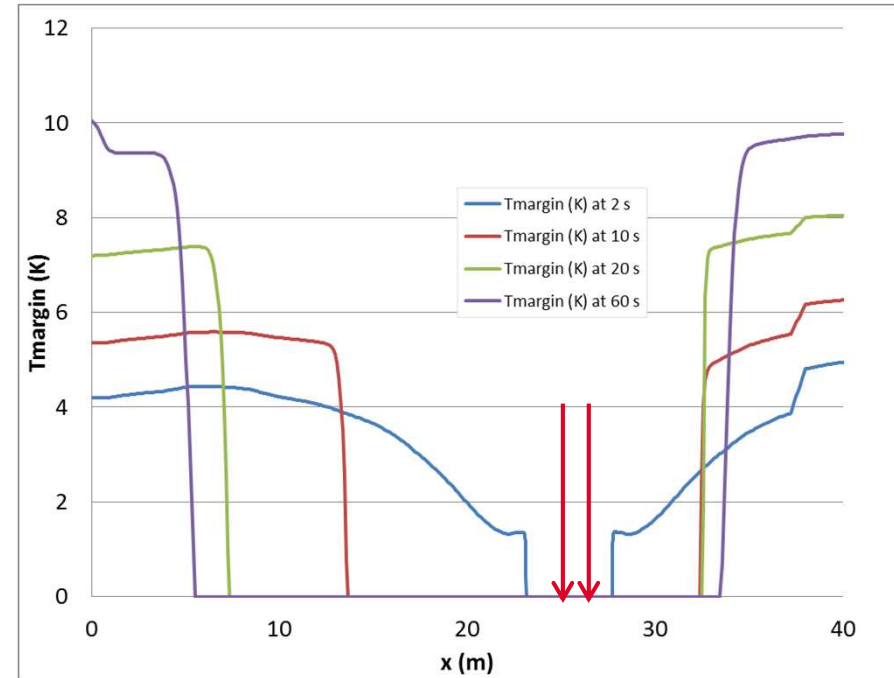
T hot spot = 148.6 K



Requirement is met



ΔT margin illustrates the Normal zone advancing along the conductor



Normal region at 60 s

Velocity from disturbed area is higher towards He inlet than towards He outlet. The disturbance location is closer to He inlet

- ❑ Transient thermo-hydraulic studies are performed on CEA conductor proposal for winding pack of DEMO TF magnet in the currently allocated space proposed by the PROCESS system code.
- ❑ Two models (THEA code and simplified model) are used, in complementary approach, to investigate the temperature behavior.
- ❑ Currently plasma **burn reference scenario** and one of the **realistic quench scenario** are analyzed for the most critical pancake, i.e. median one.

❑ **Burn scenario:**

The long pulse operation of 2 hours plasma burn and 4 hour Dwell is analyzed under proposed operating values and criteria. For this, it is used detailed electromagnetic and a new neutron heating load maps diffusing on the casing and falling directly to the conductor. In addition to that, analyses are led in order to assess the sensitivity of cooling channels in casing. **In all cases the temperature margin criteria of 1.5 K is respected, in particular in cooled casing case with ΔT_{margin} equal to 1.63 K.**

❑ **Quench scenario:**

The requirement for $T_{hot\ spot}$ is met ($149.2\text{ K} < 150\text{ K}$)

- ❑ In this first assessment of the thermo-hydraulic efficiency of the CEA conductor, the analysis shows than the thermohydraulic operating values can operate correctly.
- ❑ **In spite of** the fact that the design conductor proposes an acceptable thermohydraulic solution, the mechanical analysis ongoing could validate in a future the more realistic winding pack area.