

# Graphite and Titanium Alloy Radiation Damage Tests

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<sup>2</sup>Facility For Rare Isotope Beams

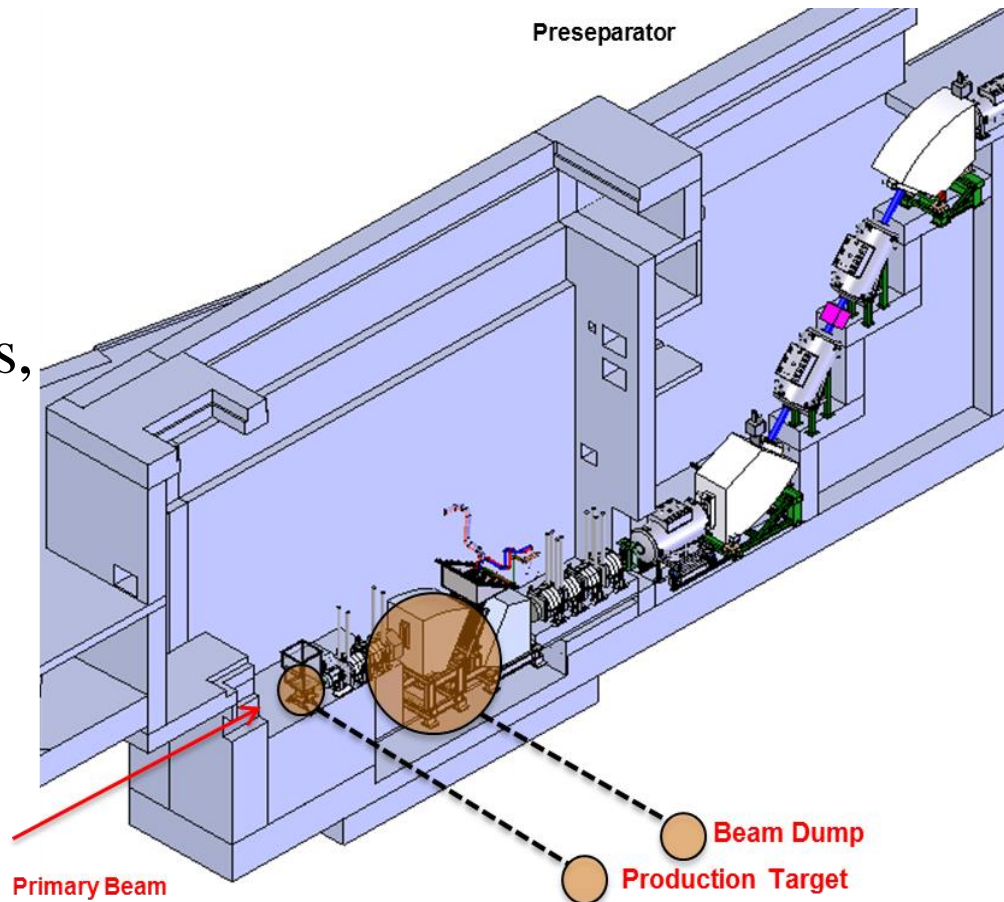


# Outline

- 1. Introduction**
- 2. Microstructure characterization**
- 3. Experimental Methods**
- 4. Irradiation damage in Ti-6Al-4V: Literature review**
- 5. Microstructure characterization**
- 6. Hardness measurements**
- 7. Conclusion**

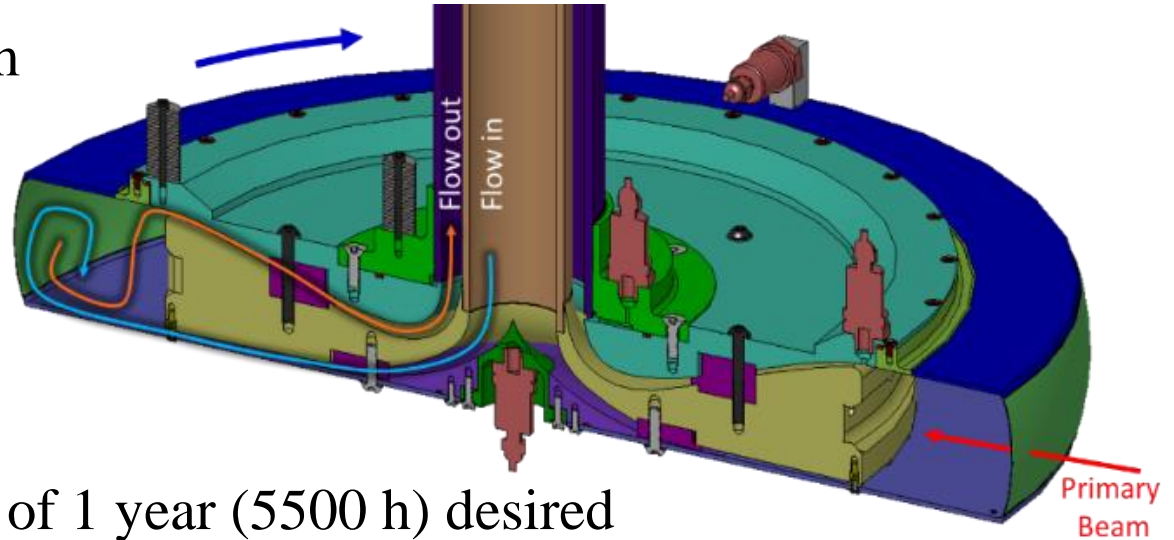
# Facility for Rare Isotope Beams at Michigan State University

- The FRIB at Michigan State University is a new generation accelerator with high power heavy ion beams.
- It will provide primary beams from O to U with an energy of 200 MeV/u for heavy ion beams, and higher energies for lighter beams.
- Beam Dump
  - Up to 325 kW



# FRIB Beam Dump

- Water-filled rotating drum beam dump chosen for FRIB baseline



- FRIB conditions:
  - Beam Dump lifetime of 1 year (5500 h) desired
  - Estimated cumulative dpa after one year of use ~9 dpa with a fluence of  $10^{15}$  ions.cm<sup>-2</sup>
  - Se from 0.08 keV/nm (with O beam) to 12.6 keV/nm (with U beam)
    - Ti-6Al-4V and Ti-6Al-4V-1B were chosen as candidate materials
    - The current study addresses the radiation damage challenge and focuses on understanding Swift Heavy Ion (SHI) effects on Ti-alloy that can limit beam dump lifetime

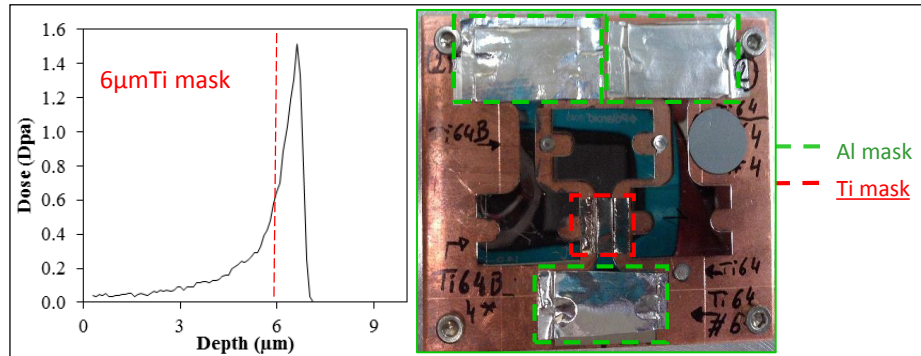


# Irradiation set up

- Two main irradiation experiments with Ti-6Al-4V and Ti-6Al-4V-1B samples were performed at the IRRSUD beamline facility at the GANIL-CIMAP Laboratory, Caen France.
- The IRRSUD beam line was chosen due to comparable  $S_e$  values to the FRIB conditions (0.08 -13 keV.nm<sup>-1</sup> ) without the activation of the sample (> coulomb barrier)

Beam	Energy (MeV/u)	Ranges ( $\mu\text{m}$ )	$S_e$ (keV.nm <sup>-1</sup> )	Temperature ( $^{\circ}\text{C}$ )	Fluence (ions.cm <sup>-2</sup> )
<sup>36</sup> Ar	1	6.8	7.5	25 - 350	10 <sup>15</sup>
<sup>131</sup> Xe	1.4	8.5	19.7	25 - 350	2-7. 10 <sup>14</sup>

**The SRIM-2013 calculation of the dose in a Ti-6Al-4V sample for the <sup>36</sup>Ar @36 MeV beam with a fluence of 10<sup>15</sup> ions.cm<sup>-2</sup>**



