



MAX-PLANCK-GESELLSCHAFT

The long way to steady-state fusion plasma - the superconducting stellarator device Wendelstein 7-X

Thomas Klinger

Max-Planck-Institut für Plasmaphysik
Ernst-Moritz-Arndt University

Greifswald





MAX-PLANCK-GESELLSCHAFT

Outline of the talk

IPP

An Institute of the
Max-Planck Society



National Funding via the
Helmholtz Association



Co-Funded by the
European Commission



- I. Fusion basics
- II. The device
- III. Construction
- IV. Research

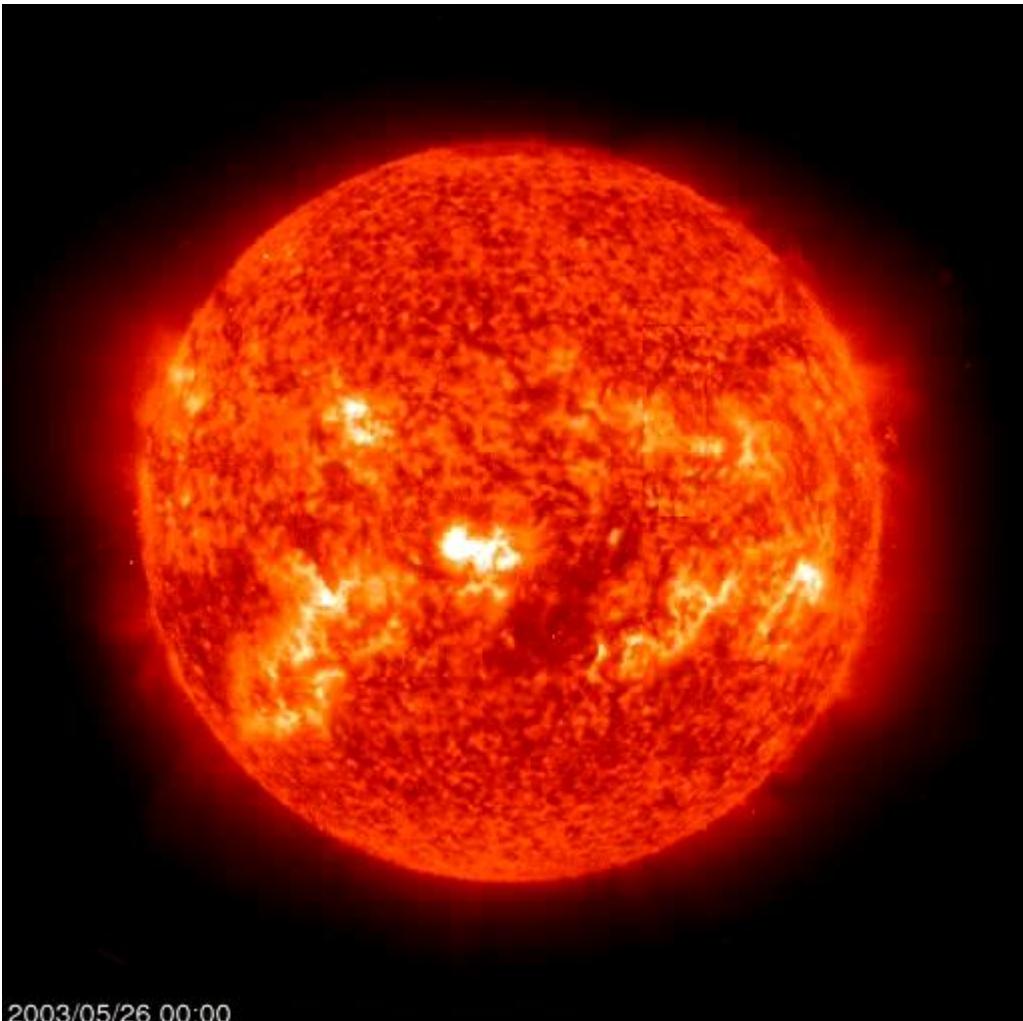


Colloquium CERN



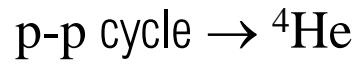
The p-p-cycle in the sun

Credits to ESA, NASA, SOHO – EIT Consortium



power generation in the sun

fusion of light nuclei



difference in binding energy

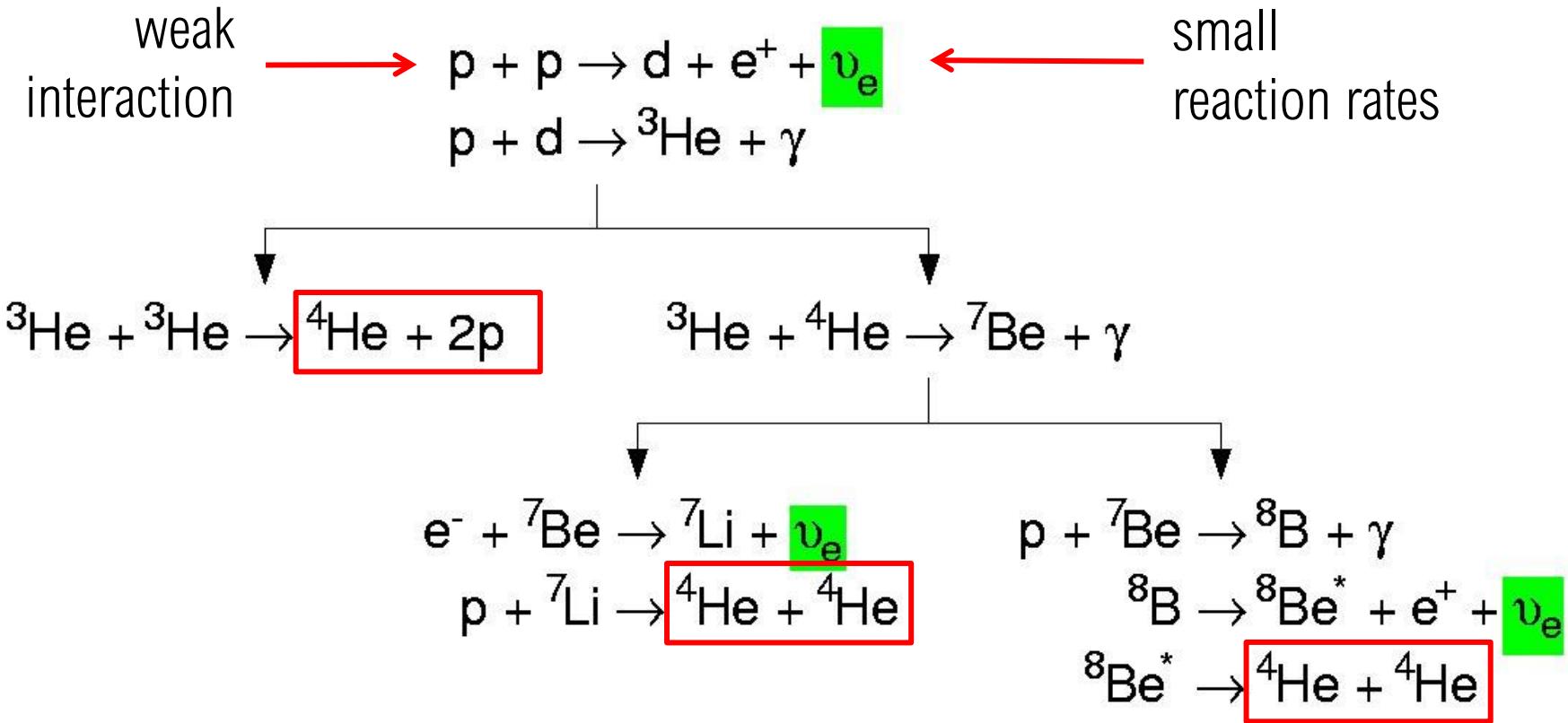


energy surplus

- plasma state (H, He, Fe)
- core temperature ~ 1.3 keV
- extremely small reaction rates
- gravitational confinement m_\odot



The p-p reactions in the sun



branch I
(85 %)

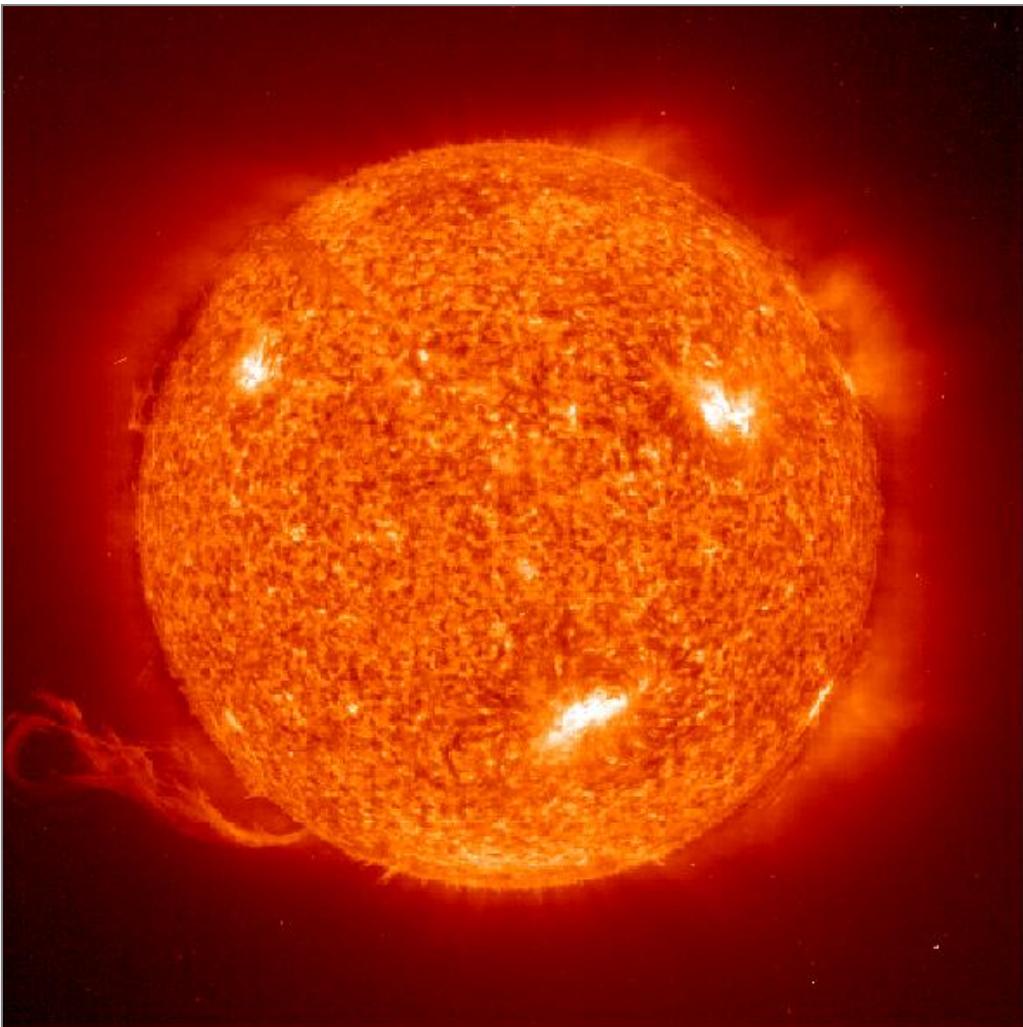
branch II
(15 %)

branch III
(0.02 %)

- neutrinos observed on earth

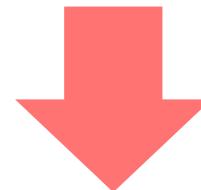
p-p fusion on earth?

