

Radiation Protection in J-PARC neutrino beam line

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Happy birthday to me!

Summary of Regulations

The design of neutrino beamline must satisfy following radiation level.

- $H < 0.25\mu\text{Sv/h}$ for free access area
- $H < 12.5\mu\text{Sv/h}$ for control area
(Human access limit is $< 1\text{mSv/week}$)
- $H < 5\text{mSv/h}$ (line loss) and $H < 11\text{mSv/h}$ (point loss)
for soil around buildings
- **Radioactive waters** can be disposed to outside (ocean) if the radioactivity is less than 30Bq/cc .
- **Radioactive gas** (air/Helium) can be ventilated to environment if the radioactivity is less than 5mBq/cc .

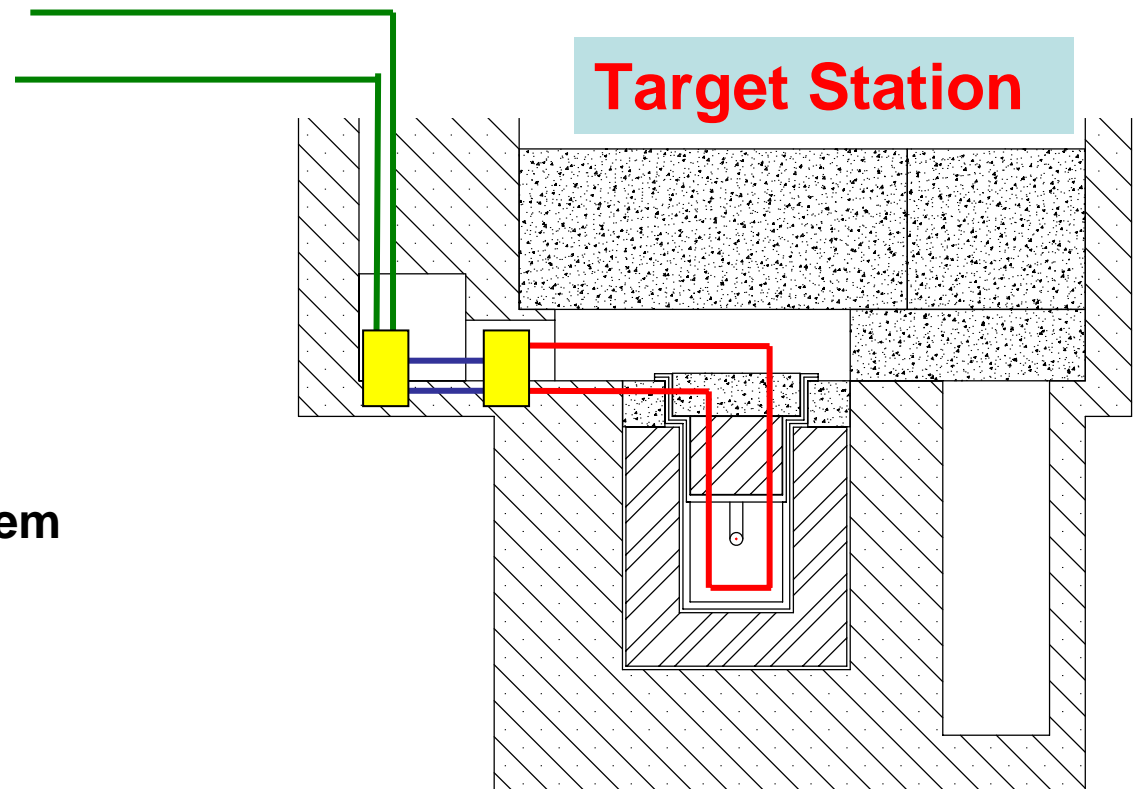
Overview

- Works on radiation protection are essentially adjustments of radiation shielding thickness. The calculations are based on the MARS simulation and other utilities.
- Studies on **Proton Beamline, Target Station, Decay Volume, Beam Dump/Muon Pit** and **access tunnels** were already discussed in the previous NBI workshops.
See <http://www-nu.kek.jp/~oyama/nbi2003.oyama.ppt>
- After a hundred times of iterations between geometry change and radiation calculation, the final design were almost fixed. They were reported in the individual talks.
- In my talk, I would like to concentrate on management of **cooling waters** and **Air/Helium**.

Cooling Water Systems(1)

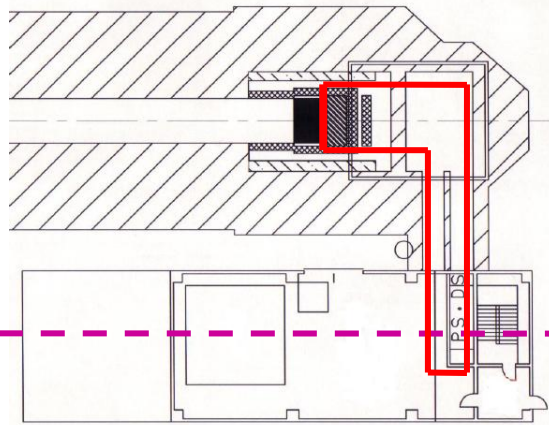
- Primary cooling waters are highly radio-activated when they travel the beam surroundings. They are circulated only in the underground control area during 20 days of beam period.
- They are serious radioactive sources when they pass the heat exchanger.
- They must be disposed to outside (ocean) after the beam stop.

- Primary cooling water system
- Secondary cooling water system
- Third cooling water system
- Heat exchanger



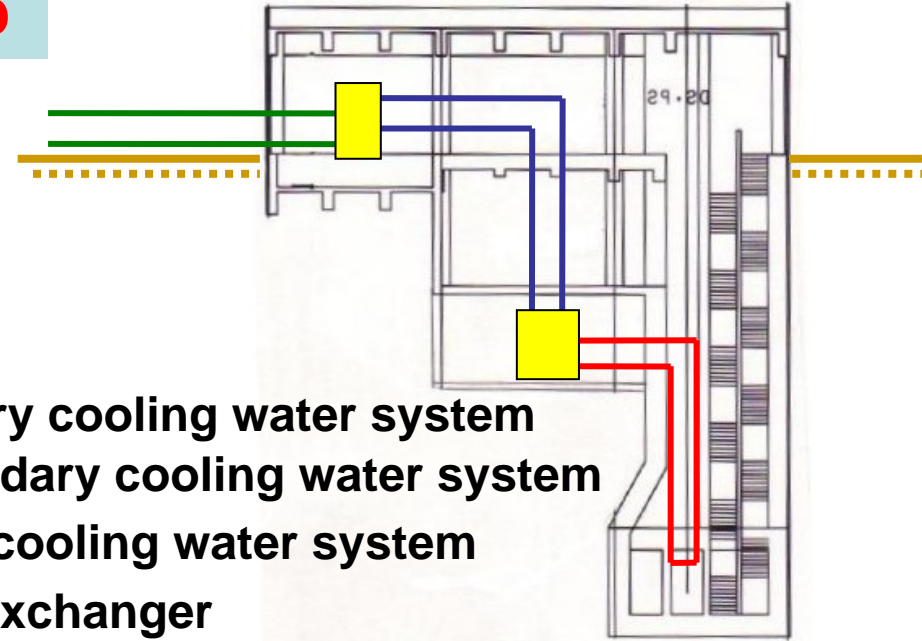
Cooling Water Systems(2)

Top view



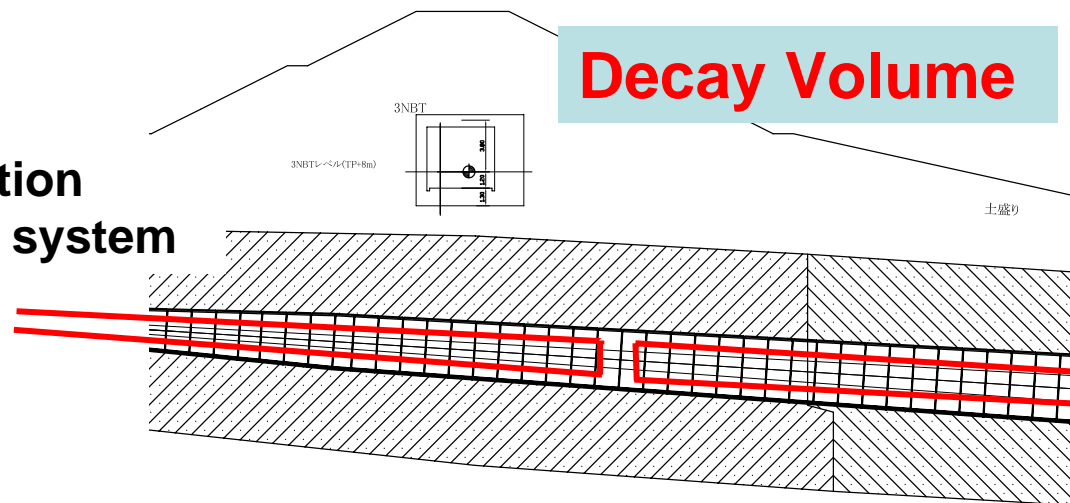
Beam dump

Side view



- Primary cooling water system
- Secondary cooling water system
- Third cooling water system
- Heat exchanger

To Target Station cooling water system



Decay Volume

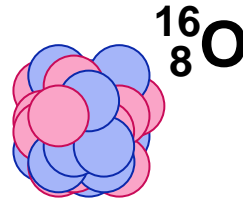
To Beam Dump cooling water system

Requirements for cooling water

- (A) When primary cooling waters pass the **heat exchanger** during beam period, radiation level from radio-activated waters should not exceed the requirements from the regulation. They are $H < 12.5\mu\text{Sv/h}$ for control area, and $H < 0.25\mu\text{Sv/h}$ for free access area
- (B) After one cycle (**20 days**) of **0.75MW** beam operation, radio-activated primary cooling waters should be **disposed** to ocean. For this purpose, concentration of radioactivity in the water must be less than **30Bq/cc**.