# Irradiation set up



## Ti-alloys irradiations at CIMAP and NSCL

Facilities	Beam	Energy [MeV]	Range [ $\mu\text{m}$ ]	$S_e$ [keV/nm]	Fluence [ions/cm <sup>2</sup> ]	Max dpa in sample	Date	Number of samples	Type
IRRSUD	<sup>82</sup> Kr	25	4.73	9.9	5.10 <sup>11</sup> - 5.10 <sup>12</sup> - 2.10 <sup>14</sup>	0.6	Jul-2013	6	Foils
	<sup>131</sup> Xe	92	8.5	19.7	2.10 <sup>11</sup>	0.001	Jul-2013	2	Foils
	<sup>82</sup> Kr	45	6.43	13.1	5.10 <sup>11</sup> - 5.10 <sup>13</sup>	0.16	Jul-2013	4	Foils
	<sup>82</sup> Kr	45	6.43	13.1	2.10 <sup>14</sup> 2.5.10 <sup>15</sup>	8	Oct-2013	6	Foils
	<sup>36</sup> Ar	36	6.8	7.5	10 <sup>15</sup>	1.5	Dec-2013	23	TEM and dogbone
	<sup>131</sup> Xe	92	8.5	19.7	2 10 <sup>14</sup> 7 10 <sup>14</sup>	3.5	June-2014	6	Dogbone
NSCL	<sup>40</sup> Ca	2000	800	1.5	6 10 <sup>12</sup>	10 <sup>-5</sup>	Aug-2013	1 x Ti64	Dogbone

### FRIB conditions

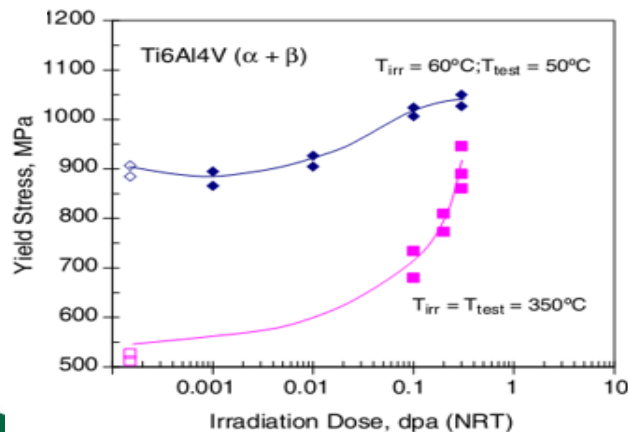
- Estimated cumulative dpa after one year of use ~9 dpa with a fluence of 10<sup>15</sup> ions/cm<sup>2</sup>
- $S_e$  from 0.08 keV/nm (with O beam) and 12.6 keV/nm (with U beam)



# Irradiation damage in Ti-6Al-4V

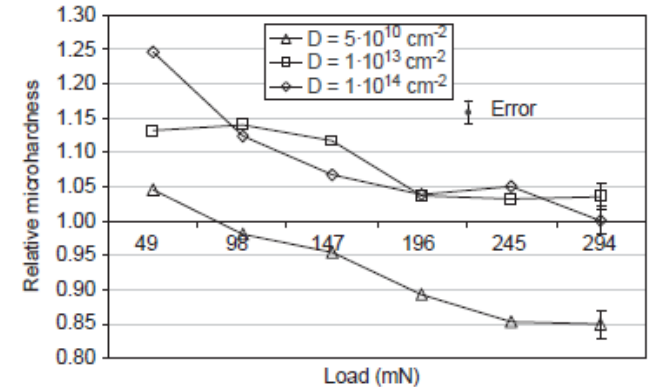
Effect of dose and temperature on the microstructure of **neutron** irradiated Ti-6Al-4V (Tähtinen *et al.*, Sastry *et al.*, Peterson)

Temperature and dose level	Microstructure change observations
50°C, 0.3 dpa	A high concentration of uniformly distributed defect clusters in the $\alpha$ -phase
350°C, 0.3 dpa	Dislocation loops Vanadium precipitates
450°C, Dose 2.1 and 32 dpa	Dislocation loops $\beta$ -phase precipitates in $\alpha$ phase
550°C 32 dpa	Extensive void formation Coarse $\beta$ -precipitates



**Different hardening mechanisms operate at 50°C than at 350°C.**

P. Budzynski, V. A. Skuratov, and T. Kochanski, “Mechanical properties of the alloy Ti-6Al-4V irradiated with swift Kr ion,” *Tribol. Int.*, vol. 42, no. 7, pp. 1067–1073, Jul. 2009.



**Relative micro-hardness in Ti-6Al-4V irradiated with swift 250MeV Kr<sup>+26</sup> at different fluences**



**Dose dependence of yield strength of Ti-6Al-4V irradiated with neutrons**

Tähtinen *et al.* / *Journal of Nuclear Materials*, 367-370 (2007), 627–632

Sastry *et al.* / *Fourth International Conference on Titanium*, Kyoto, Japan, 1980, vol. 1, p. p. 651.

D.T. Peterson, / *Effects of Radiation on Materials: 11th International Symposium*, Philadelphia, PA, 1982, p. p. 260.

# Microstructure of the as-received materials

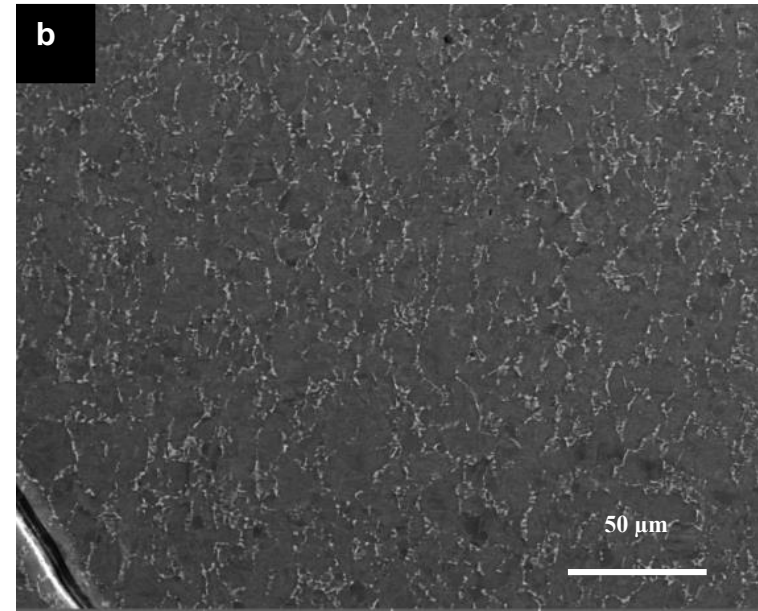
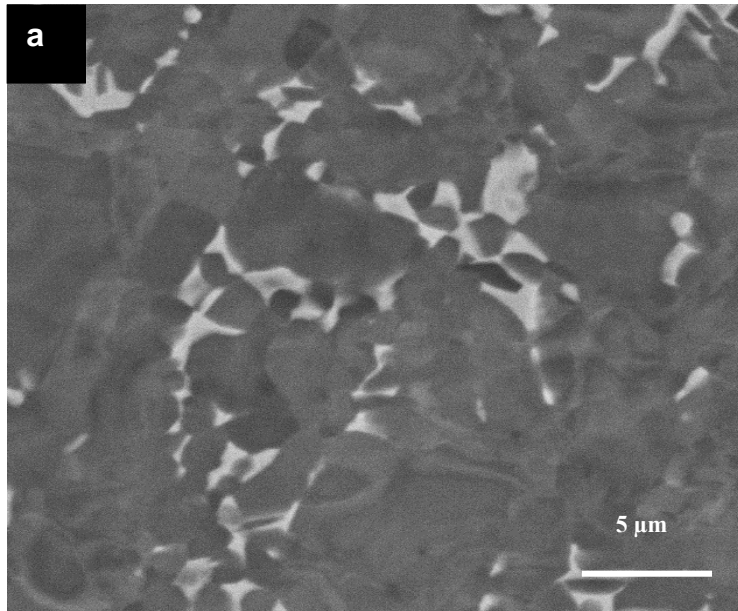


## Ti-6Al-4V:

Lenticular  $\alpha$ -phase with mostly an intergranular  $\beta$ -phase. Intra-granular  $\beta$ -phase was also observed.

The volume fraction of the  $\beta$ -phase was  $\sim 6.6$  vol.% and the  $\alpha$ -phase  $\sim 93.4$  vol.%.

The grain size of the  $\alpha$ -phase ranged between  $5 \sim 20 \mu\text{m}$ .