Credits to ESA, NASA, SOHO – EIT Consortium



conditions in the sun core

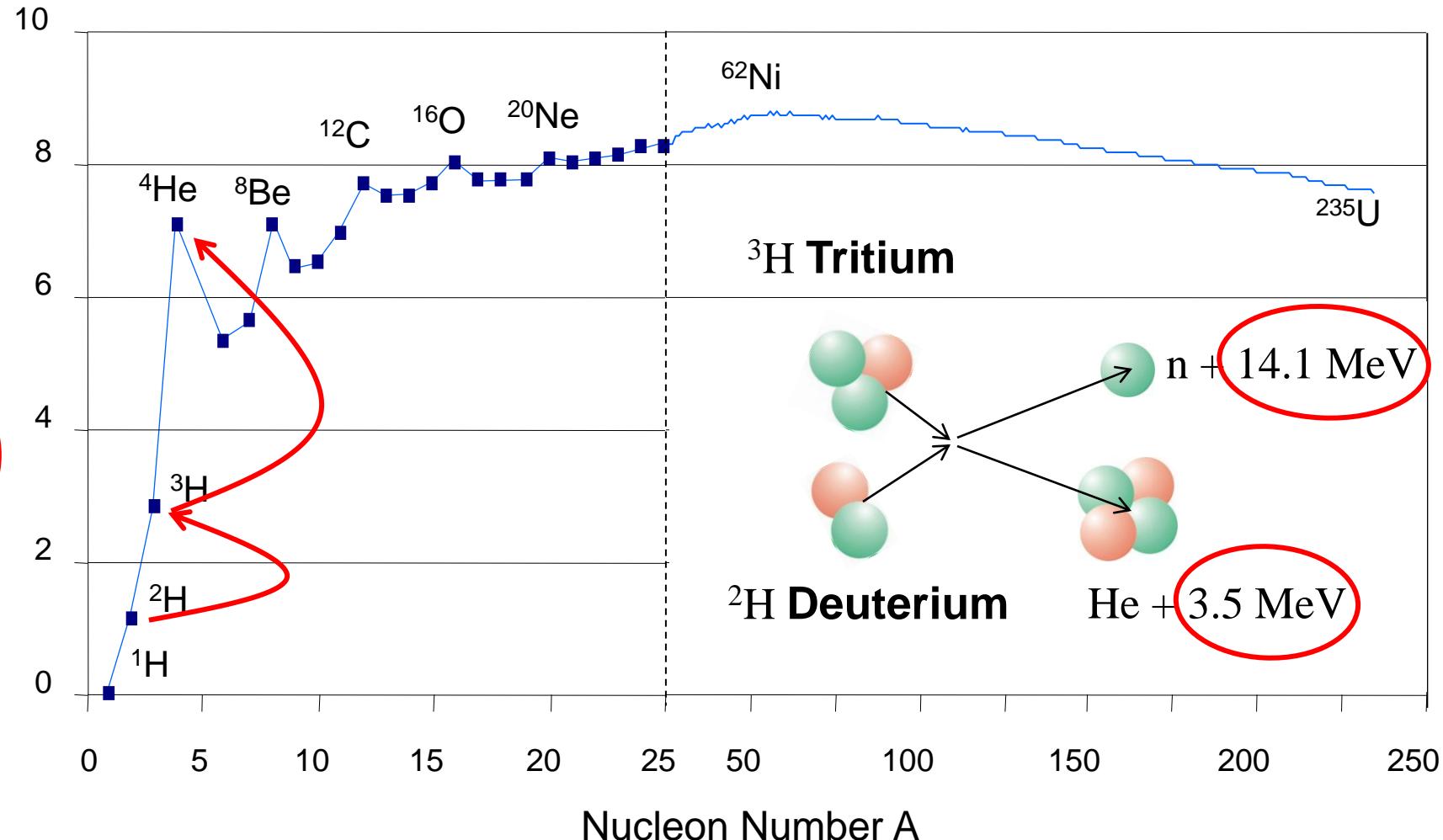
- core plasma density $\sim 10^{31} \text{ m}^{-3}$
- core ion temperature $\sim 1.5 \text{ keV}$
- plasma pressure $\sim 2 \times 10^{16} \text{ Pa}$
- total mass $m_{\odot} = 3.3 \times 10^5 m_{\oplus}$
- reaction rate $\langle \sigma v \rangle = 10^{-43} \text{ cm}^3/\text{s}$



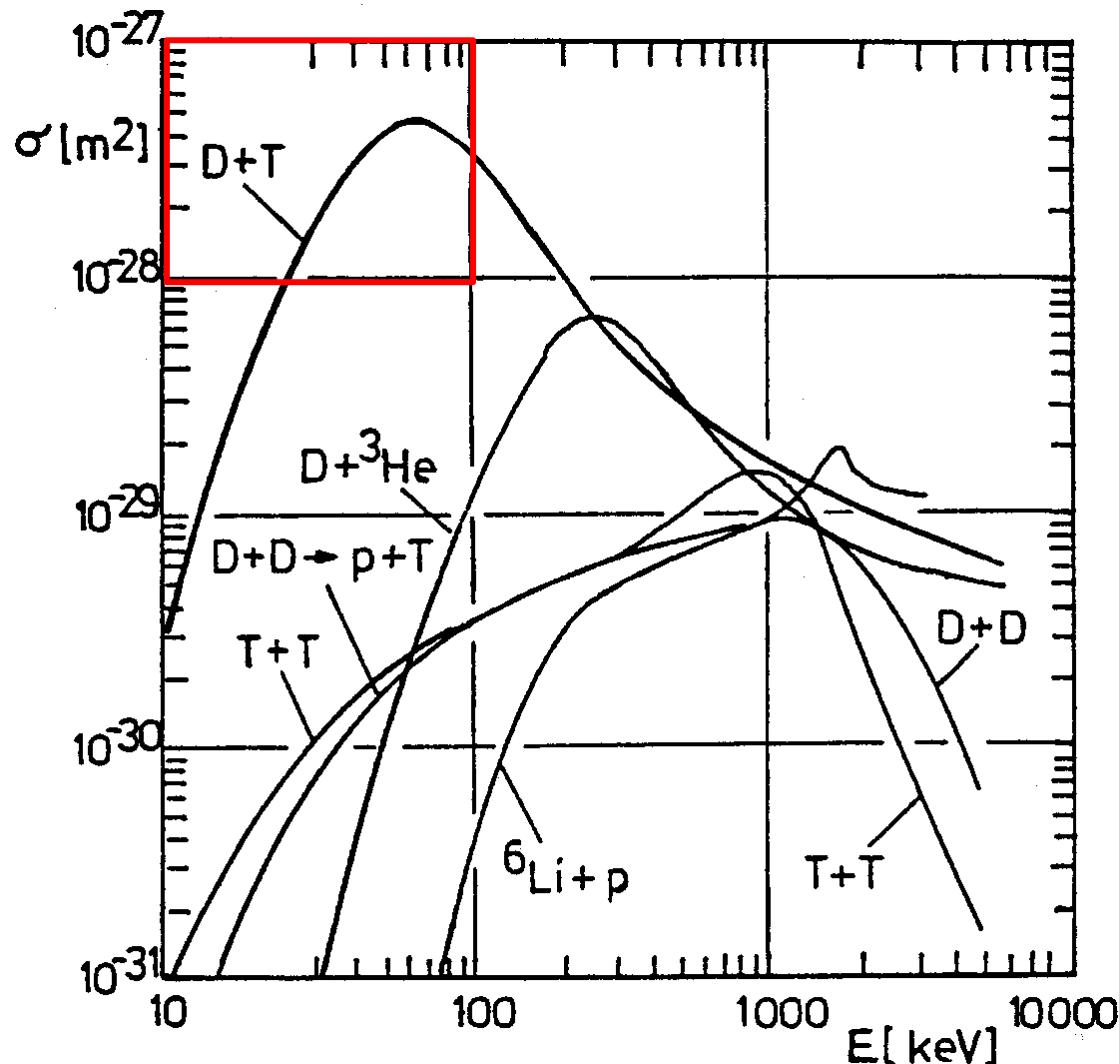
no!