Radio-activation of cooling water

- Radioisotopes are produced from Oxygen in H₂O.
Neutrons from the beam with energy larger than **~20MeV** break the ¹⁶₈O nuclei.



- Several kinds of isotopes with **Z O 8** and **N O 8** are made as spallation products.
They are **³He, ⁷Be, ¹¹C, ¹³N, ¹⁵O, ¹⁴O, ¹⁶N, ¹⁴C**.
Nominal cross sections are **~30mb**.
- Production rate of the radioisotope is obtained from (neutron flux) x (cross section) x (volume of water).

Isotopes generated in cooling water

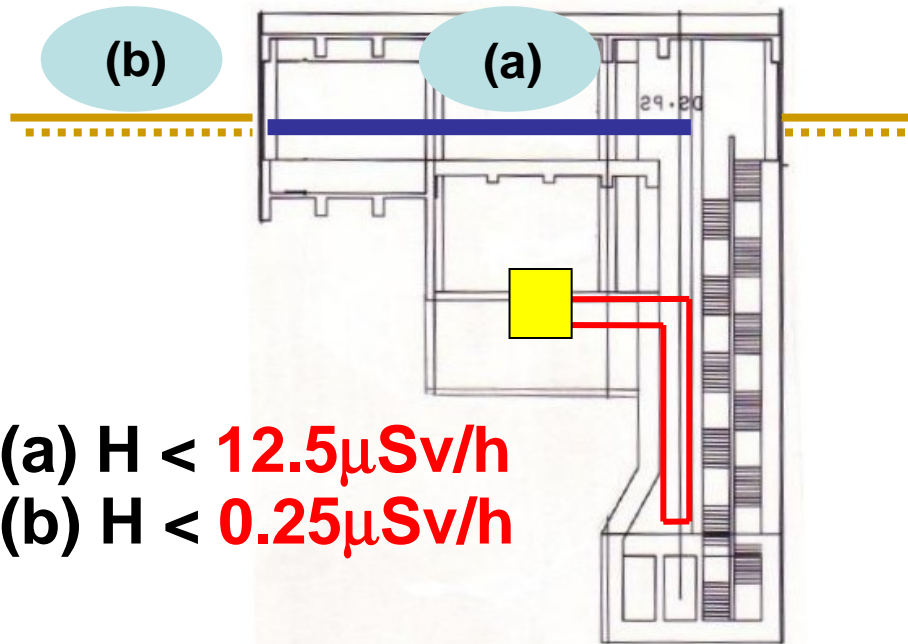
	E(MeV)	lifetime	comments
^3H	0.0186	12.3y	main obstacle in disposal
^7Be	0.478 (10%)	53.3d	removed by ion filters
^{11}C	0.511x2	20m	} Radiation sources in heat exchanger Negligible after a few hours of cooling
^{13}N	0.511x2	10m	
^{15}O	0.511x2	2m	
^{14}O	2.3	70s	
^{16}N	6	7s	
^{14}C	0.156	5730y	decay rate is negligible

- (A) During the beam operation, ^{11}C , ^{13}N , ^{15}O , ^{14}O , ^{16}N in the heat exchanger are the radiation sources.
- (B) In the disposal scenario, ^3H is the dominant radioactive source.

Radioactive water in heat exchanger

- All short-life radioisotopes are in (production rate) = (decay rate) equilibrium.

Example : Beam dump and Decay volume



(a) $H < 12.5 \mu\text{Sv/h}$
 (b) $H < 0.25 \mu\text{Sv/h}$

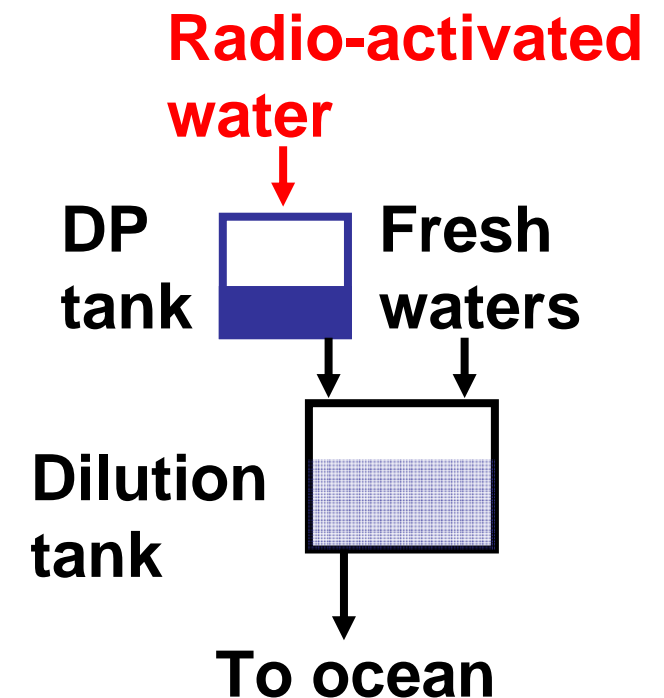
	(GBq)	E(MeV)	lifetime	H($\mu\text{Sv/h}$)
^{11}C	400	0.511×2	20m	16 / 0.4
^{13}N	400	0.511×2	10m	15 / 0.4
^{15}O	400	0.511×2	2m	13 / 0.3
^{14}O	400	2.3	70s	110 / 12
^{16}N	400	6	7s	27 / 6

(a) / (b), floor thickness = 20cm, 
 time from beam exposure = 30sec

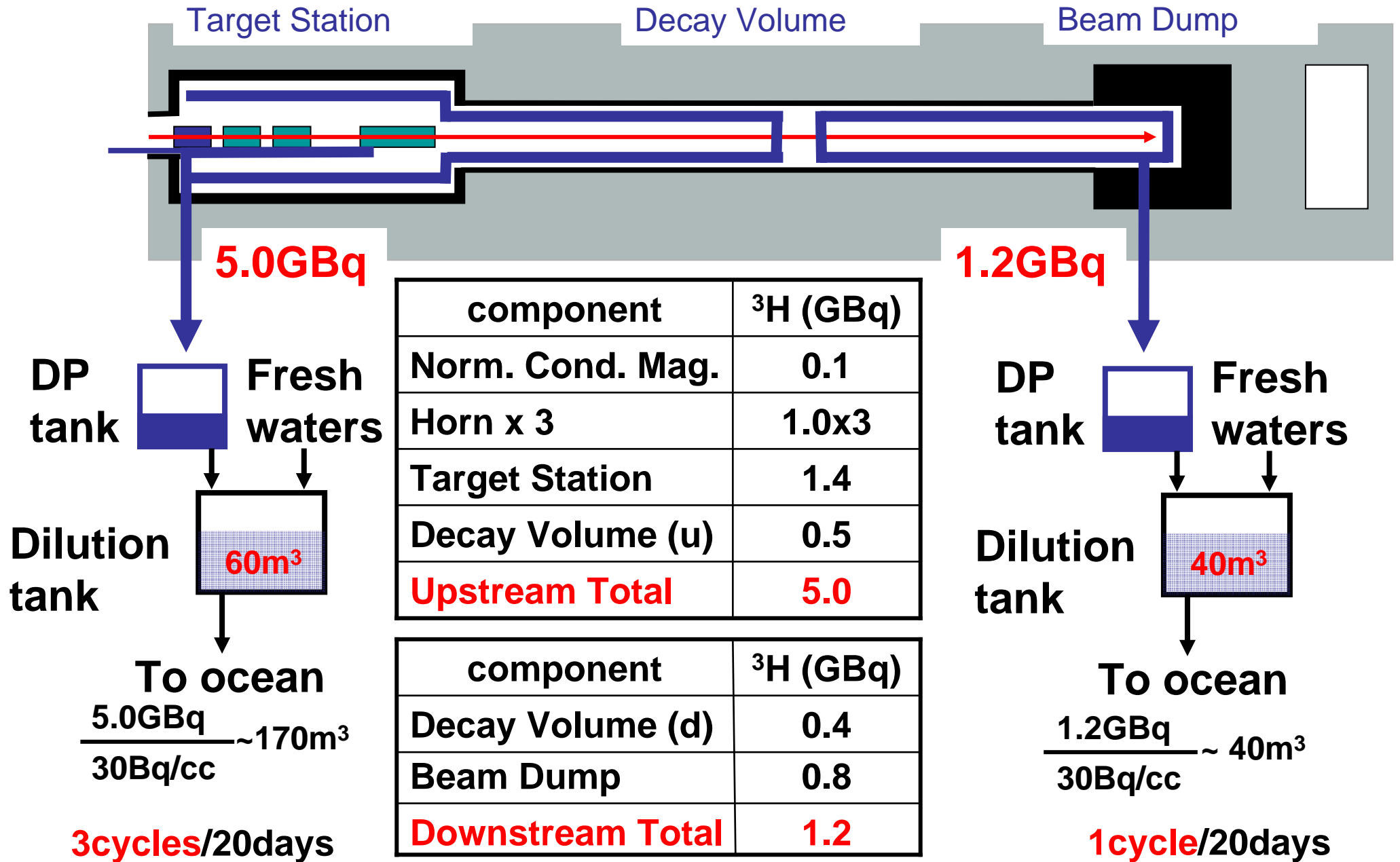
- Thick floor concrete is effective for ^{11}C , ^{13}N , ^{15}O
 ^{11}C : $16 \mu\text{Sv/h}$ (20cm) \rightarrow $0.02 \mu\text{Sv/h}$ (60cm)
- Long travel time of water is effective for ^{16}N
 ^{16}N : $27 \mu\text{Sv/h}$ (30sec) \rightarrow $0.03 \mu\text{Sv/h}$ (100sec)
- From simulations of all radioisotopes, we conclude that concrete thickness **70cm** and travel time **100sec** are needed.