**BSE images of the initial microstructure of Ti-6Al-4V (a) higher and (b) lower magnification**



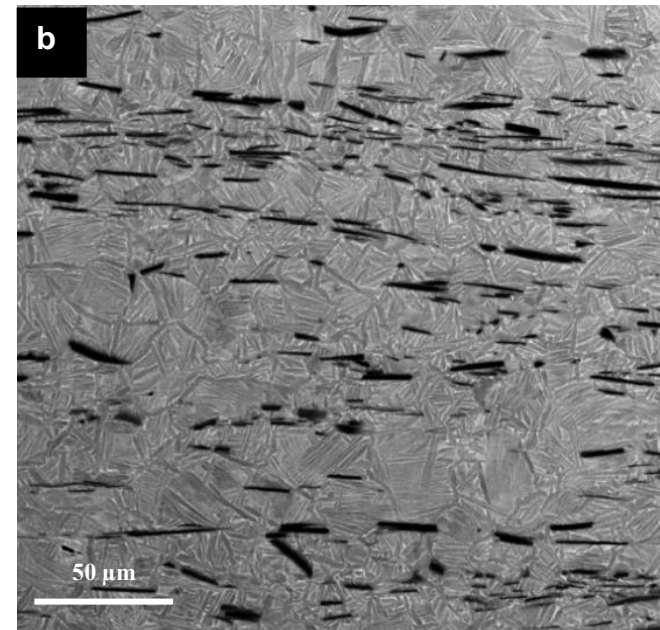
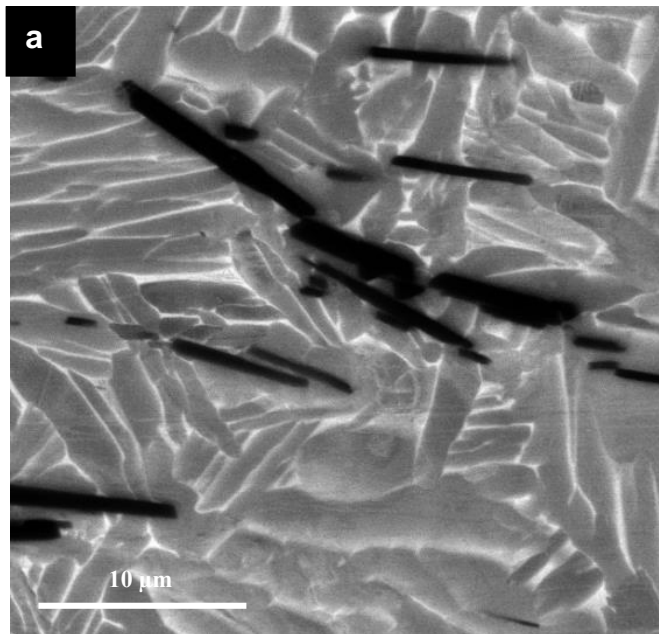
# Microstructure of the as-received materials



## Ti-6Al-4V-1B

The microstructure contained both an equiaxed ( $7.4\mu\text{m}$ ) and lenticular  $\alpha$ -phase; total volume percent  $\alpha$ -phase was  $\sim 79\%$ .

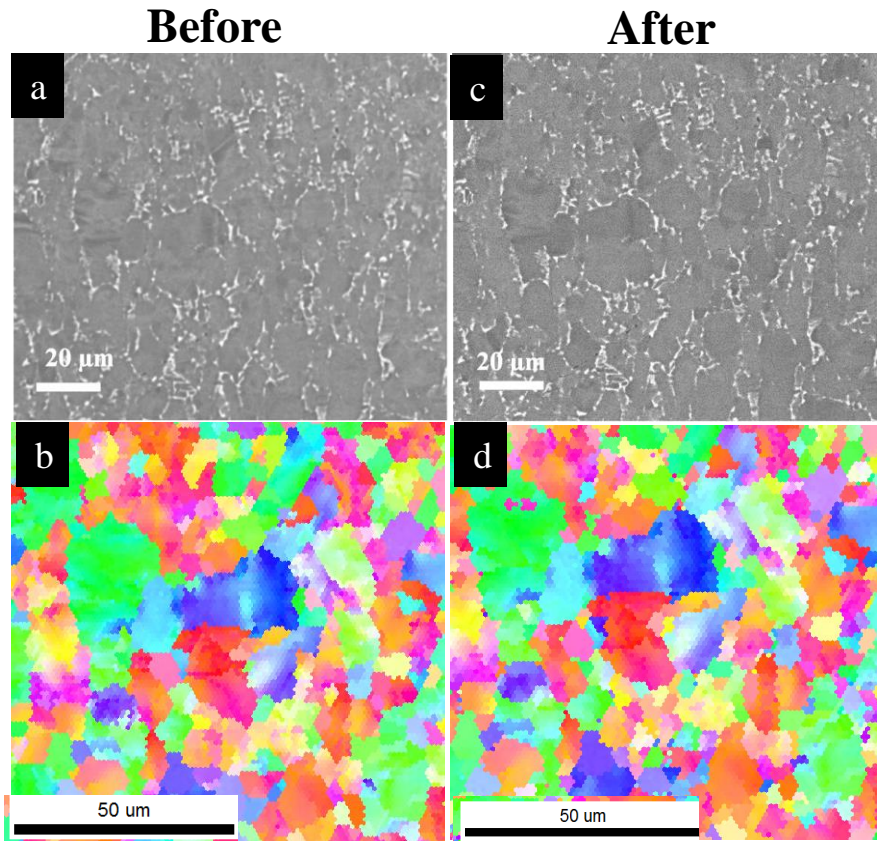
The  $\beta$ -phase volume percent was  $\sim 15$  vol.% while the TiB phase volume percent was  $\sim 5.9$  vol.%.(Chen *et al.*)



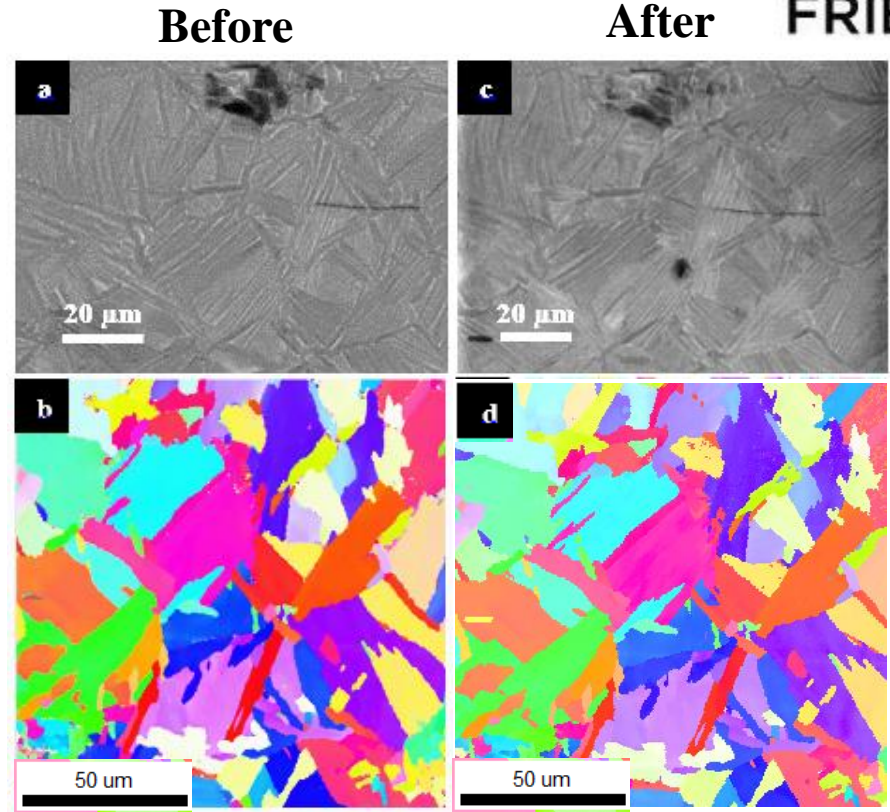
**BSE images of the initial microstructure of Ti-6Al-4V-1B (a) higher and (b) lower magnification**

W. Chen et al / *Key Eng. Mater.*, vol. 436, pp. 195–203, May 2010.

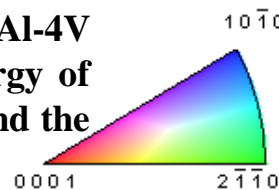
# Microstructure Characterization



BSE images and IPF maps before (a,b) and after irradiation at the same area (c,d) in a Ti-6Al-4V sample irradiated with  $^{131}\text{Xe}$  with an energy of 92 MeV. The fluence was  $2 \cdot 10^{14}$  ions. $\text{cm}^{-2}$  and the temperature 25°C



BSE images and IPF maps before (a,b) and after irradiation at the same area (c,d) in a Ti-6Al-4V-1B sample irradiated with  $^{36}\text{Ar}$  with an energy of 36 MeV. The fluence was  $1 \cdot 10^{15}$  ions. $\text{cm}^{-2}$  and the temperature 350°C



➤ No change in microstructure or grain orientation at the surface.



# Hardness measurements

## Nano-indentation

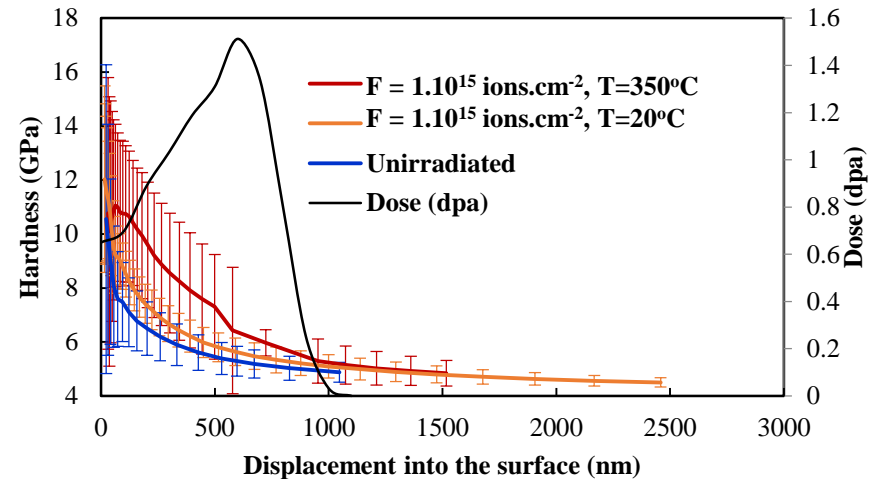
Obtain the properties of the materials in depth.