but

D-T nuclear fusion – binding energy

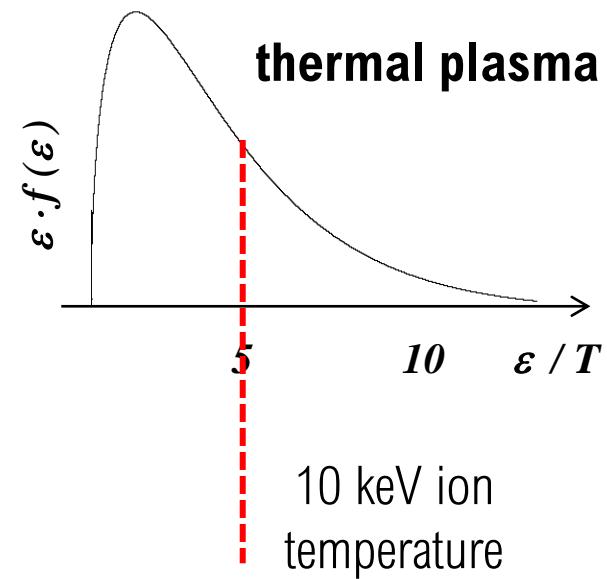


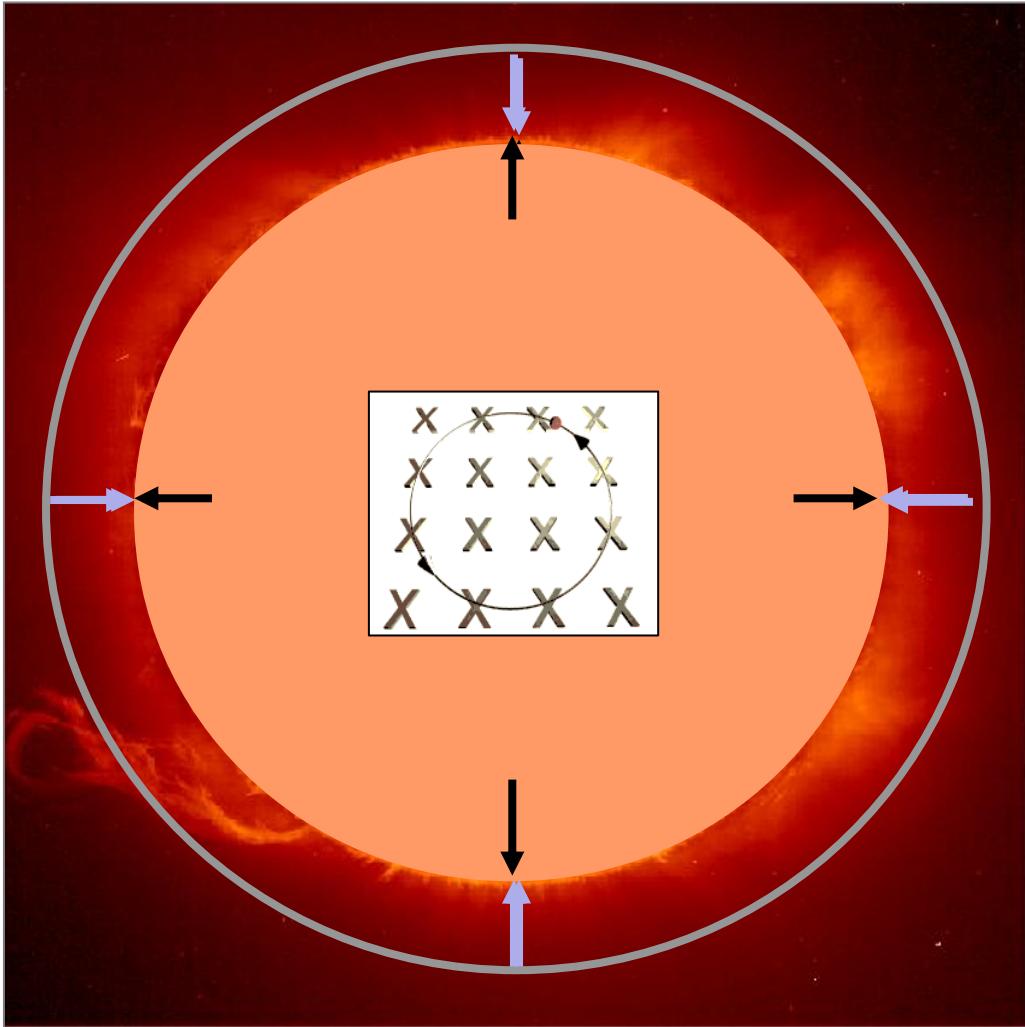
Fusion collision cross sections



σ for p + p fusion 20 orders of magnitude below

- D + D → 3 He + n + 3.27 MeV
- D + D → T + p + 4.03 MeV
- D + T → 4 He + n + 17.59 MeV
- D + 3 He → 4 He + p + 18.35 MeV





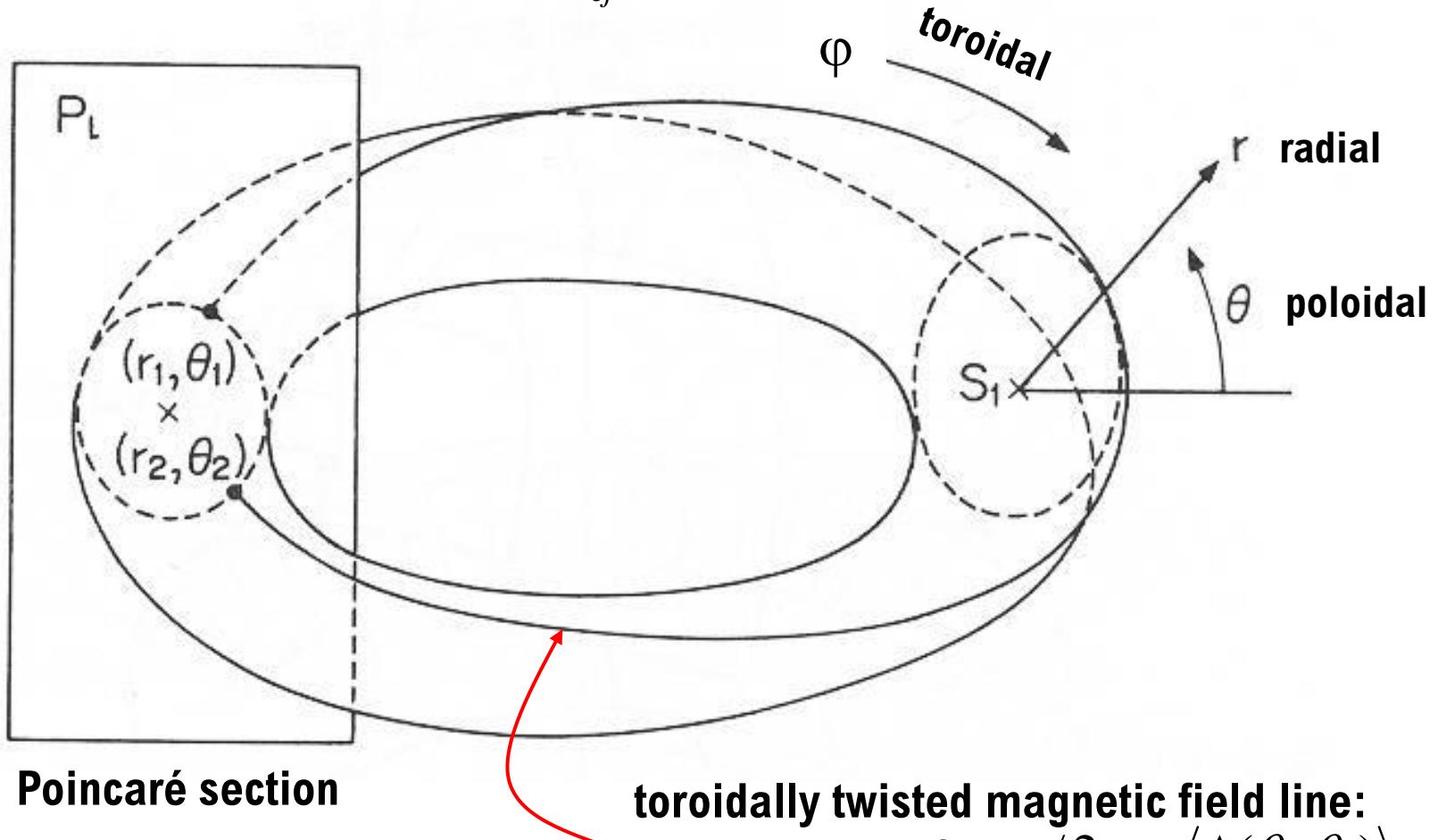
- high density solar plasma
- high plasma pressure
- sun's gravitation field
- gravitational confinement

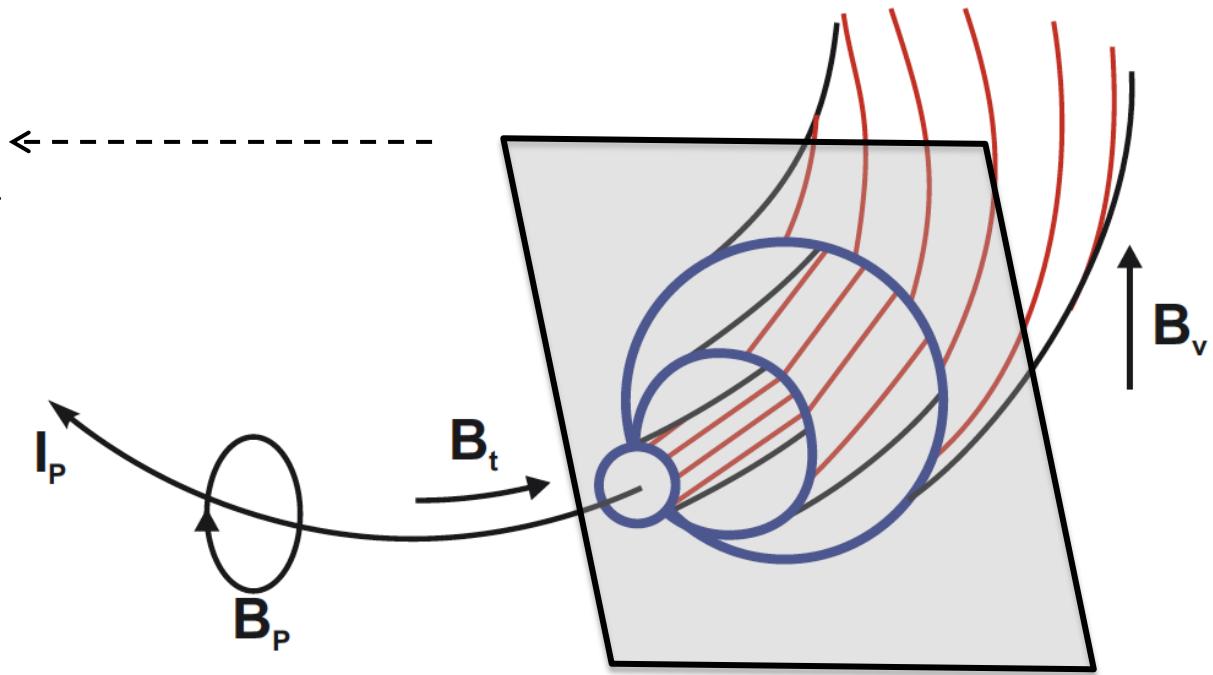
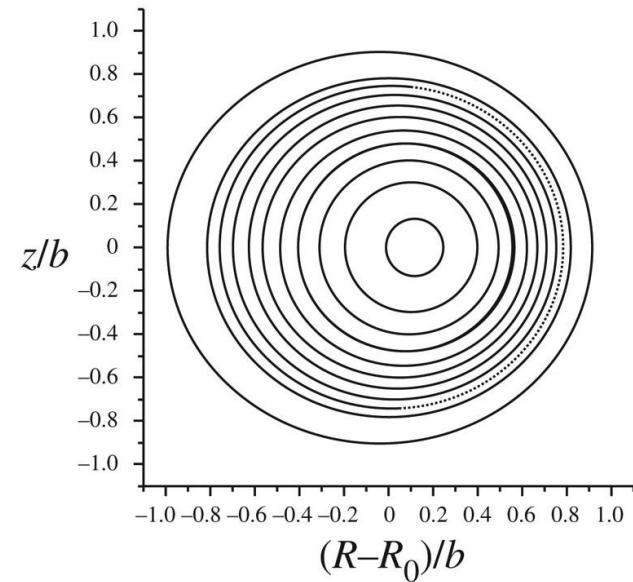
$$\beta = \frac{p_{kin}}{p_{mag}} = \frac{nk_B T}{B^2 / (2\mu_0)}$$

- evacuated plasma vessel
- low density fusion plasma
- plasma pressure O(1 bar)
- magnetic field → Lorenz force
- magnetic confinement

Toroidal magnetic fields

radial drift $v_R + v_{\nabla B} = \frac{v_{\parallel}^2 + v_{\perp}^2 / 2}{\omega_{cj}} \frac{\vec{B} \times \nabla \vec{B}}{B^2} \rightarrow \text{charge separation}$