Disposal procedure of radioactive water

- After **20 days** of beam operation, we wait several hours. All radioisotopes with short lifetime decay.
- Primary cooling waters are moved to DP tank. The water volume is small ($< 10\text{m}^3$), but it is highly radio-activated ($100\sim 500\text{Bq/cc}$).
 - (1) Appropriate volume of radio-active waters are moved to the dilution tank, and mixed with fresh waters.
 - (2) The radioactivity in of the diluted waters is measured.
 - (3) The waters are disposed to the outside (ocean) if the radiation is less than **30Bq/cc** .
- Repeat process (1)~(3) until the DP tank become empty. It takes **2 days** for one cycle.



Summary of cooling water disposal



component	³ H (GBq)
Norm. Cond. Mag.	0.1
Horn x 3	1.0x3
Target Station	1.4
Decay Volume (u)	0.5
Upstream Total	5.0

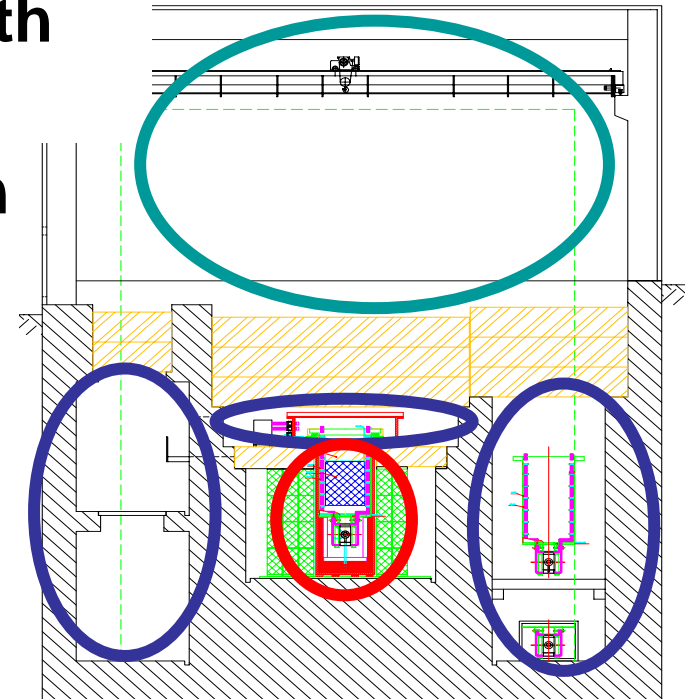
component	³ H (GBq)
Decay Volume (d)	0.4
Beam Dump	0.8
Downstream Total	1.2

Radio-activation of Air/Helium

- Radioactive gas (Air/Helium) can be ventilated to environment if radioactivity is less than **5mBq/cc**. If the radiation level exceeds this requirement, it must be ventilated **by mixing with fresh air**.
- The dominant radioactive source in air is **^3H** . They are spallation products from **$^{16}\text{O}/^{14}\text{N}$** and **neutrons** with energy larger than 20MeV. The process is almost the same as the case of the cooling waters.
- Production rate of **^3H** from **Helium** is smaller than that from air: **$\sigma_{\text{He}} \sim \sigma_{\text{Air}}/25$** . It is because He nuclei is hard to break.

Ventilation procedure (Target Station)

- We are planning to make a ventilation system of **13000m³/h** capability in the target station.
- Air in the surface building is always ventilated even in the beam period.
- After **20** days **0.75MW** beam operation, radio-activation of air in service pit, machine room, radioactive storage are less than **10⁻²mBq/cc**. They can be ventilated without mixing with fresh air. It takes less than **1 hour**.
- Radio-activation of cooling air for the iron shielding, Helium in target station and decay volume are more than **1Bq/cc**. They must be ventilated by mixing with fresh air. It takes about **2 days** after **20days** beam. Even after **one year** of operation, it takes less than **one month**.



Summary of Air/Helium ventilation in Target Station

0.75MW , 20days operation

$$\sigma_{\text{air}}=30\text{mb}, \sigma_{\text{He}}\sim\sigma_{\text{air}}/25$$

component	volume (m ³)	Neutron fluence (/p/cm ²)	Radio-activation (Bq/cc)	Total tritium (Bq)	Ventilation time(h)
Surface building	13000	1×10 ⁻¹⁹	4×10 ⁻¹⁴	-----	realtime
Service pit	230	5×10 ⁻¹²	3×10 ⁻⁶	7×10 ²	0.02(direct)
U.g. machine room	330	5×10 ⁻¹²	3×10 ⁻⁶	1×10 ³	0.03(direct)
radioactive storage	780	5×10 ⁻¹²	3×10 ⁻⁶	2×10 ³	0.06(direct)
Iron cooling	30	10 ⁻¹⁰ ~ 5×10 ⁻⁶	5×10 ⁻⁵ ~2.6	1.6×10 ⁷	0.25(mixed)
TS Helium	135	2×10 ⁻⁴	3.2	6.3×10 ⁸	10(mixed)
DV Helium	1600	5×10 ⁻⁵	0.8	1.9×10 ⁹	30(mixed)

Summary

- **Calculations of radiation shielding were almost done.**
- **Management of cooling waters and Air/Helium are established for beam operation period as well as for their disposal/ventilation.**

Lunch time!