Parameters:

- Berkovich tip
- Strain rate :  $0.05\text{s}^{-1}$
- Poisson ratio=0.33
- Distance between indents:  $50\mu\text{m}$

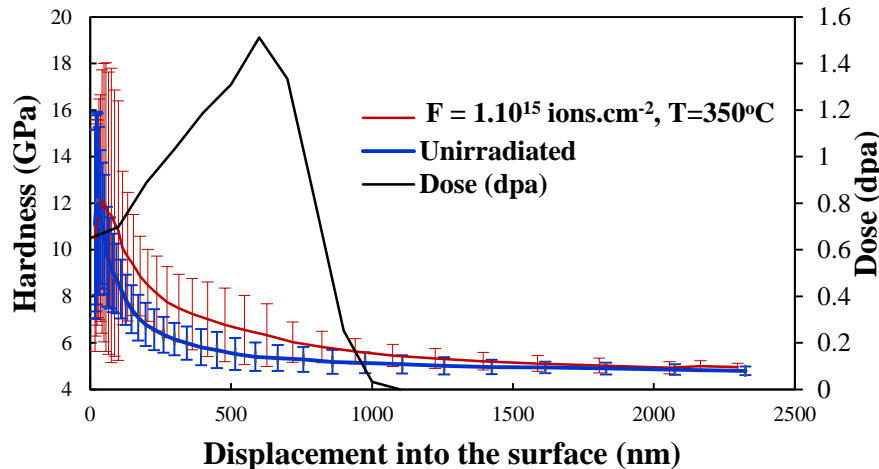
### Ti-6Al-4V



**Nano-indentation results for Ti-6Al-4V and Ti-6Al-4V-1B irradiated with  $^{36}\text{Ar}$  @36 MeV at fluence of  $1.10^{15}$  ions. $\text{cm}^{-2}$  with the CP-Ti foil on the surface.**

**Boron addition to Ti-6Al-4V did not change its irradiation resistance**

### Ti-6Al-4V-1B



- A slight increase in hardness observed for the sample irradiated with a higher fluence ( $1.10^{15}$  ions. $\text{cm}^{-2}$ ) and lower temperature ( $T = 350^\circ\text{C}$ ) for the higher doses

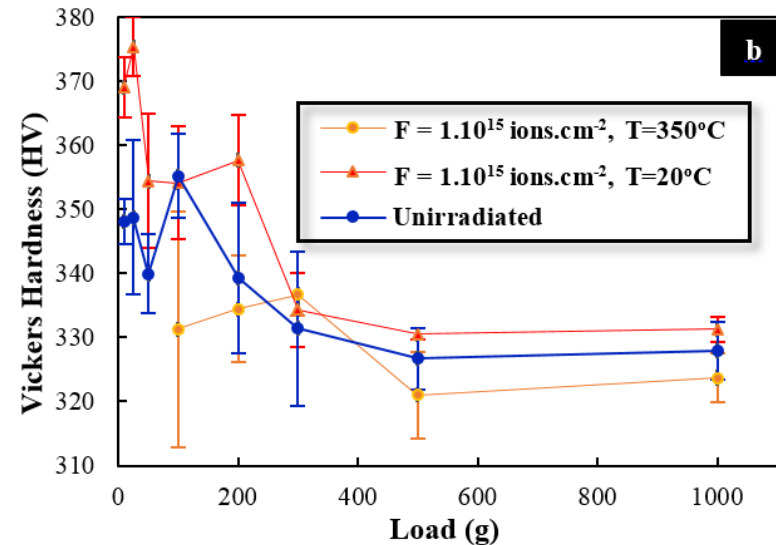
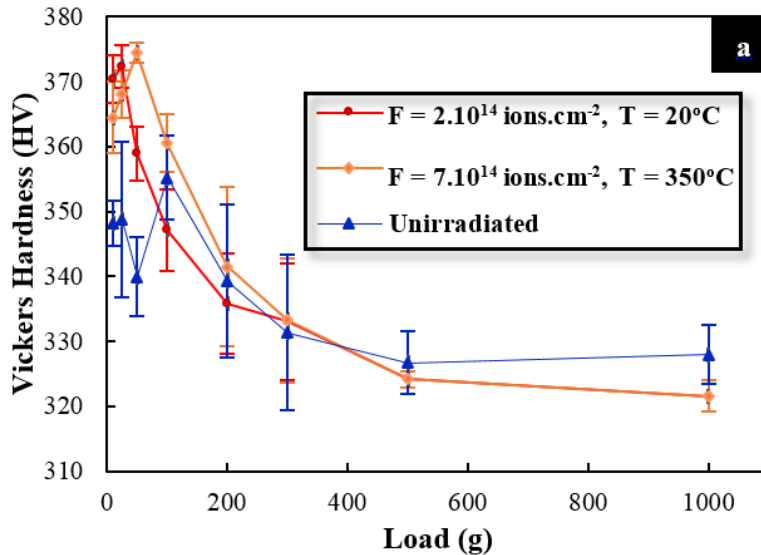


# Hardness measurements

## Vickers Hardness



- Vickers hardness was performed on 4 irradiated Ti-6Al-4V samples.



Vickers Hardness measurements for Ti-6Al-4V irradiated with: a)  $^{131}\text{Xe}$  @ 92 MeV and b)  $^{36}\text{Ar}$  @ 36 MeV

- The large scatter is due to the presence of two phases in the material
- A slight increase in hardness was observed for the sample irradiated with a higher fluence at lower loads ( $< 50\text{g}$ ) (depth  $\sim 1.6\mu\text{m}$ )

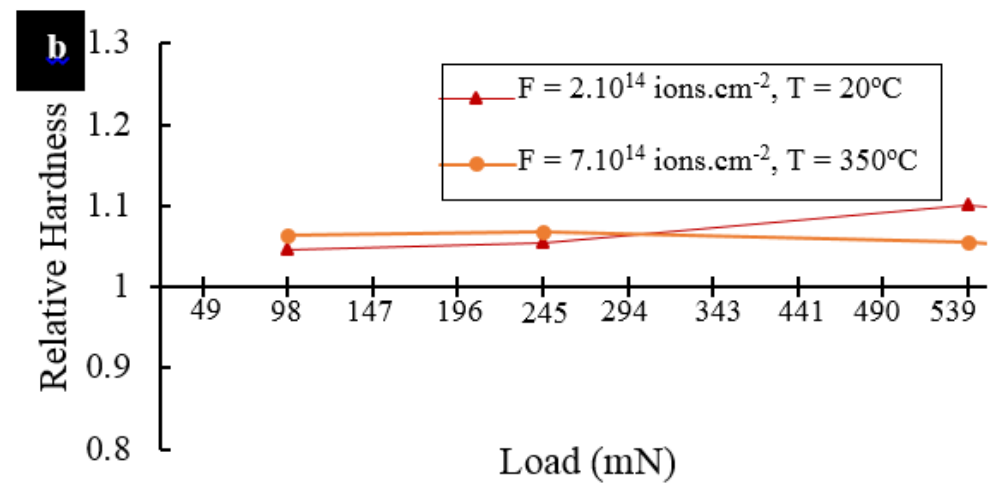
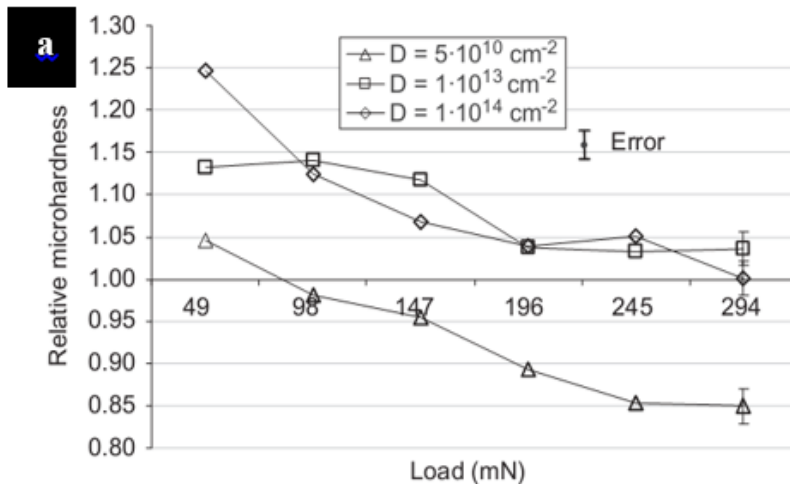


# Hardness measurements



## Vickers Hardness

- The lower irradiation damage observed in our investigated Ti-6Al-4V samples compared to results reported by Budzynski et al. (2009) could be explained by
  - The difference in microstructure: larger grains ( $\sim 100\mu\text{m}$ )
  - The grain size was  $5\text{-}20\mu\text{m}$  in our material and grain boundaries act as sinks for radiation-induced effects



**Relative micro-hardness of the Ti-6Al-4V alloy as a function of applied load for Ti-6Al-4V irradiated with: a) Kr<sup>+26</sup>@350 MeV (Budzynski et al. 2009) and b) <sup>131</sup>Xe @ 92 MeV.**