Hamiltonian form

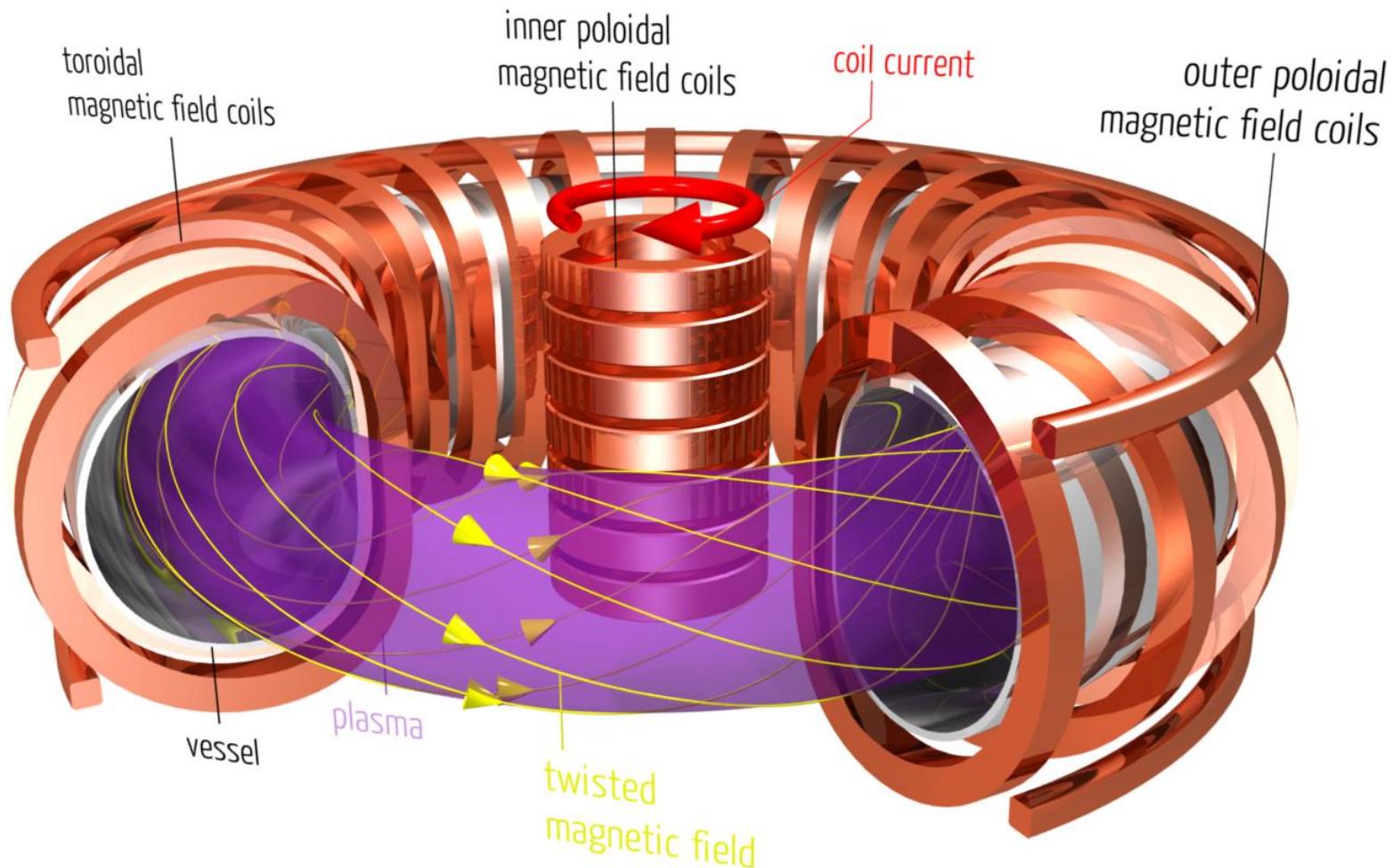
$$\frac{d\psi}{d\varphi} = -\frac{\partial H}{\partial \zeta} \quad \psi \text{ toroidal flux}$$

$$\frac{d\zeta}{d\varphi} = \frac{\partial H}{\partial \psi} \quad \zeta \text{ canonical angle}$$

Poincaré section

The tokamak device

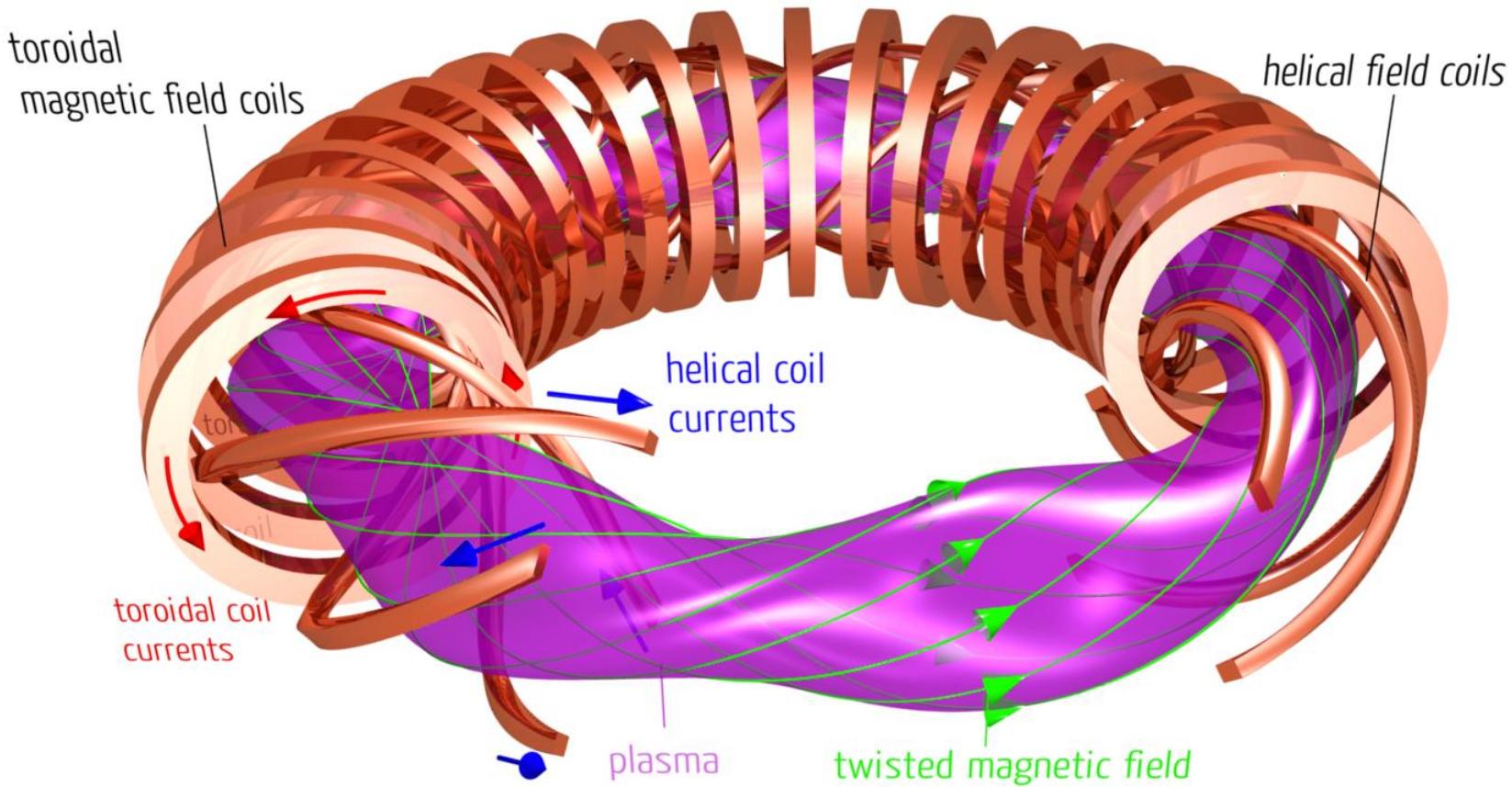
Tokamak (1951 Sacharov und Tamm) тороидальная камера в магнитных катушках „toroidal chamber in magnets“



The stellarator device

Stellarator (1951 Spitzer)

Stella = star
„bringing the star“

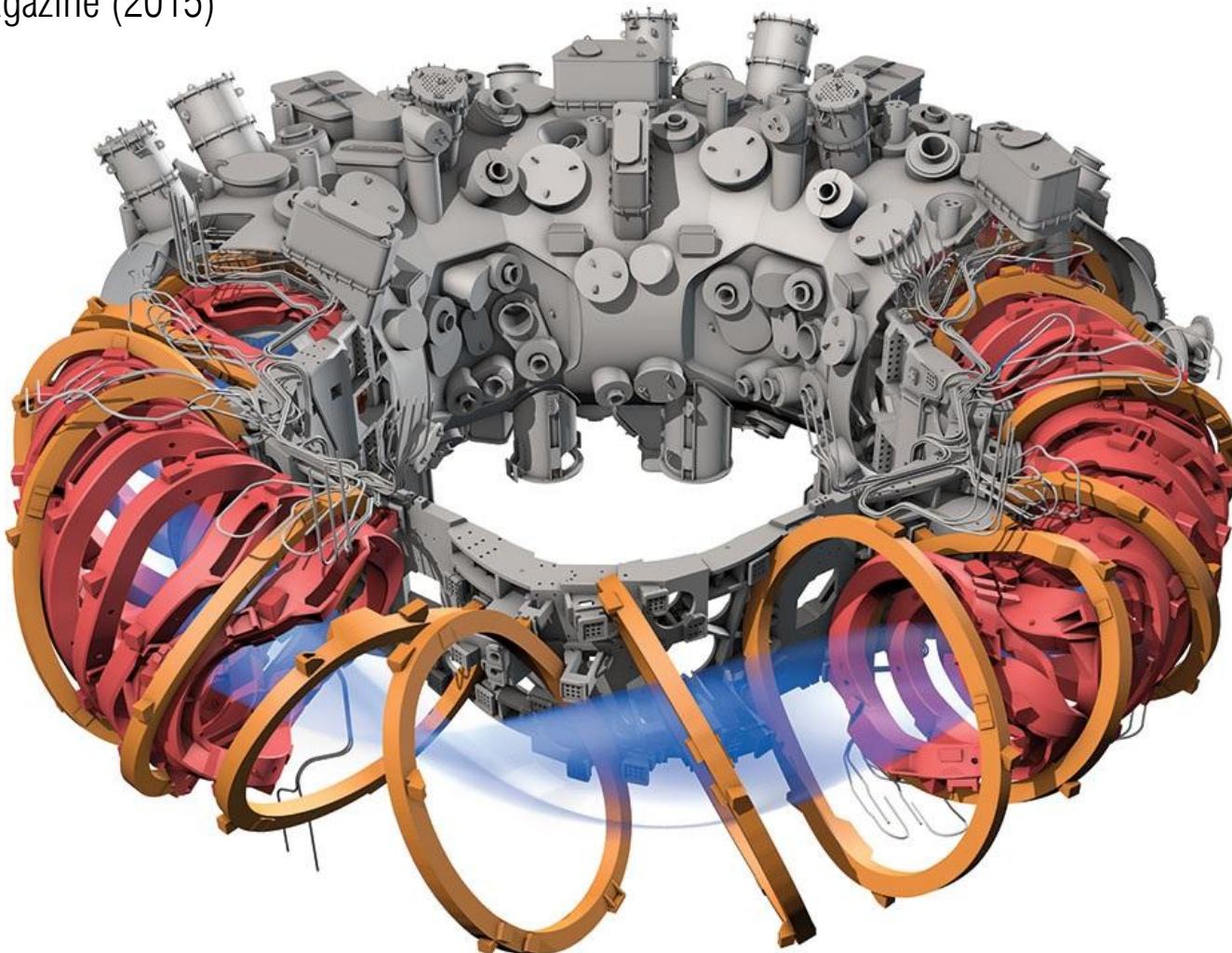


Stellarator now – how and why?