Supplements

Radiations in the Proton Beamline

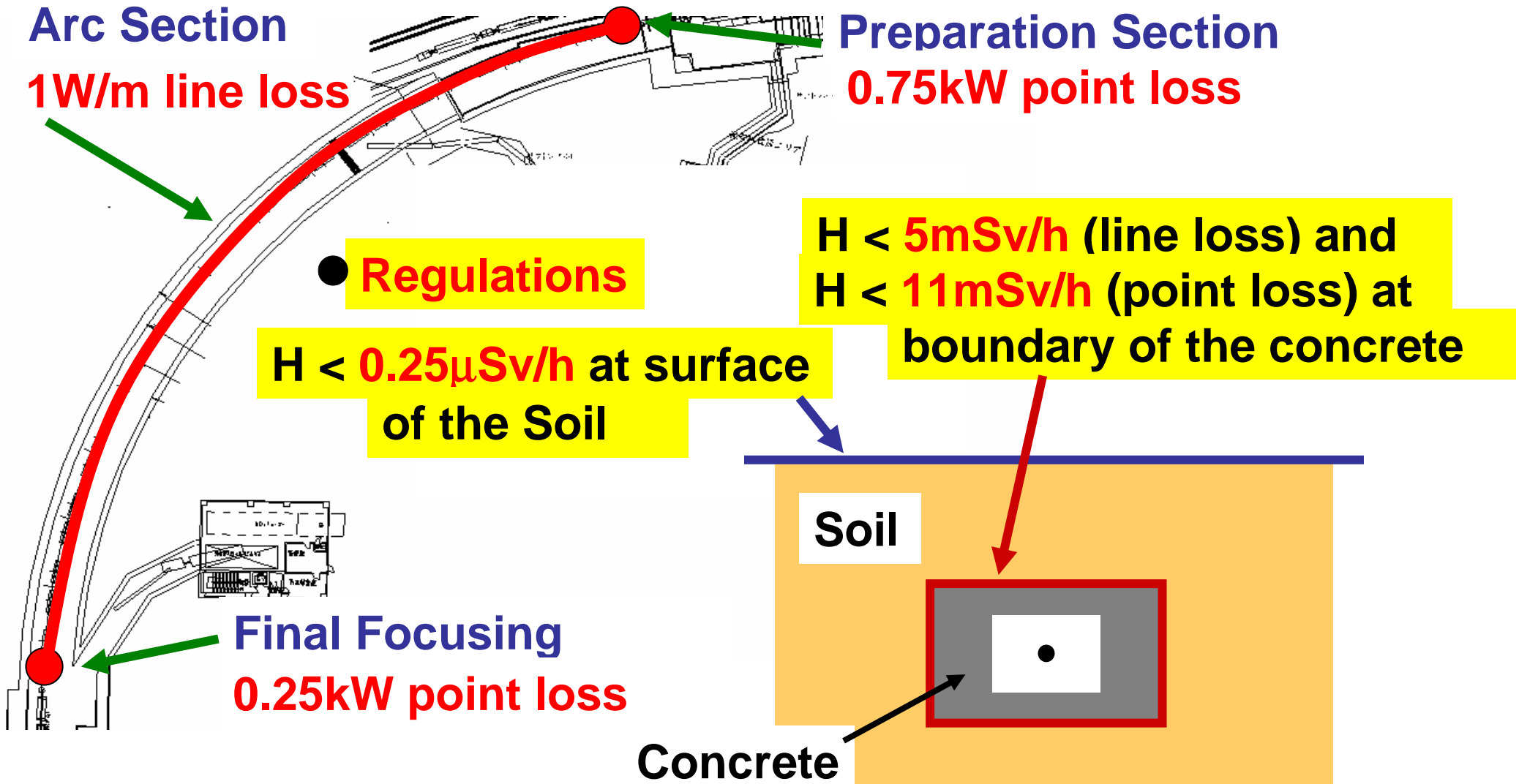
- Following energy loss are assumed from our experience.