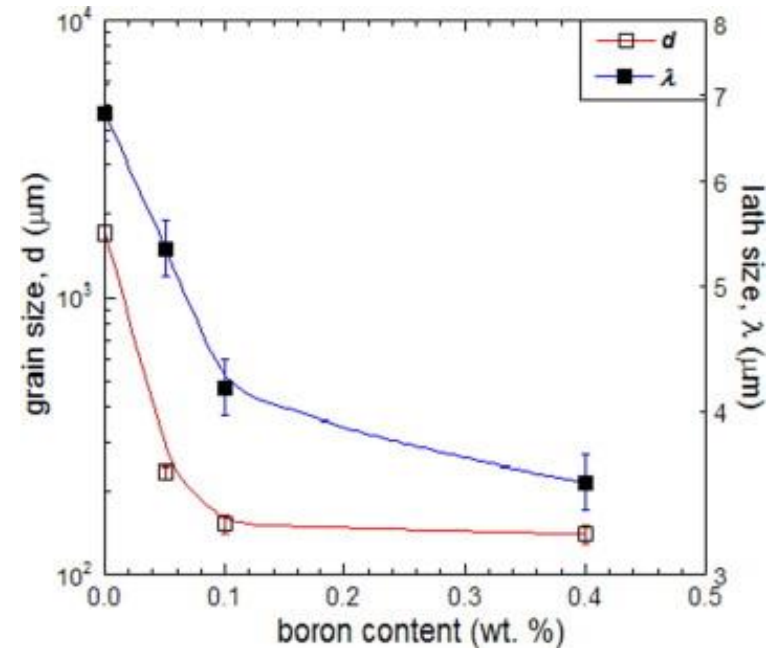


P. Budzynski, V. A. Skuratov, and T. Kochanski, "Mechanical properties of the alloy Ti-6Al-4V irradiated with swift Kr ion," *Tribol. Int.*, vol. 42, no. 7, pp. 1067-1073, Jul. 2009.

# Discussion



- Effect of the microstructure in the irradiation resistance of this Ti-alloy.
- Effect of the small grains (5-20 $\mu\text{m}$ )
- Boron addition causes grain refinement
- Thermomechanical processing can improve its properties



Variation of prior  $\beta$  grain size,  $d$ , and the  $\alpha$  lath size,  $\lambda$ , in Ti64 with wt.% B addition (Sen *et al.*)



# Conclusion



- The analyzed hardness and nano-indentation suggest a higher irradiation damage resistivity in the two studied Ti-alloys than reported in literature for Ti-6Al-4V.
- Slight differences in the microstructure caused by the thermomechanical processing may be responsible for this difference.
- 1% boron addition to Ti-6Al-4V didn't degrade the radiation resistance
- Ongoing and Future work:
  - ❖ Irradiation creep test
  - ❖ In-situ tensile tests and slip trace analysis: Deformation mechanisms
  - ❖ X-ray diffraction: Investigate phase transformation
  - ❖ Effect of the microstructure on the irradiation damage in Ti-alloys



# Acknowledgements

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  - Mikhail Avilov
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  - Isabelle Monnet
  - Florent Moisy
  - Marcel Toulemonde







# Radiation Damage and Annealing in Graphite:

## Ways to Improve the Lifetime of Targets

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U.S. DEPARTMENT OF  
**ENERGY**

Office of  
Science

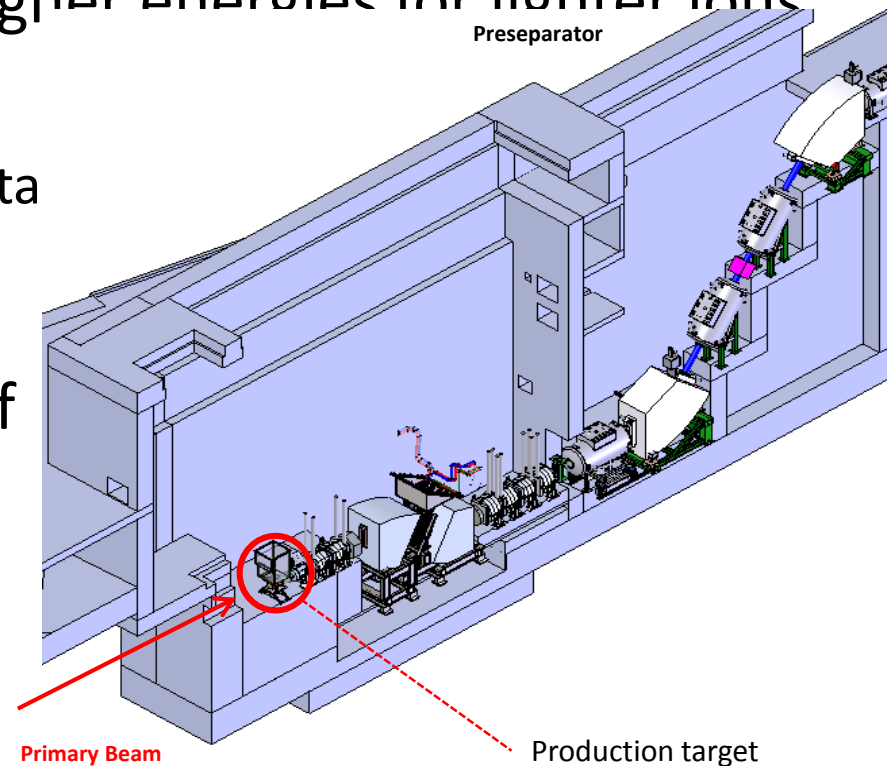
# Outline

- FRIB High-power production targets
  - Design and challenges
  - Irradiation and annealing studies of graphite
    - Temperature effect
- NSCL-FRIB stripper
  - Challenges
  - Irradiation and annealing studies of graphite
    - Temperature effects
- Conclusions

# High-Power Production Target

## Scope and Technical Requirements

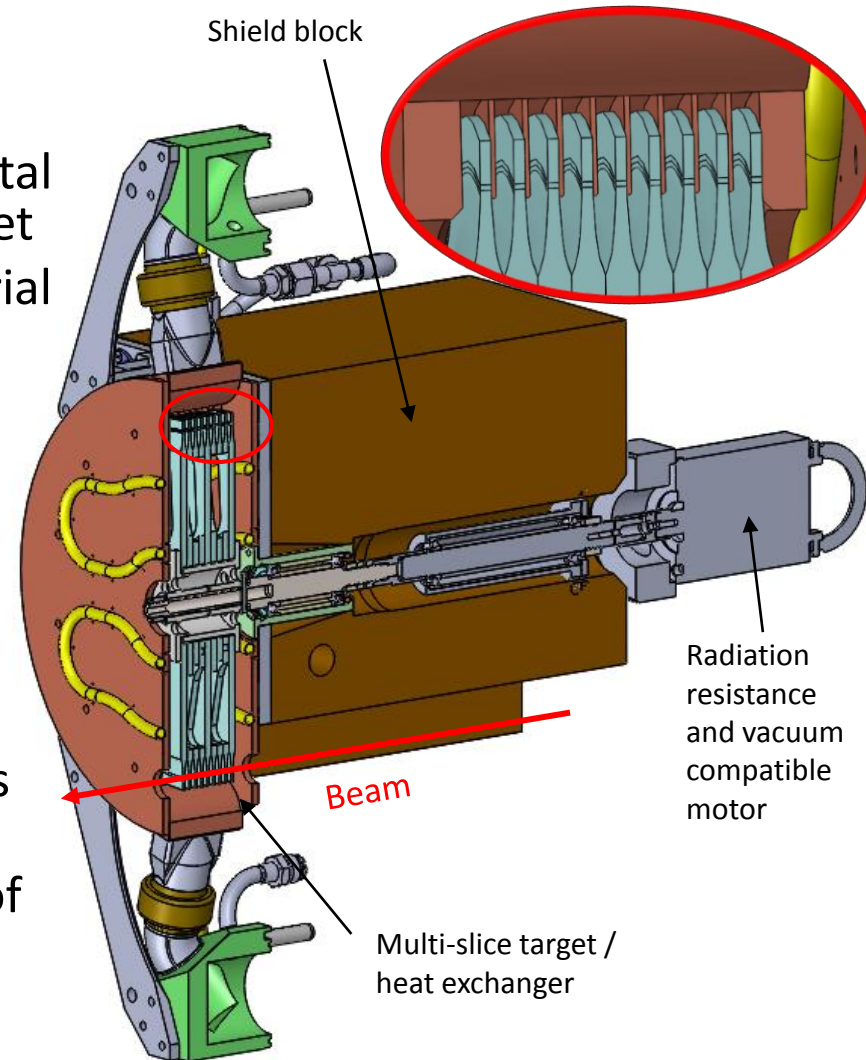
- In-flight rare isotope beam production with beam power of 400 kW at 200 MeV/u for  $^{238}\text{U}$  and higher energies for lighter ions
- High power capability
  - Up to 100 kW in a  $\sim 0.3 - 8 \text{ g/cm}^2$  ta isotope production via projectile fragmentation and fission
- Required high resolving power of fragment separator
  - 1 mm diameter beam spot
  - Maximum extension of 50 mm in beam direction
- Target lifetime of 2 weeks to meet experimental program requirements