optimized stellarator (2015)

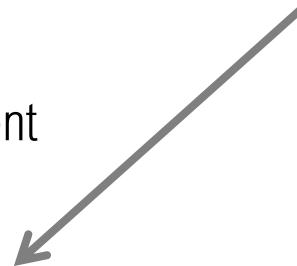
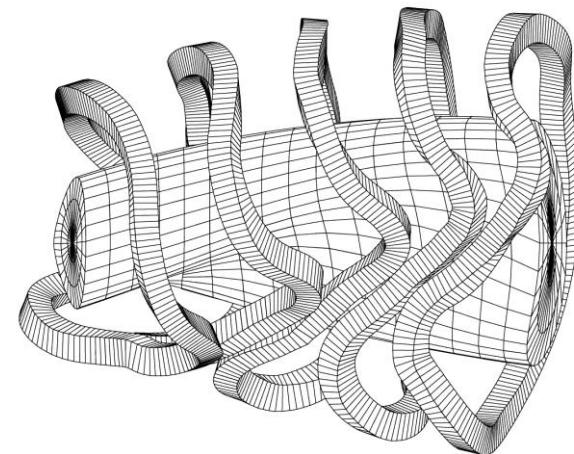
„Wendelstein 7-X“

cf. Science Magazine (2015)



seven optimisation criteria

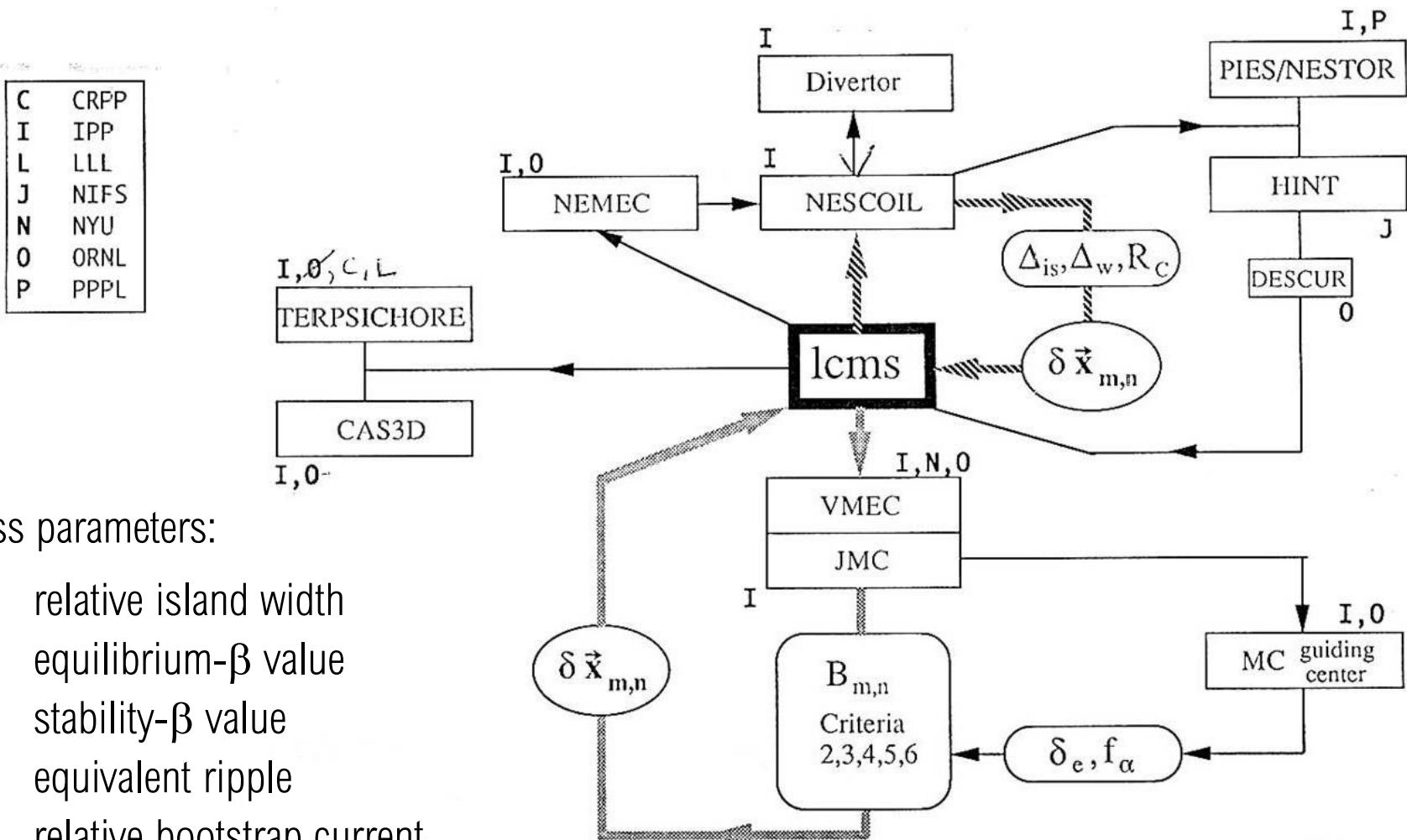
1. high quality of vacuum magnetic surfaces
2. good finite equilibrium properties @ $\langle \beta \rangle = 5\%$
3. good MHD stability properties @ $\langle \beta \rangle = 5\%$
4. reduced diffusive (neoclassical) transport
5. small equilibrium (bootstrap) current
6. good collisionless fast particle confinement
7. good modular coil feasibility



3d numerical codes

- vacuum field and coils
- MHD equilibrium
- MHD linear stability
- neoclassical transport
- Monte Carlo test particle
- edge and divertor

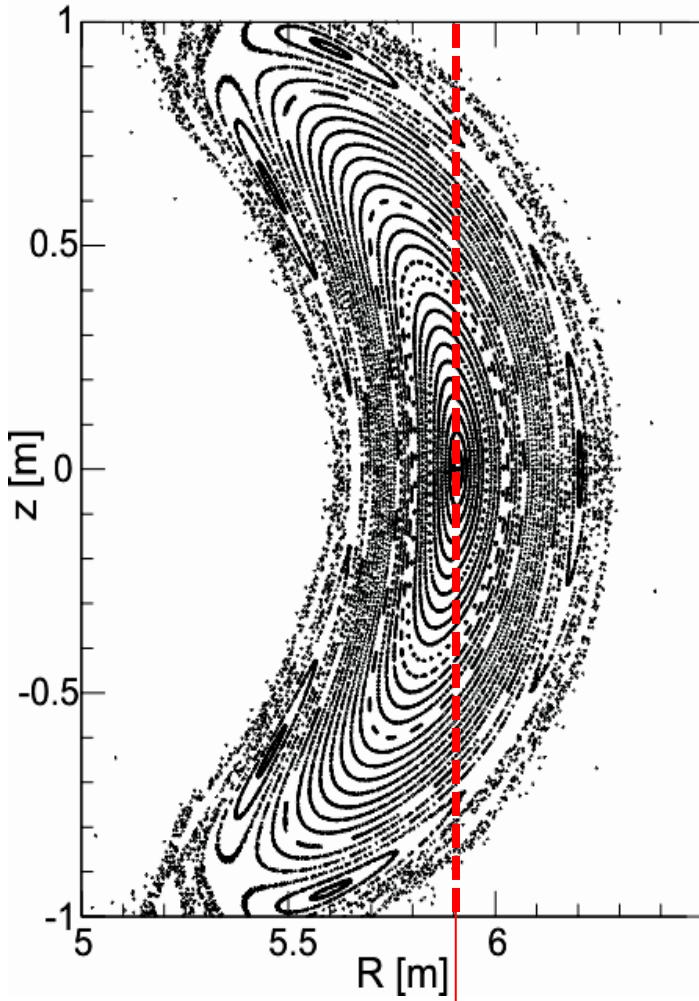
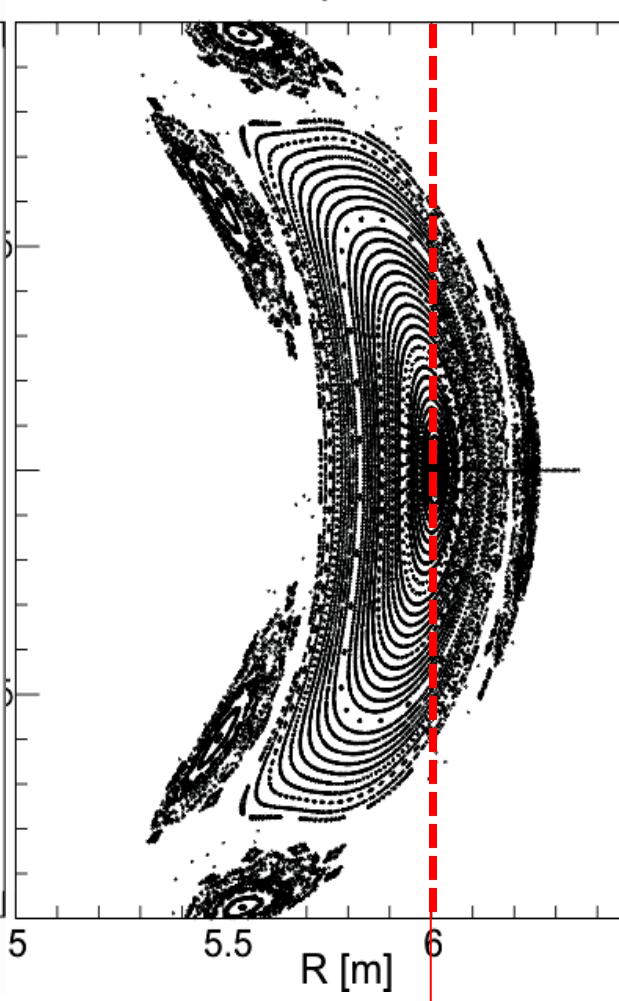
Example of a code map



goodness parameters:

- | | |
|----------------|--------------------------------------|
| Δ_{is} | relative island width |
| β_{eq} | equilibrium- β value |
| β_{stab} | stability- β value |
| δ_e | equivalent ripple |
| i_{BS} | relative bootstrap current |
| f_α | fraction of lost α -particles |
| ΔW | distance plasma-first wall |
| R_c | minimum coil curvature |

W7-X Vacuum Field

W7-X: $\langle \beta \rangle = 4\%$ 

plasma equilibrium

$$\nabla p = \vec{j} \times \vec{B}$$

bean shaped cross section

high plasma pressure



- **equilibrium stiff**
- **island location**
- **confined volume**
- **plasma location**