Arc Section

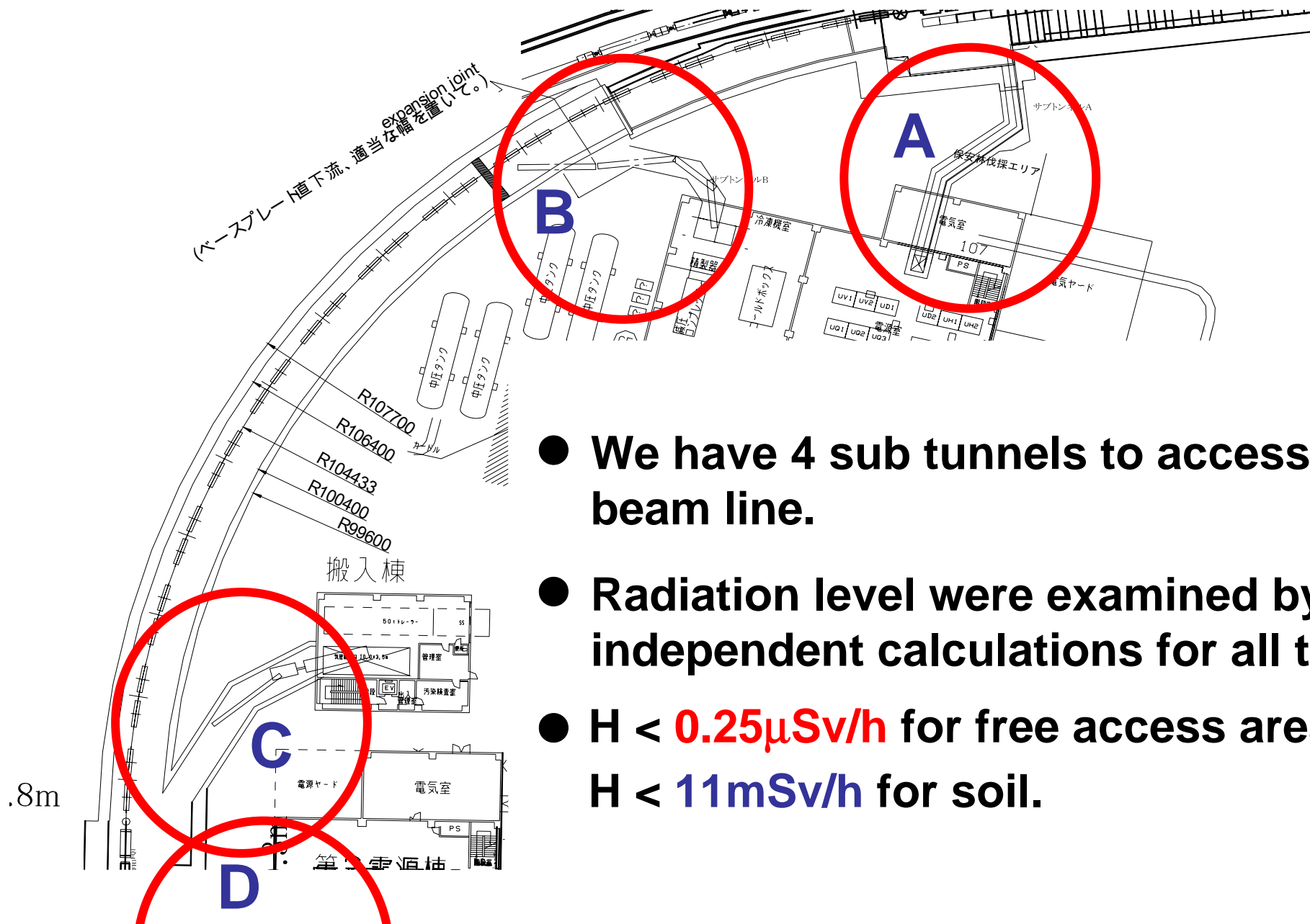
1W/m line loss

Preparation Section

0.75kW point loss

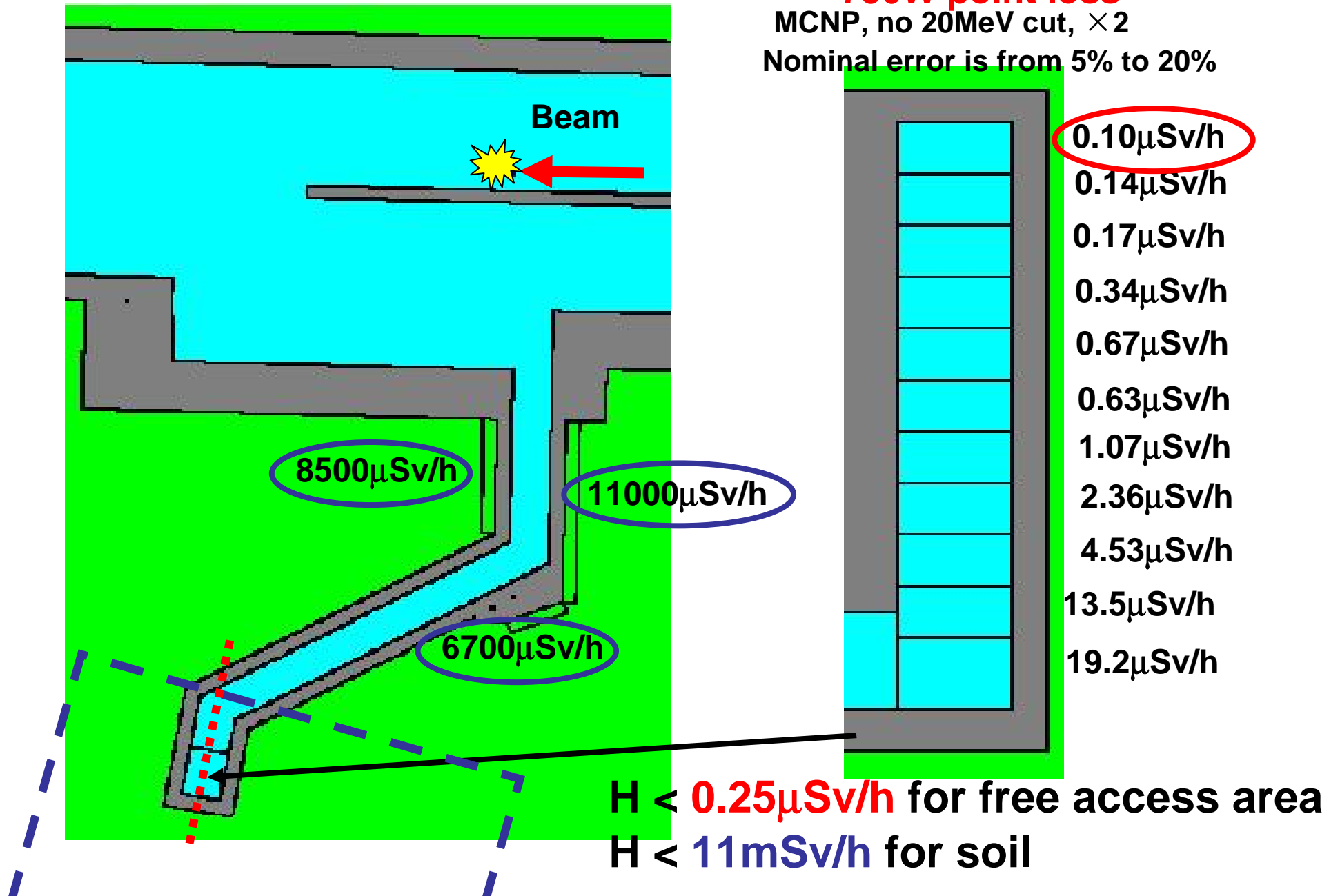


Sub Tunnels



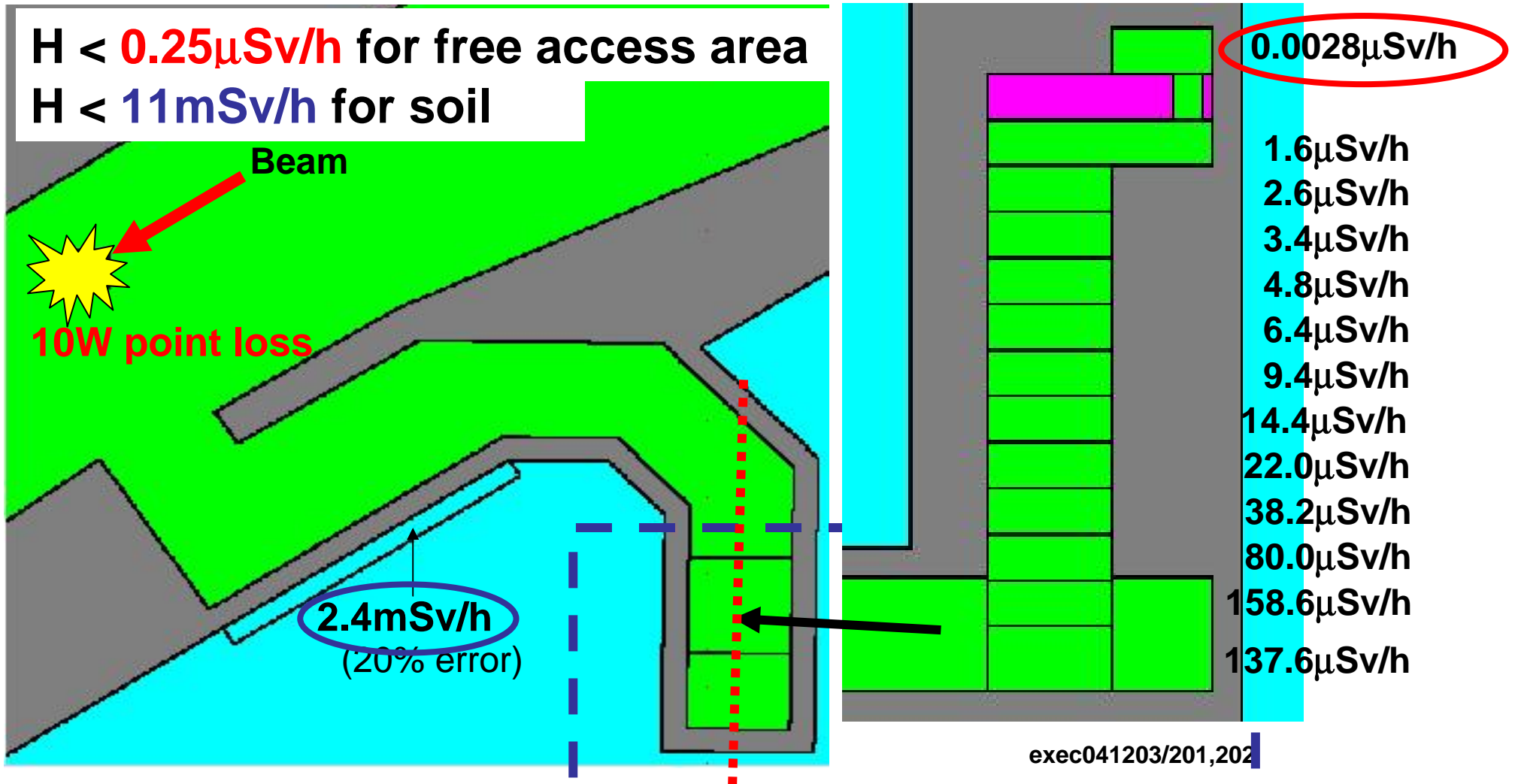
- We have 4 sub tunnels to access the beam line.
- Radiation level were examined by 2 independent calculations for all tunnels.
- $H < 0.25\mu\text{Sv/h}$ for free access area and $H < 11\text{mSv/h}$ for soil.

Sub Tunnel A



Sub Tunnel B

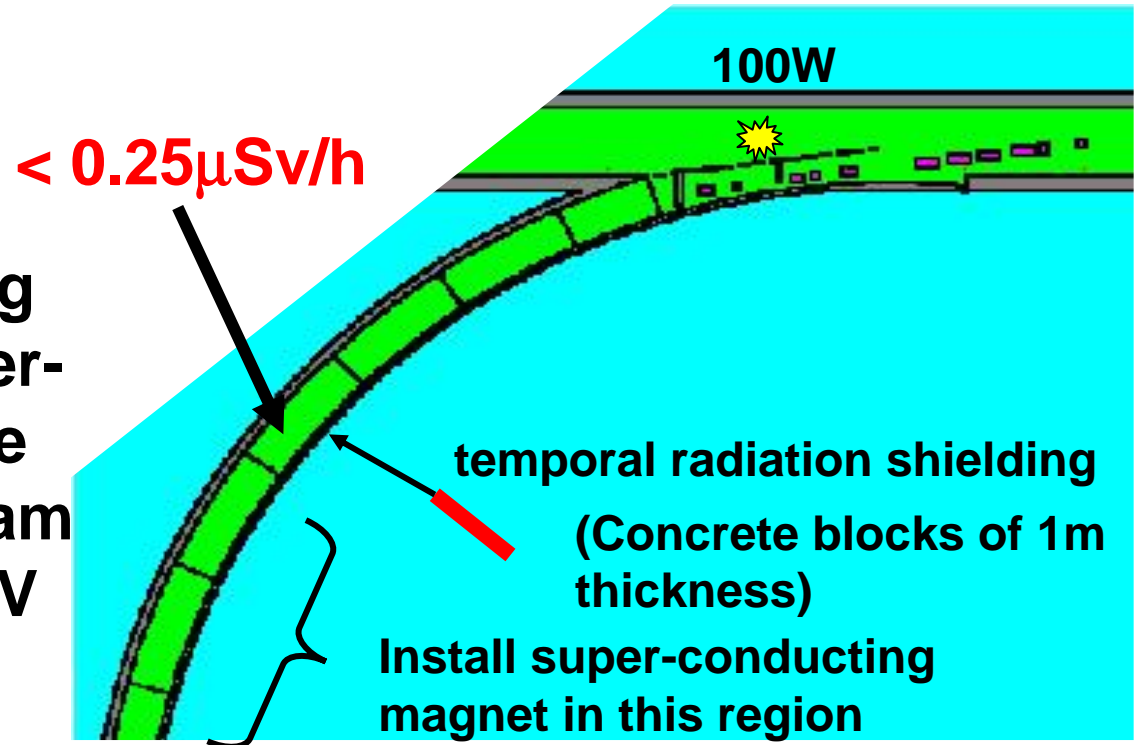
MCNP, no 20MeV cut, $\times 2$



- Complicated simulations were executed for all facilities very carefully.