# FRIB Production Target

## Rotating Multi-slice Graphite Target Design

- Rotating multi-slice graphite target chosen for FRIB baseline
  - Increased radiating area and reduced total power per slice by using multi-slice target
  - Use graphite as high temperature material
  - Radiation cooling
- Design parameters
  - Optimum target thickness is  $\sim \frac{1}{3}$  of ion range
    - 0.15 mm to several mm
  - Maximum extension of 50 mm in beam direction including slice thickness and cooling fins to meet optics requirements
  - 5000 rpm and 30 cm diameter to limit maximum temperature and amplitude of temperature changes



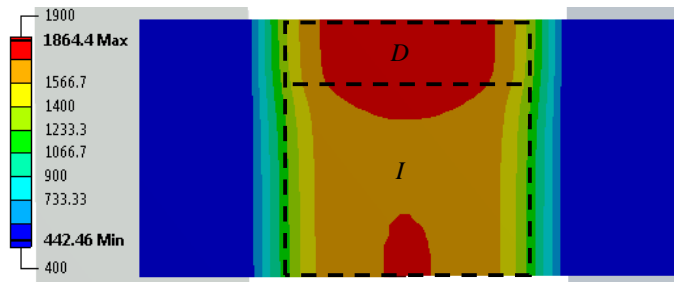
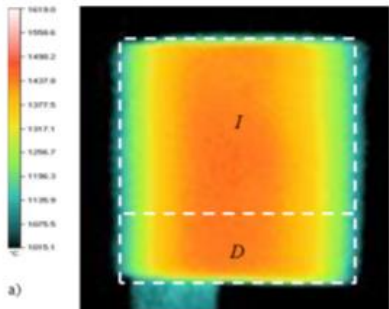
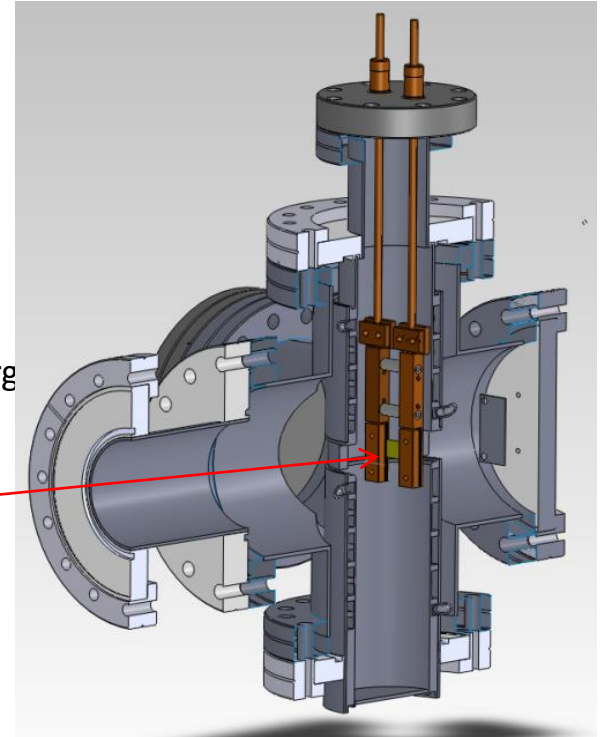
# FRIB Production Target

## Challenges

- Thermo-mechanical challenges
  - High power density:  $\sim 20 - 60 \text{ MW/cm}^3$ 
    - High temperature:  $\sim 1900 \text{ }^\circ\text{C}$ : Evaporation of graphite, stress
  - Rotating target
    - Temperature variation: Fatigue, Stress waves through target
- Swift Heavy Ion (SHI) effects on graphite
  - Radiation damage induce material changes
    - Property changes: thermal conductivity, tensile and flexural strength, electrical resistivity, microstructure and dimensional changes, ...
  - Swift heavy ions (SHI) damage not well-known
  - $5 \cdot 10^{13}$  U ions/s at 203 MeV/u may limit target lifetime
    - Fluence of  $\sim 9.4 \cdot 10^{18}$  ions/cm<sup>2</sup> and 10 dpa estimated for 2 weeks of operation
- Similar challenges at
  - Facility for Antiproton and Ion Research (FAIR) at GSI
  - Radioactive Ion Beam Factory (RIBF) at RIKEN

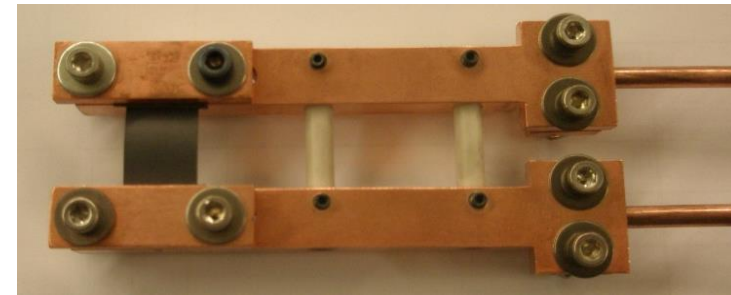
# Irradiation Test at UNILAC at GSI/Darmstadt

- Polycrystalline isotropic graphite
  - 2 Grades MERSEN 2360 (5  $\mu\text{m}$ ) / 2320 (13  $\mu\text{m}$ )
- Irradiation test at UNILAC at GSI/Darmstadt
  - Au-beam 8.6 MeV/u
    - Up to  $5.6 \cdot 10^{10}$  ions/cm<sup>2</sup>·s and fluence up to  $10^{15}$  ions/cm<sup>2</sup>
      - Equivalent to a fluence of  $10^{18}$  ions/cm<sup>2</sup> for FRIB beam energy or 2 days of operation
    - Electronic energy loss  $\approx 20$  keV/nm
  - Ohmic heating (up to 35 A, 250 W) of samples to different temperature during irradiation
    - $I = 35$  A
    - $I = 35$  A + beam



$T_{\text{max}} = 1480 (\pm 30) \text{ } ^\circ\text{C}$

$T_{\text{max}} = 1635 \text{ } ^\circ\text{C}$



# Radiation Damage Studies in Graphite [1]

Annealing of Damage at High Temperature (> 1300°C)

1 A - 350°C  
10<sup>14</sup> cm<sup>-2</sup>



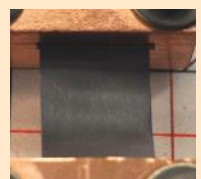
11 A - 750°C  
10<sup>14</sup> cm<sup>-2</sup>



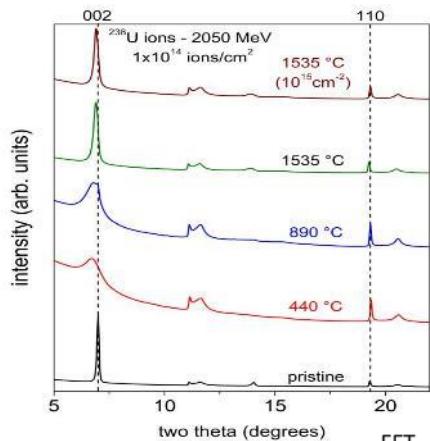
25 A - 1205°C  
10<sup>14</sup> cm<sup>-2</sup>



35 A - 1635°C  
10<sup>15</sup> cm<sup>-2</sup>

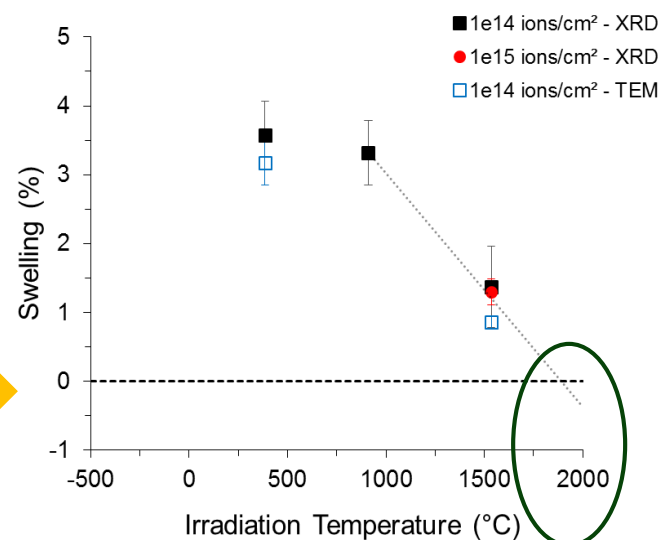
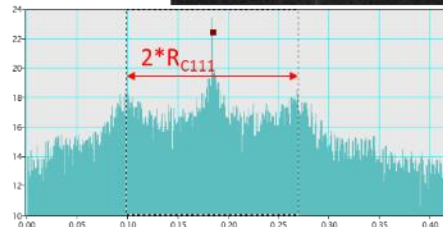
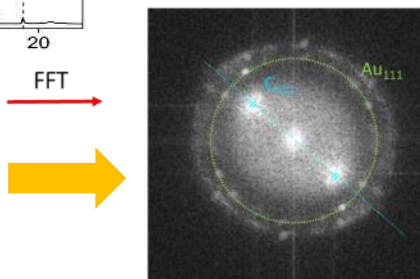
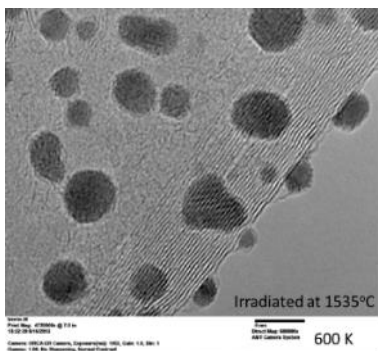