Shafarnov shift ~10cm

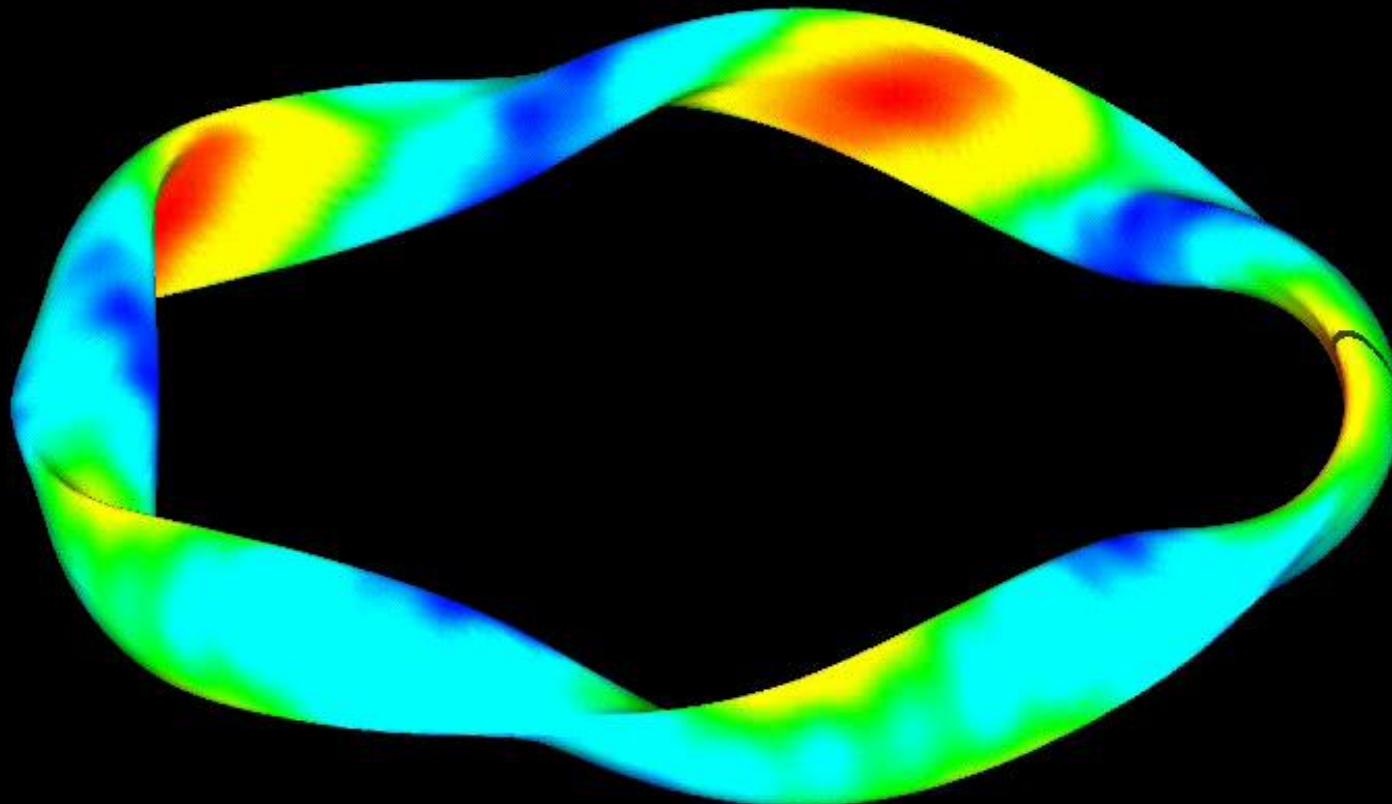




MAX-PLANCK-GESELLSCHAFT

Drift optimization 50 keV ions

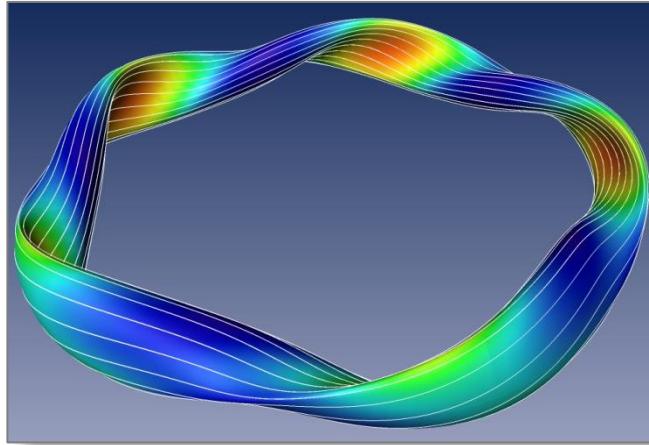
IPP



Tesla

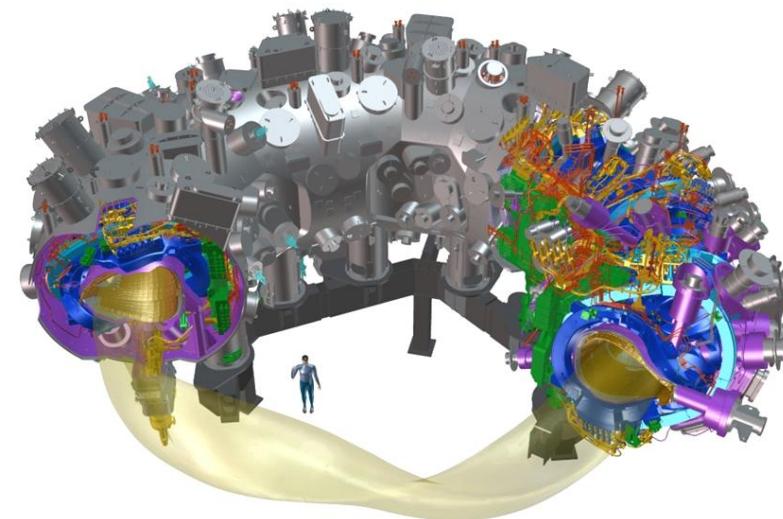
2.0 2.2 2.4 2.6 2.8 3.0

Facts and figures



- five magnetic field periods
- modular non-planar coils
- optimized plasma equilibrium
- low equilibrium current $\rightarrow O(10 \text{ kA})$
- high iota and low shear
- flexible magnetic field configurations

- 735 t mass with 435 t cold mass
- 70 superconducting NbTi coils
- 14 HTSC current leads
- 3 T magnetic induction on axis
- 254 ports of 120 different types
- 30 m³ plasma volume
- 265 m² in-vessel components
- 4.5m height and 16 m diameter