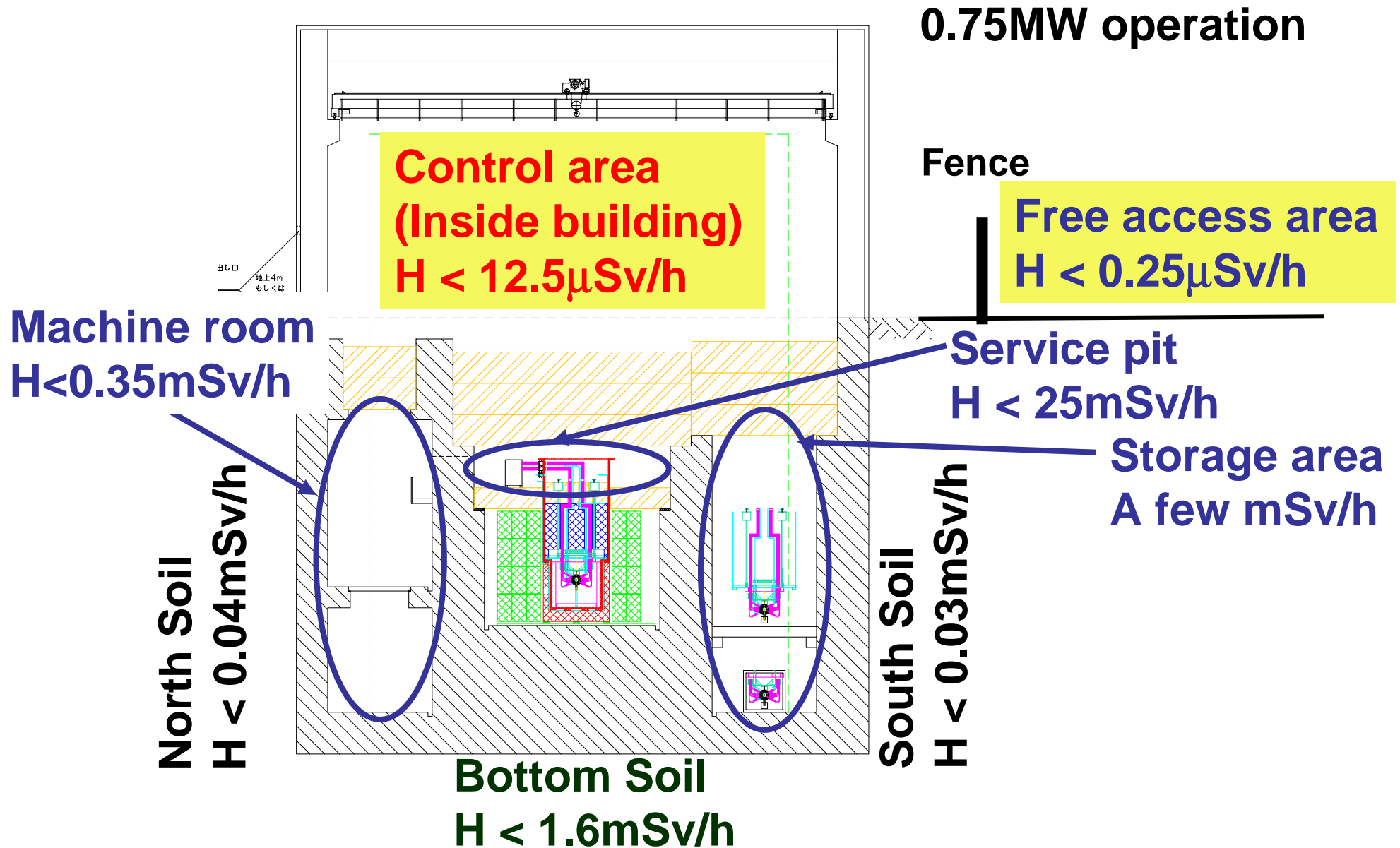
Radiation in the Proton beam line during the commission of 50GeV main ring

- From end of 2007 to spring 2008, we must install super-conducting magnets in the downstream of proton beam line. Commission of 50GeV ring has been started.



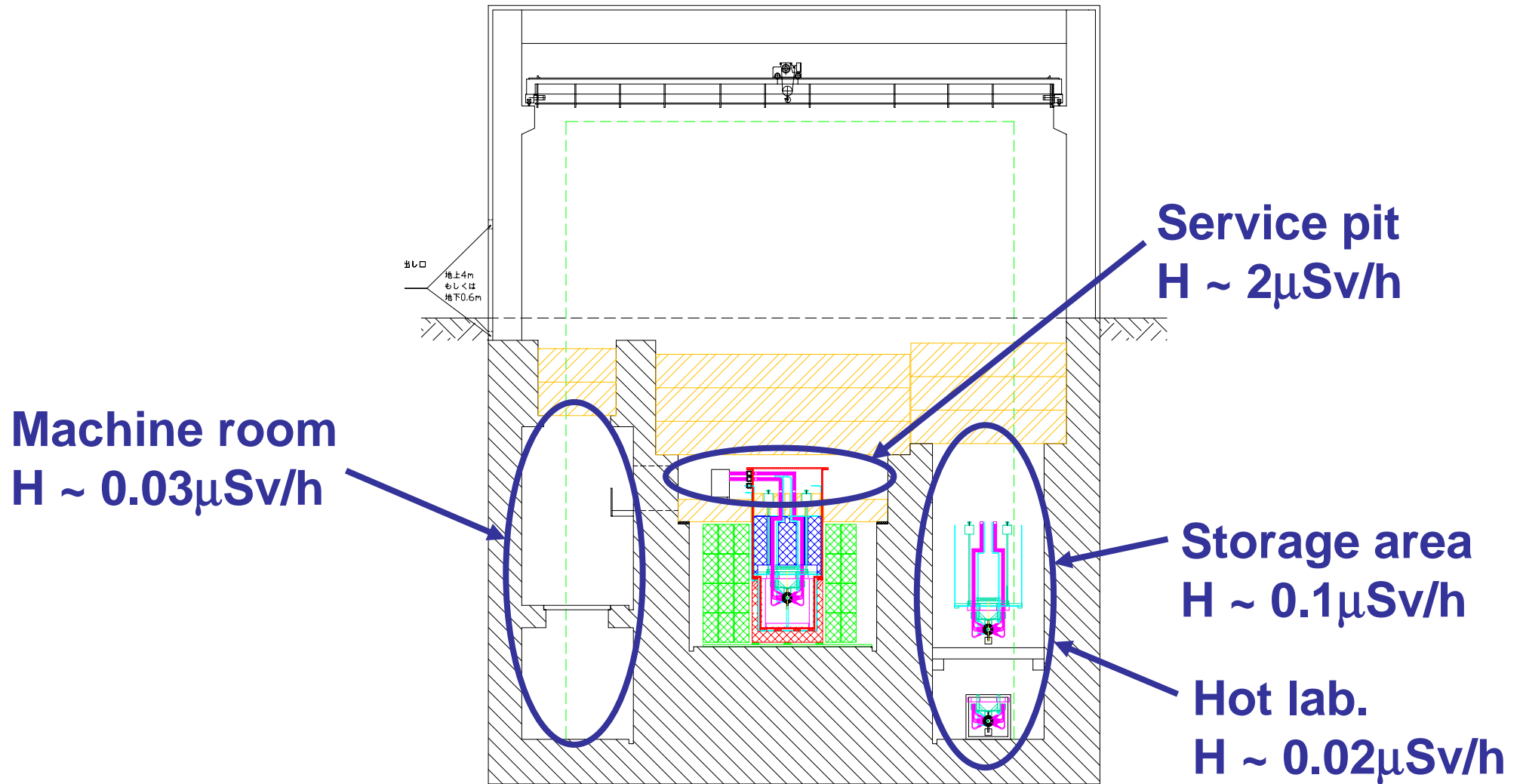
- If we install temporal radiation shielding and possible maximum energy loss is adjusted to be 100W, radiation level in the working area is less than $0.25\mu\text{Sv/h}$.
- Moving magnet installation schedule to 2008 summer shutdown is also under consideration.