## X-Ray Diffraction analyses



FFT

## TEM analyses



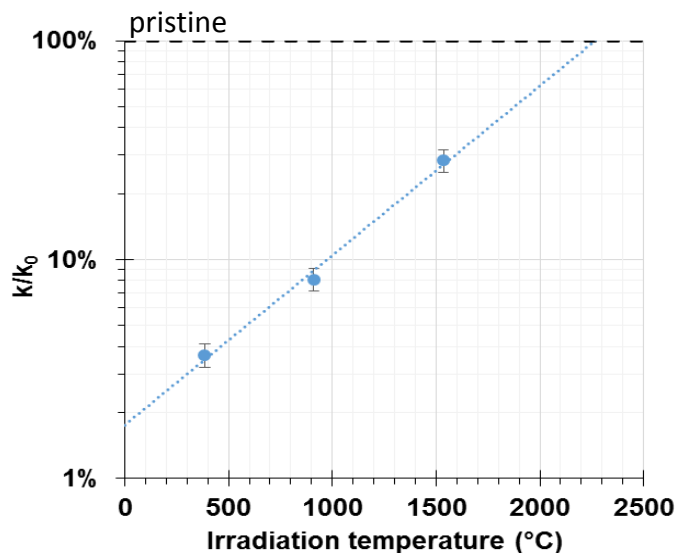
**Swelling is completely recovered at 1900°C**



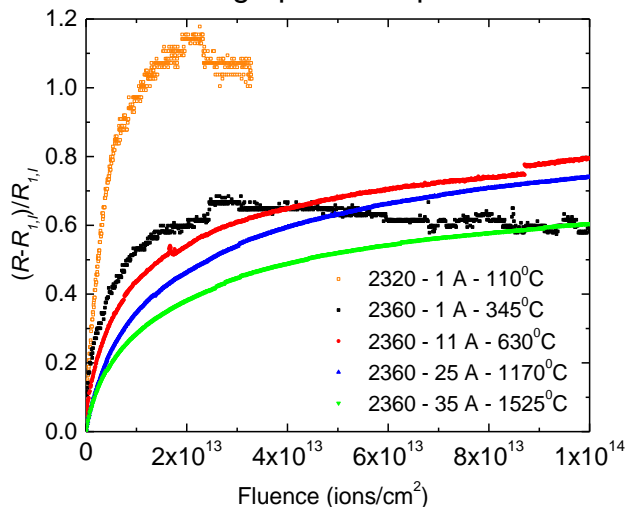
# Radiation Damage Studies in Graphite [2]

## Annealing of Damage at High Temperature ( $> 1300^{\circ}\text{C}$ )

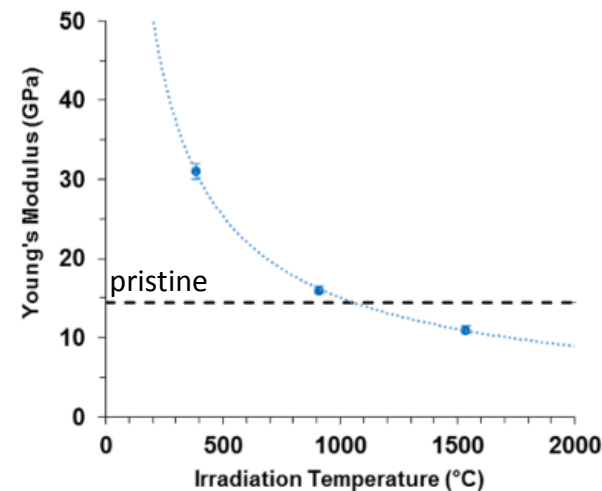
Thermal conductivity change of irradiated graphite samples -  $^{197}\text{Au}$  fluence  $10^{14}$  ions/cm $^2$



Electrical resistance change of irradiated graphite samples -  $^{197}\text{Au}$



Young's Modulus of irradiated graphite samples -  $^{197}\text{Au}$  fluence  $10^{14}$  ions/cm $^2$



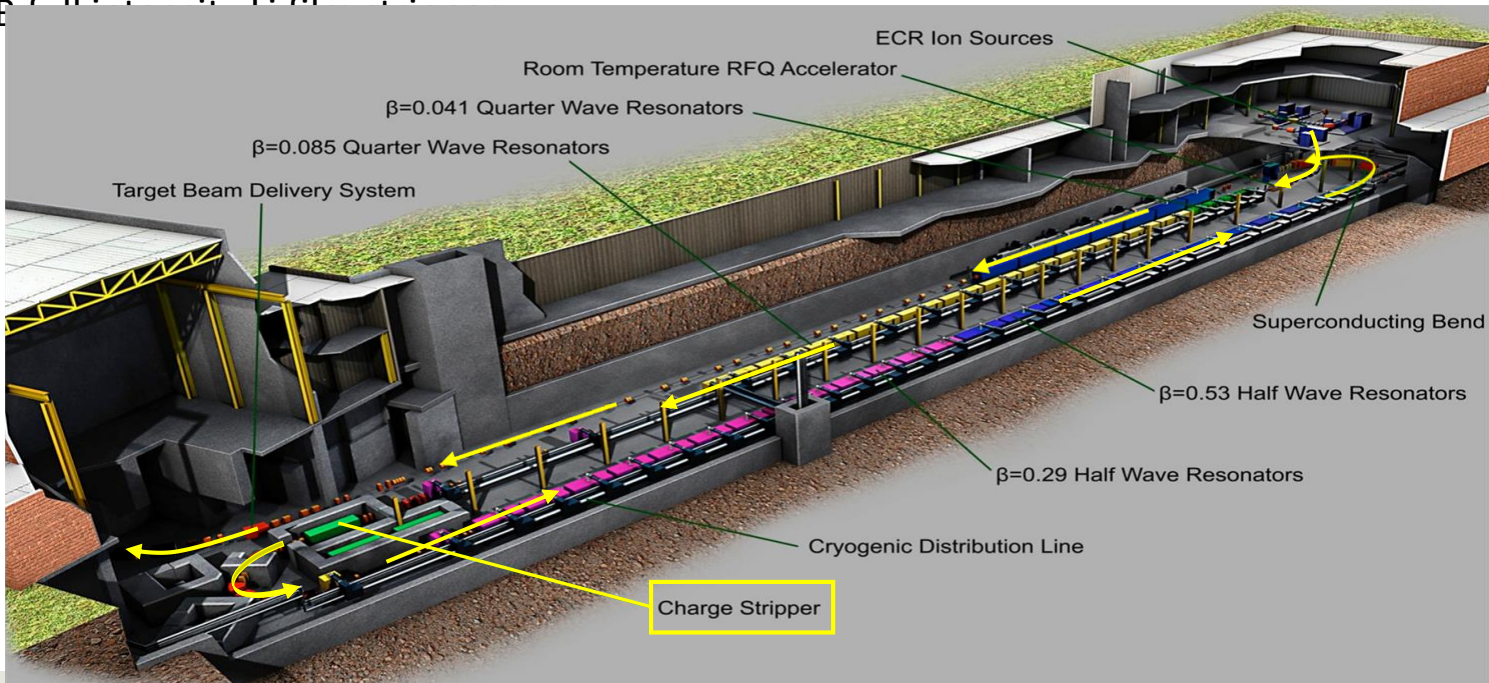
• Annealing at high temperature confirmed



# NSCL-FRIB Strippers

## Challenges

- It is known that thin foils (stripper) used in accelerator suffer a quick degradation due to radiation damage such as swelling and thermo-mechanical changes
  - Limits the lifetime of few hours
- How can we improve the lifetime?
  - Annealing at high temperature
  - Influence of nano-structure on annealing
  - FRIB's High Energy Ion Source
  - ....