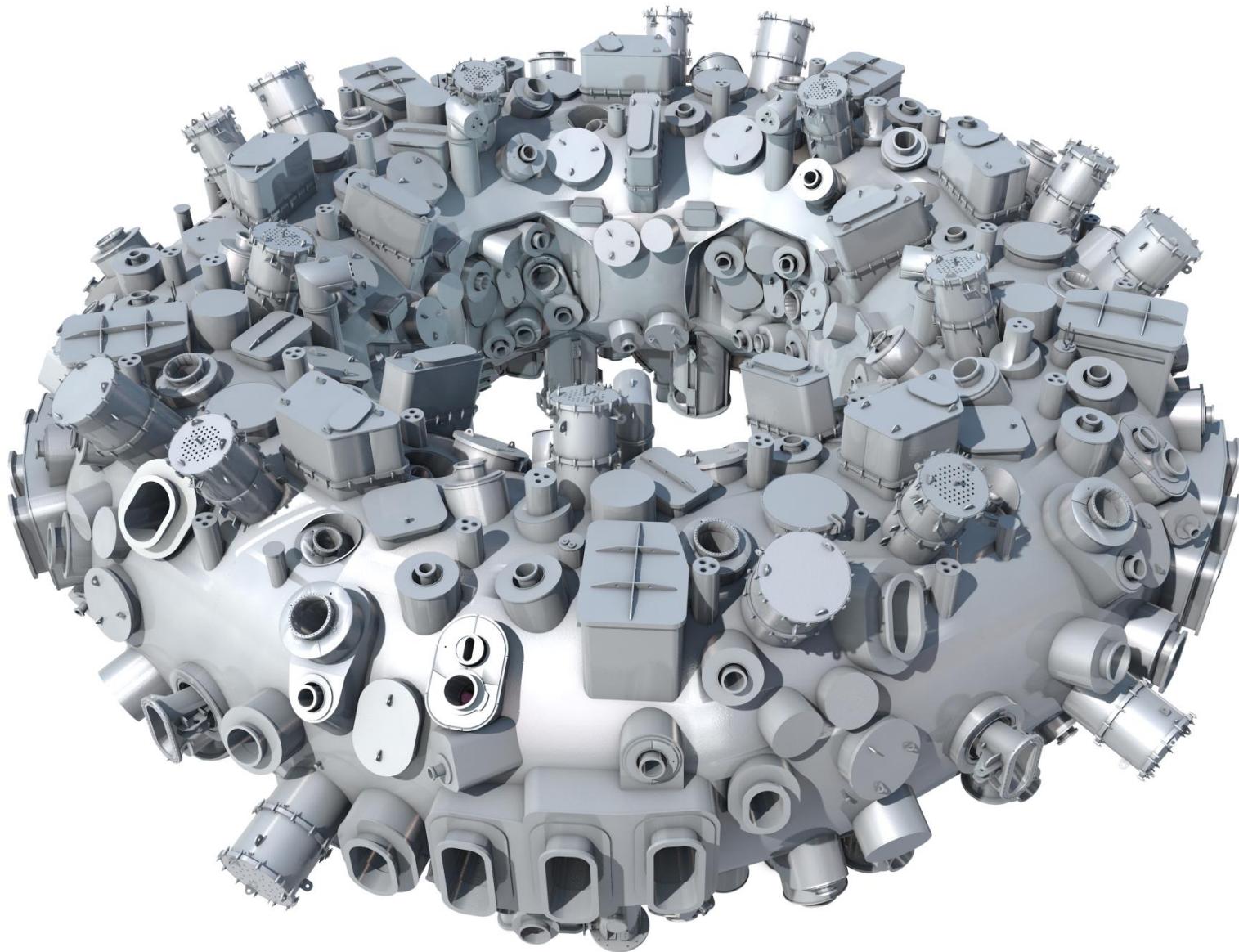




MAX-PLANCK-GESELLSCHAFT

Major elements of Wendelstein 7-X

IPP



Cryostat vessel and thermal insulation



Superconducting magnets



coil in the assembly handling unit



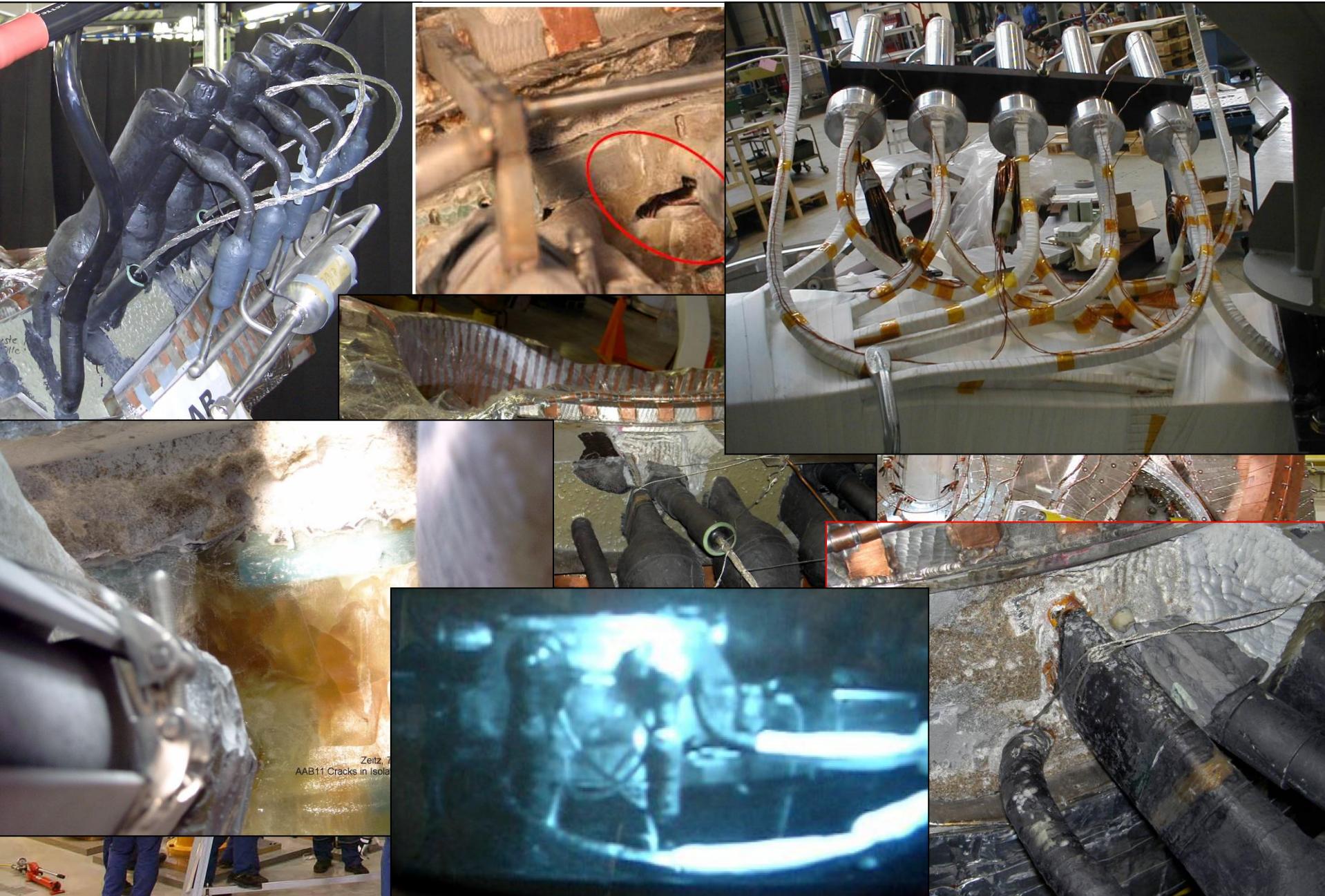
coil in the test bed @ CEA Saclay



MAX-PLANCK-GESELLSCHAFT

Magnet manufacturing was a pain

IPP

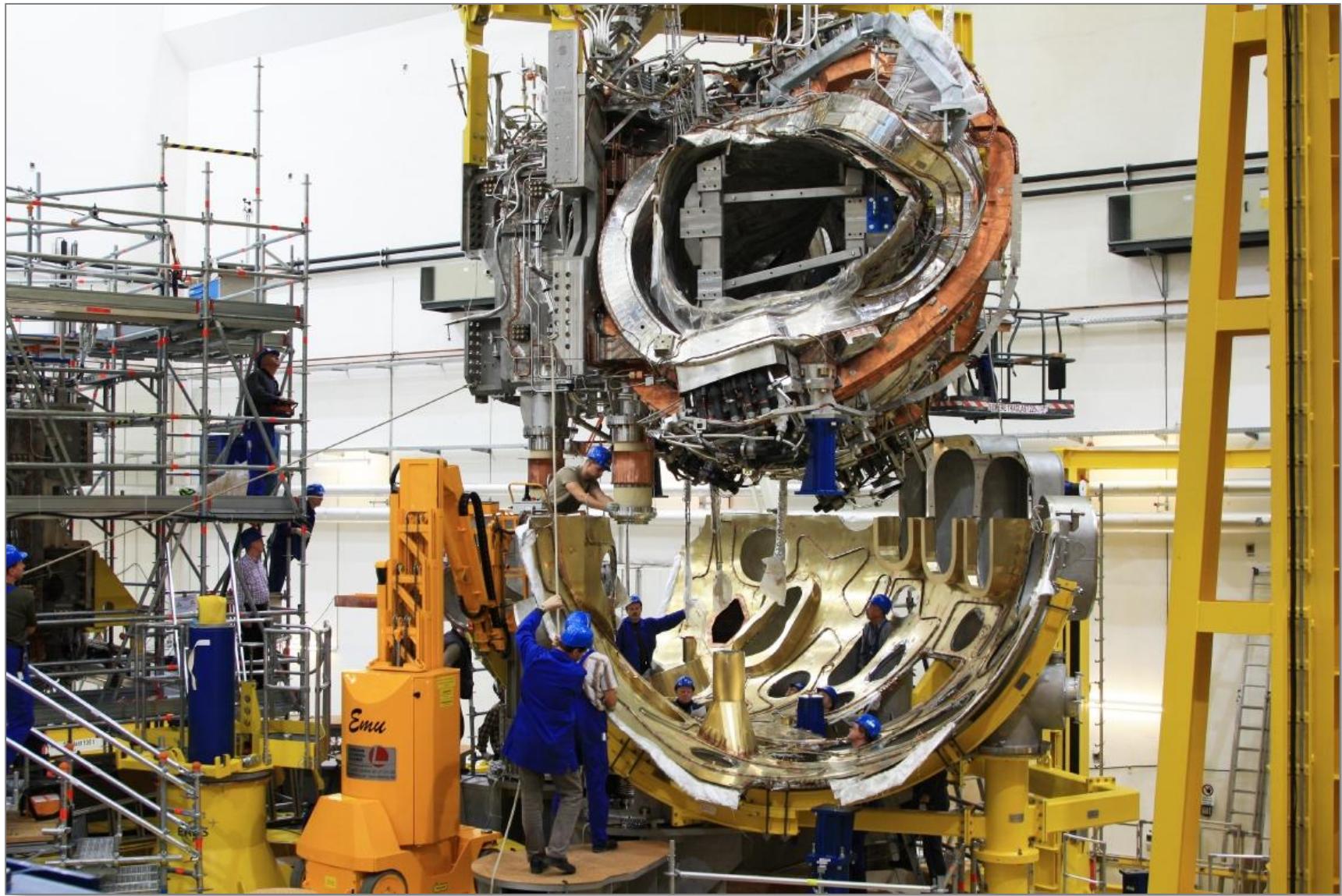




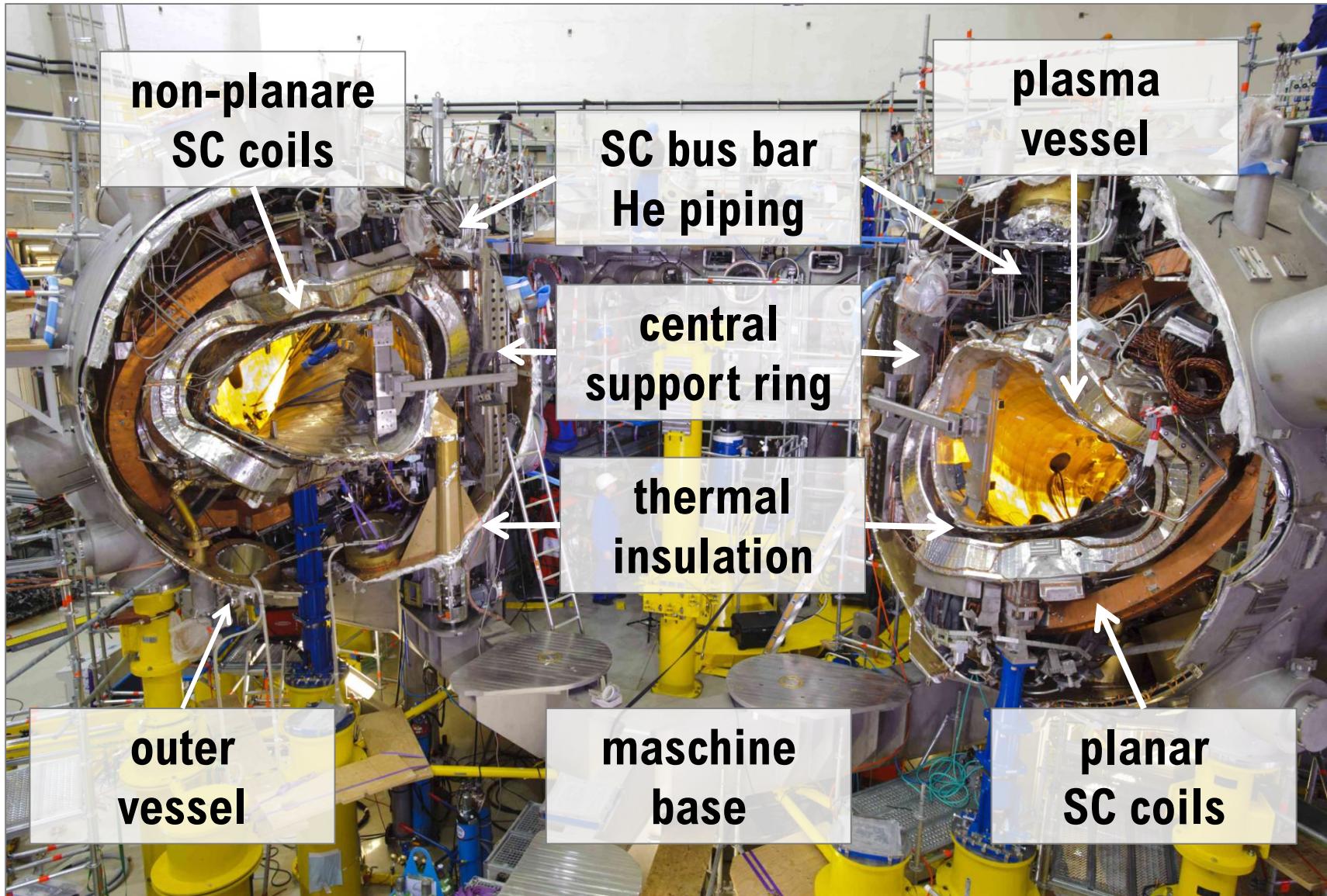
MAX-PLANCK-GESELLSCHAFT

Integration of magnets and cryostat

IPP



Four out of five modules

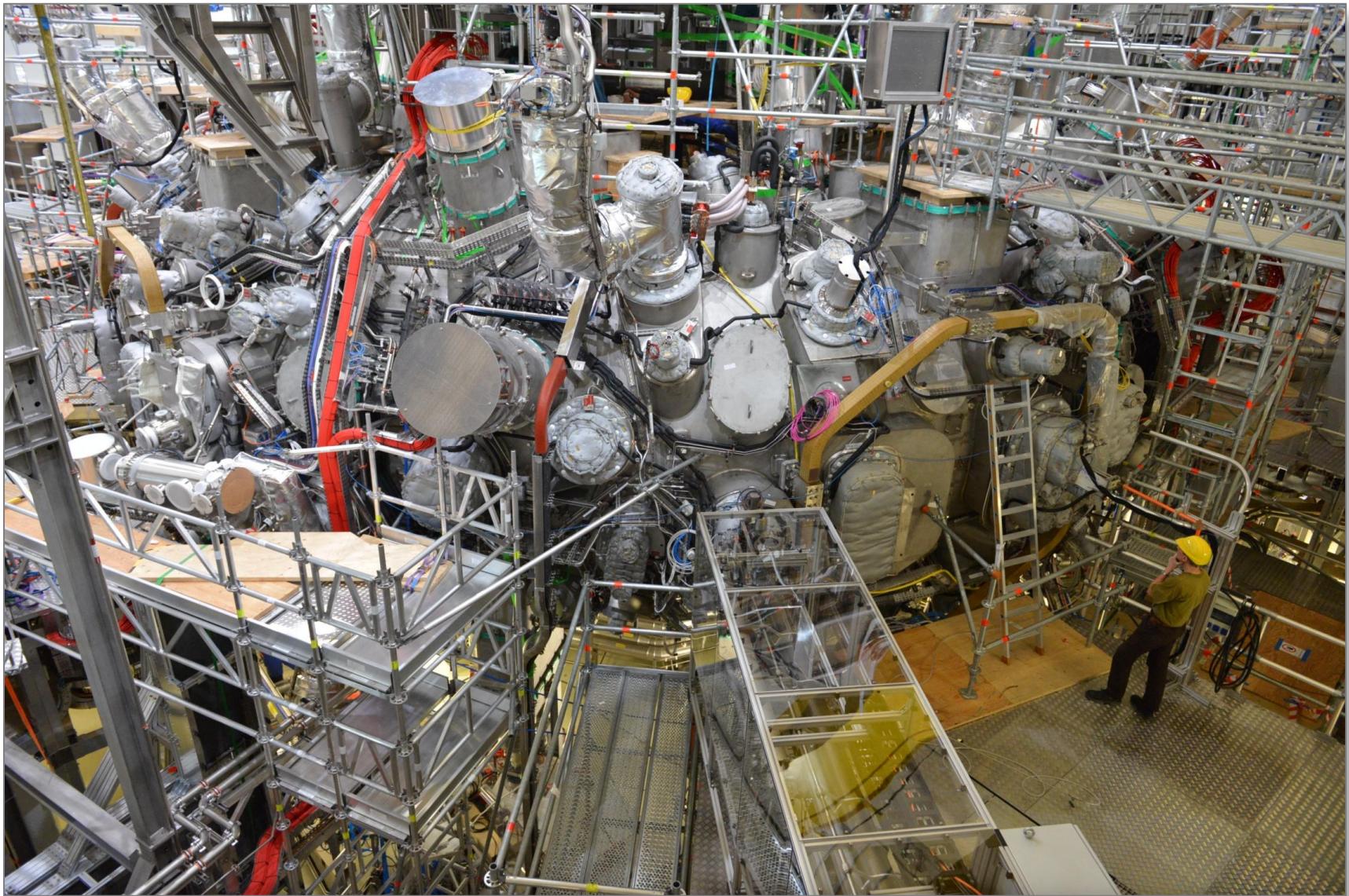




MAX-PLANCK-GESELLSCHAFT

The device is complete

IPP





First He plasma fast video camera recording

IPP

MAX-PLANCK-GESELLSCHAFT



Key parameters:

drive power 4 MW

drive frequency 140 GHz

magnetic field on axis 2.5 T

pressure $5 \cdot 10^{-4}$ mbar

Key parameters:

electron density $2 \cdot 10^{19} \text{ m}^{-3}$

■ electron temperature 4 keV

now standard hydrogen plasmas

electron temperature 10 keV