Radiation dose during beam operation



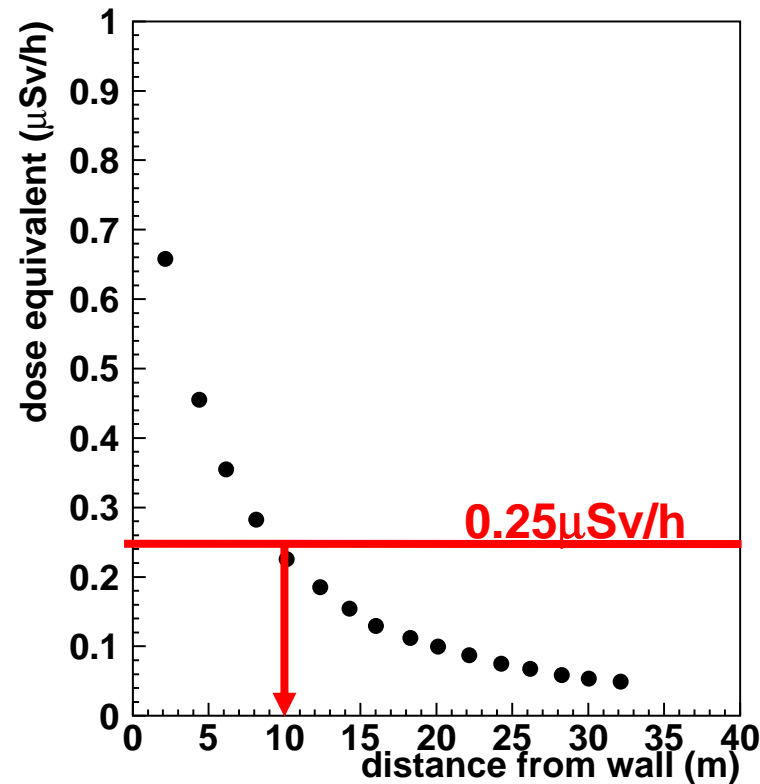
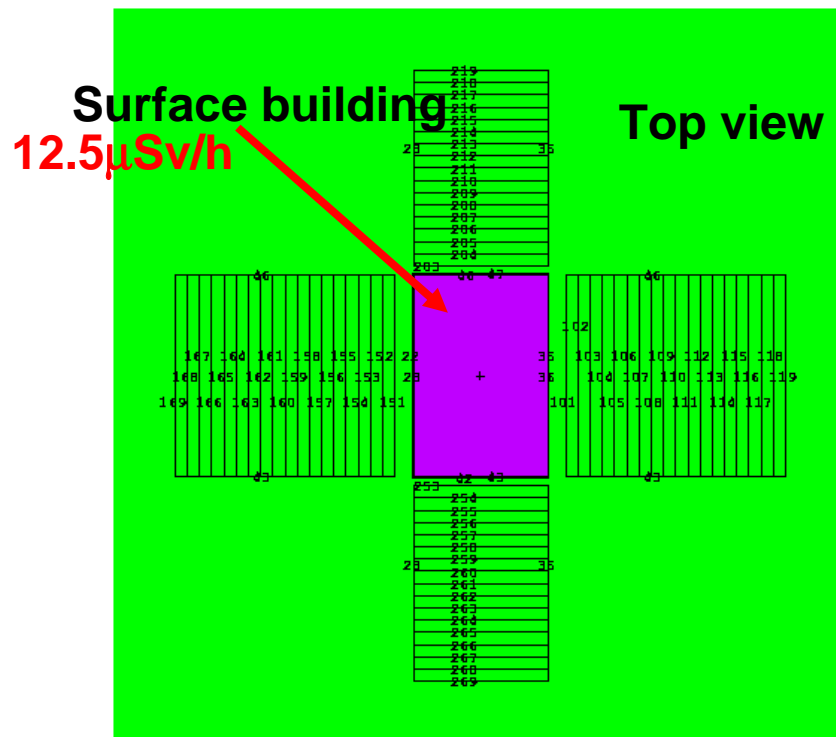
Residual dose after beam stop

30 days operation and one day cooling



Determination of the control area boundary by MCNP

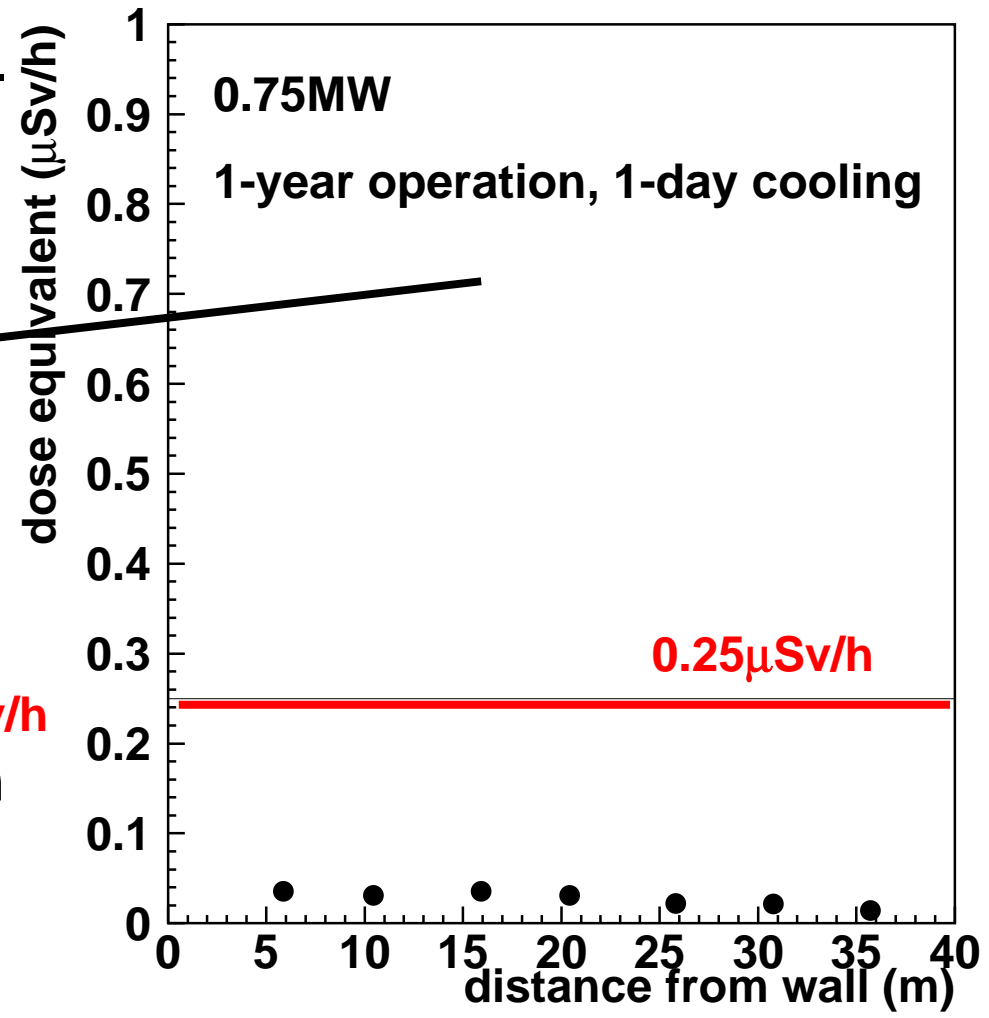
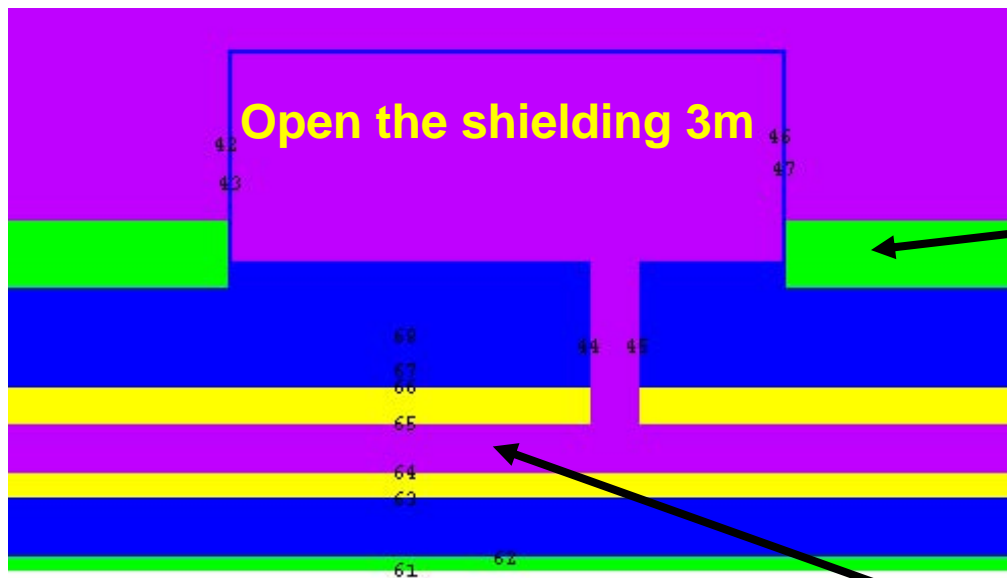
- Neutron sources are defined on the floor, and the dose above the floor is adjusted to be **12.5 μ Sv/h**.



- We need **10m** between the surface building and the fence

Requirement for the boundary during the maintenance

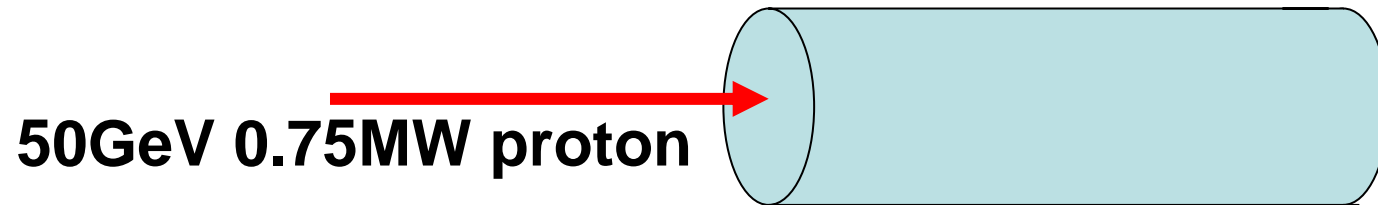
- MCNP is used. γ -ray source are defined on the Al tunnel surface.



- Radiation from residual dose in the tunnel is satisfactory small.