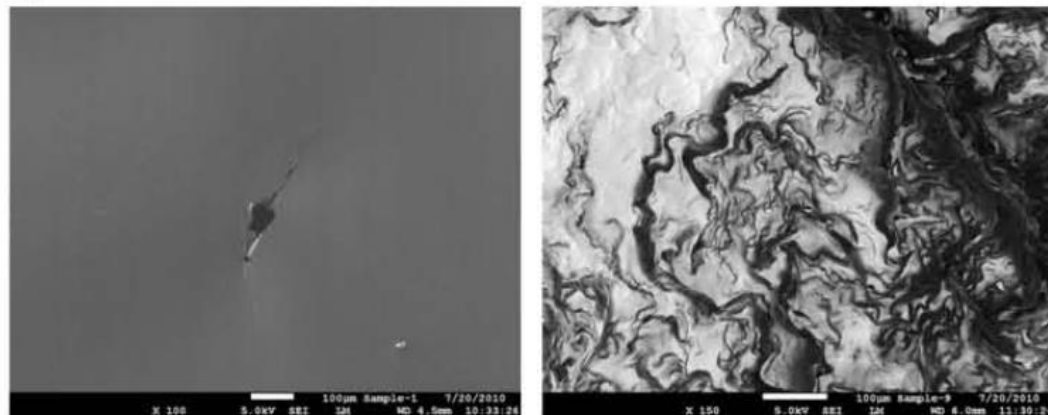


# NSCL-FRIB Strippers

## Radiation Damage

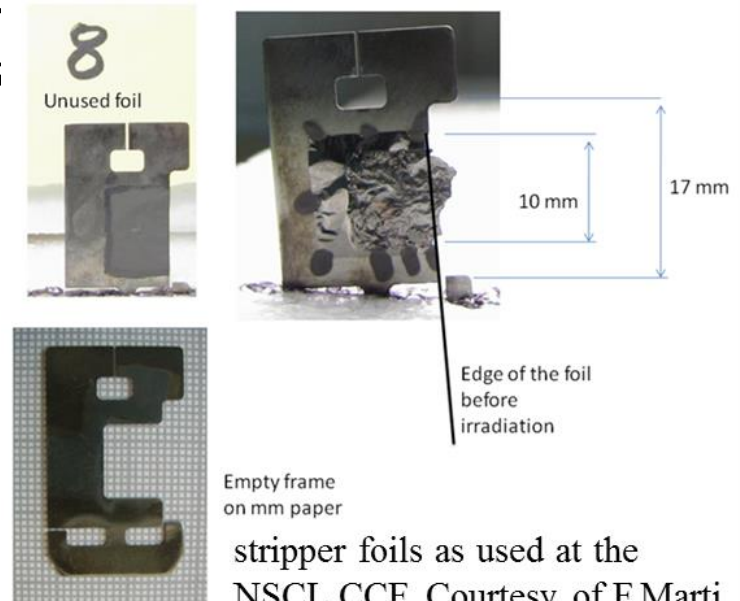
- Recent tests at NSCL have shown quick deterioration of graphite foils under heavy ion bombardment due to thermal and mechanical stresses and radiation damage

- Carbon irradiated with Pb beam @ 8.1 MeV/u



SEM photographs of unused carbon foil (left) showing a small pinhole for illustration and a foil exposed to 8.1 MeV/u Pb beam

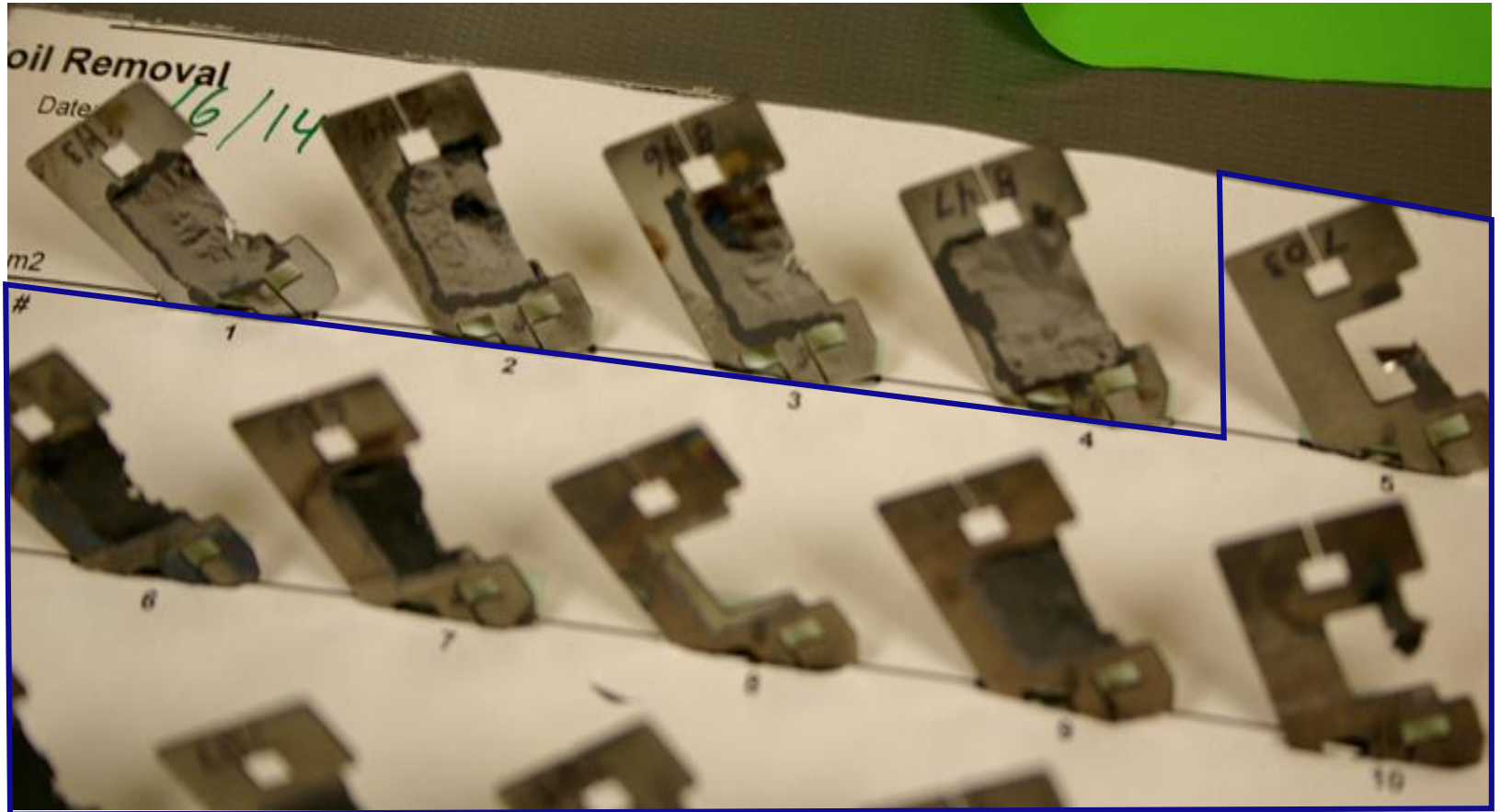
F. Marti et al., "A carbon foil stripper for FRIB", TUP 106, Proceedings of Linear Accelerator Conference LINAC2010, Tsukuba, Japan, TUP105, 2010.



stripper foils as used at the NSCL CCF. Courtesy of F.Marti

# NSCL-FRIB Strippers

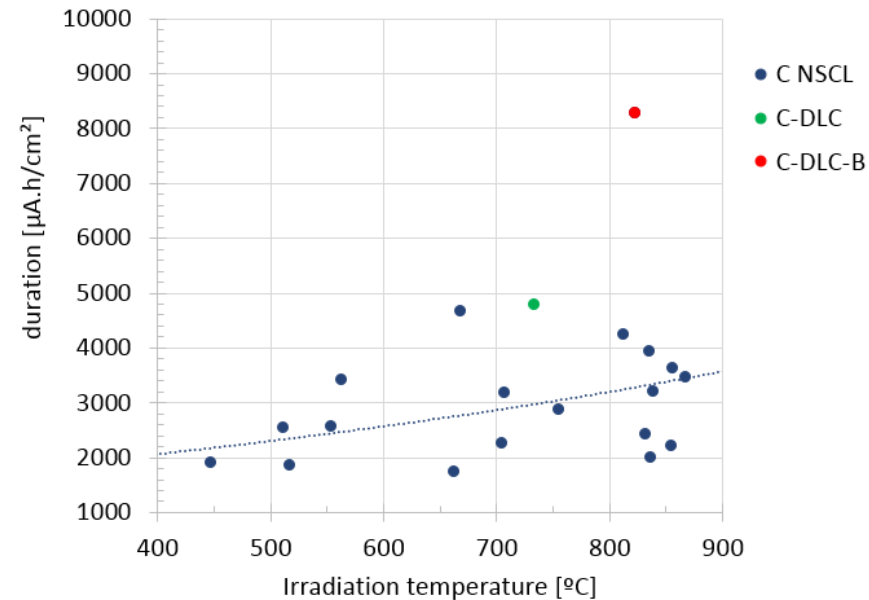
Irradiated Strippers at NSCL



Current carbon strippers used at NSCL

# Improvement of the lifetime

- Previous studies [3] showed annealing effects of radiation damage at high temperature. The heating by the beam was evaluated to produce temperatures of up  $\sim 900$  °C. A clear tendency of increased lifetime with irradiation temperature was observed.
- The lifetime of the 10 multilayer foil C-DLC-B was significantly higher (factor 3) than the standard C-NSCL foils. The 10 multilayer foil C-DLC was somewhat superior (about a factor 2) as compared to the standard foils.



*Lifetime time ( $\mu\text{A}\cdot\text{h}/\text{cm}^2$ ) as a function of the irradiation temperature and the microstructure of graphite stripper foils.*

[3] S. Fernandes et al., "In-Situ Electric Resistance Measurements and Annealing Effects of Graphite Exposed to Swift Heavy Ions", *Nucl. Instrum. Methods Phys. Res. B* 314 (2013) 125-129.

# Summary and Conclusions

- Heavy-ion irradiation tests and annealing studies performed in the context of high-power target and strippers for high intensity accelerator were performed
- High temperature annealing of heavy-ion induced radiation damage observed in production target
  - First experiment of this kind
  - Confirmed by several analysis
- Graphite as a material for FRIB beam production targets promises sufficient lifetime
- High temperature annealing of heavy-ion induced radiation damage observed in NSCL strippers

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# Thank you for your attention

FRIB construction area – October 27 2014



# NSCL-FRIB Strippers

Lifetime measurement at NSCL

- Effect of the temperature on lifetime improvement observed at NSCL

*Preliminary results*