Residual dose of the Target/Horn

- Residual dose of 3cm Φ x 90cm Carbon Graphite target (in a Al container) and 1st magnetic horn is calculated.



	Target	Target	Horn
	(1)(Sv/h)	(2)(Sv/h)	(1)(Sv/h)
1 day	16.9	18	18
1 month	11.6	12.3	3.9
1 year	0.148	0.16	2.8
5 year	8.3×10^{-10}	8.9×10^{-10}	-----
10 year	-----	-----	0.25
20 year	-----	-----	1.7×10^{-5}

After 1 year operation

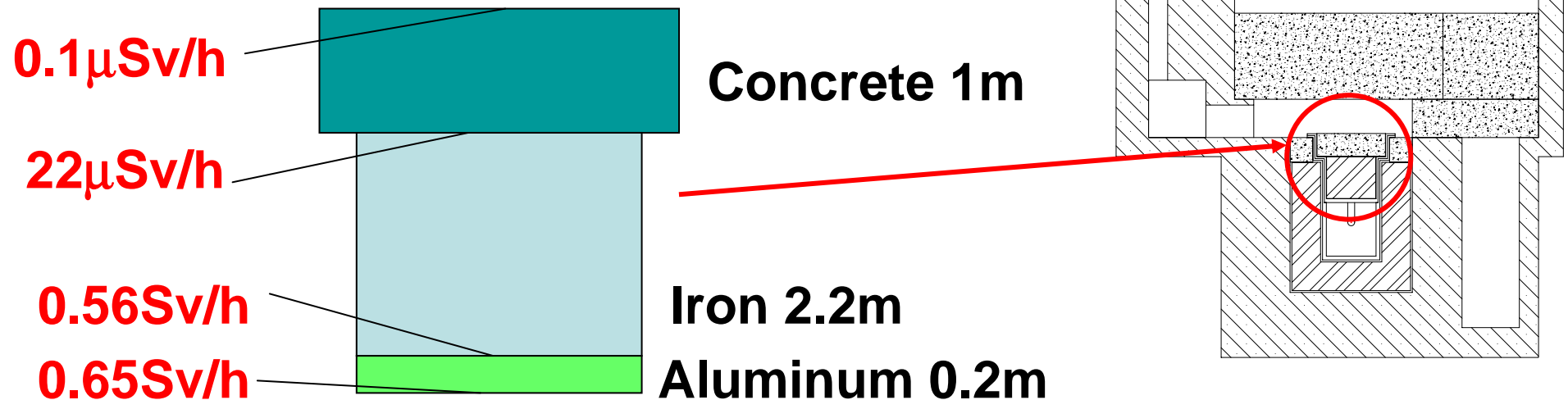
(1)NMTCLIB/JAM(nmtclib95)
+ DCHAIN-SP
+ QAD-CGGP2

(2)Hadron fluence(MARS)
+ cross section(9mb)
+ ⁷Be life

- Horn must be kept in the storage for more than **10** years.

Residual dose of the Shielding

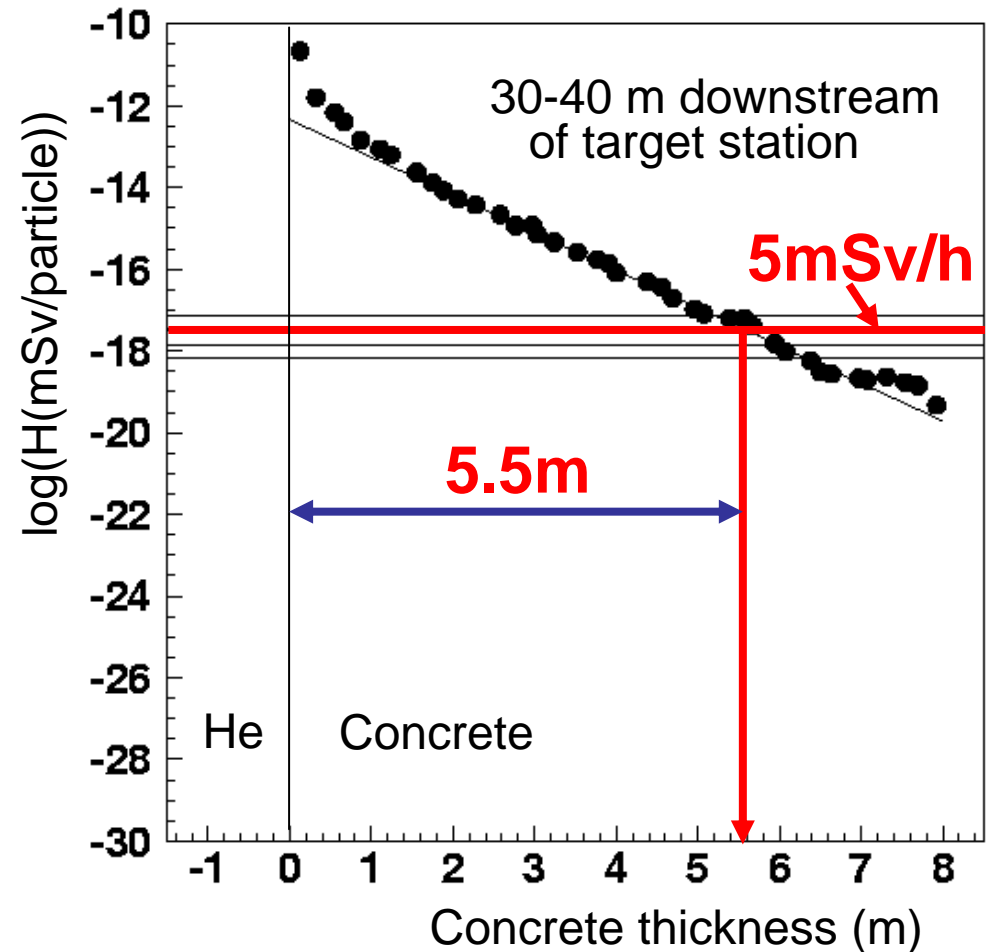
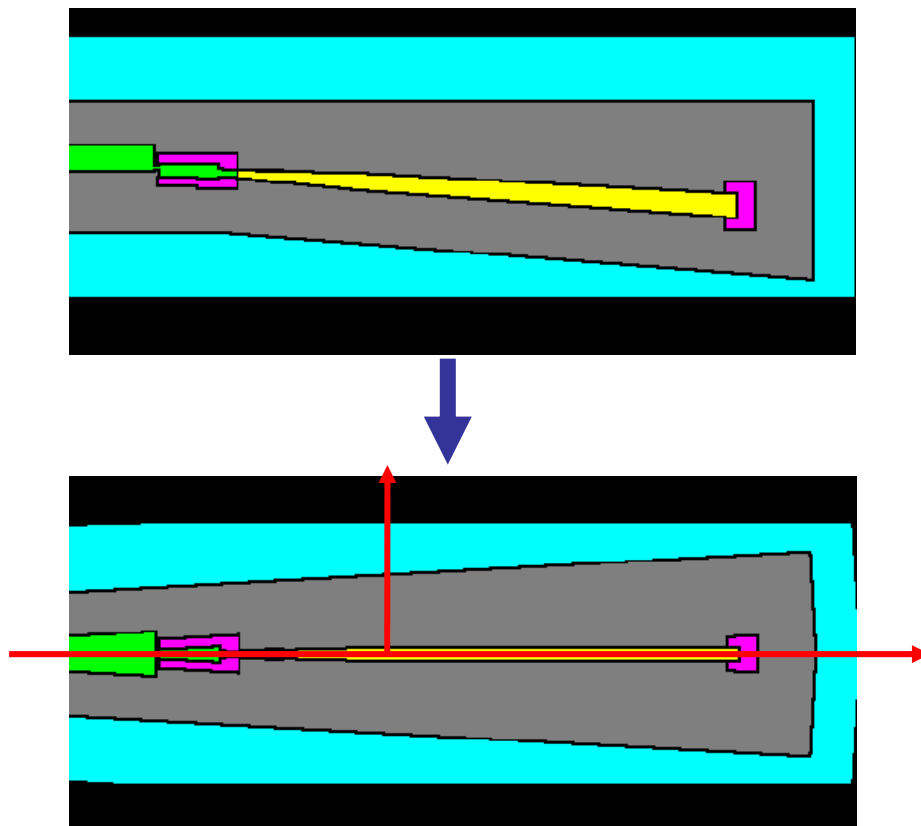
- Residual dose of the shielding calculated by MARS (1 year operation, 1 day cooling, 0.75MW)



- Use of Al surface reduce the radiation about one order of magnitude.
- Further calculation is needed after the “scenario” is fixed.

Calculation of **Decay Volume** Shielding

- As the target station, virtual cylindrical geometry is used in the MARS calculation.



- **5.0~5.9m** of concrete and additional **~6m** of soil are needed to satisfy concrete and soil surface condition

End