ion temperature 2 keV

pulse length 1- 7 s

impurities from the wall!

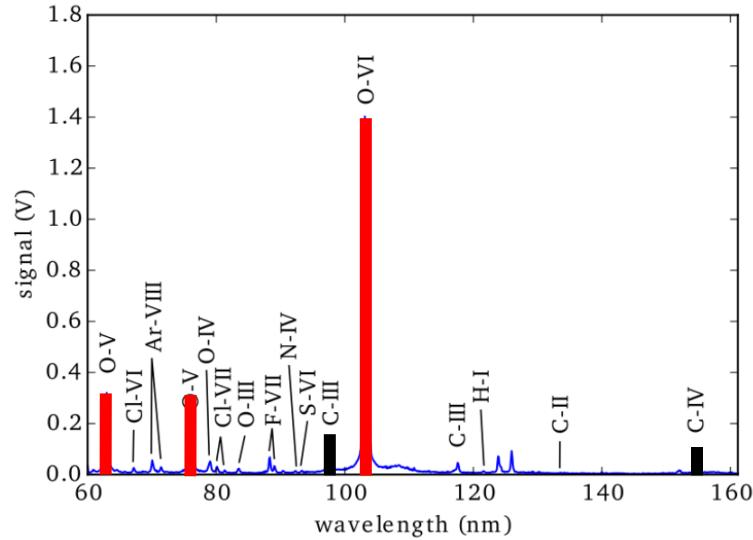
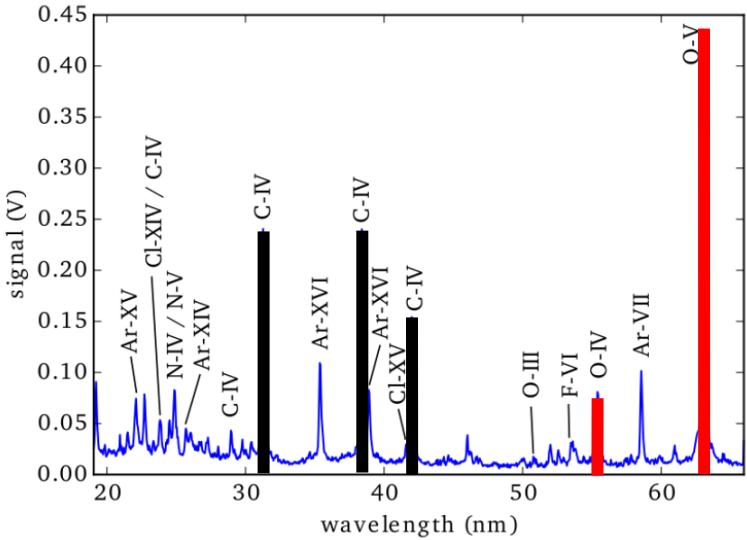
AEQ41_edi_20151215_121657.h5

Time: 51 ms after T1

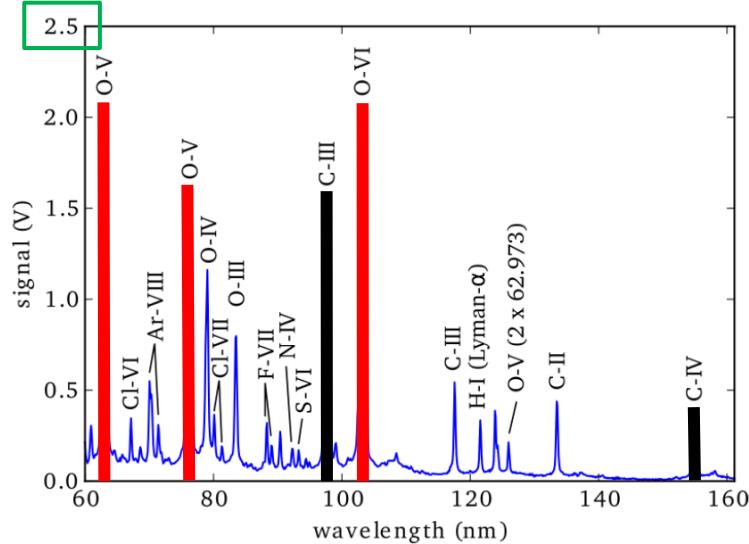
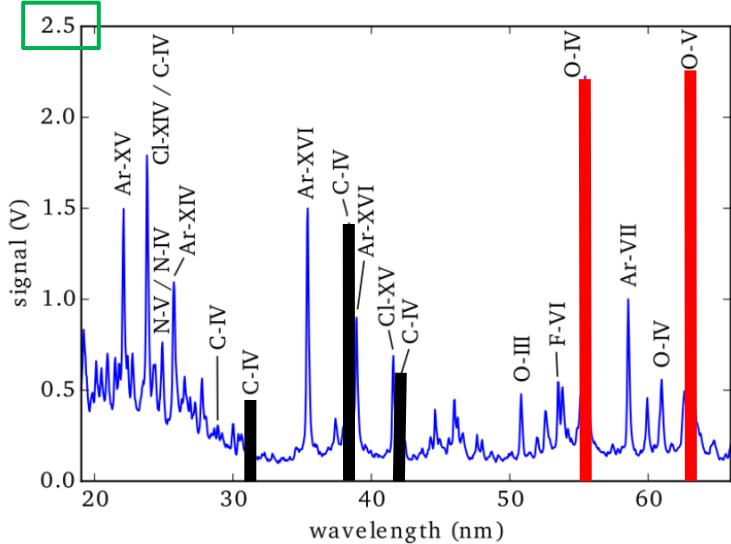
W7-X EDICAM video system

high resolution overview spectrometer wavelength range 20 – 160 nm

before
radiation
collapse



during
radiation
collapse



Overview of the first operation phase

The 10 weeks if the first operation phase has exceeded all our expectations.
In the end 25 diagnostic systems were commissioned and delivered data.

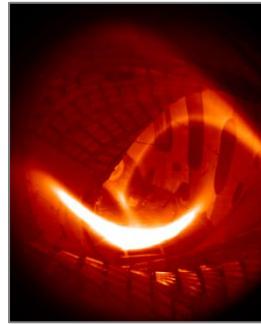


$$T_e = 1 \text{ keV}$$

$$T_i < 1 \text{ keV}$$

$$n_{e0} \sim 2 \cdot 10^{19} \text{ m}^{-3}$$

$$t_d = 50 \text{ ms}$$



$$T_e = 7 \text{ keV}$$

$$T_i = 1.2 \text{ keV}$$

$$n_{e0} = 3 \cdot 10^{19} \text{ m}^{-3}$$

$$t_d = 250 \text{ ms}$$

$$T_e = 8 \text{ keV}$$

$$T_i = 1 \text{ keV}$$

$$n_{e0} = 2 \cdot 10^{19} \text{ m}^{-3}$$

$$t_d = 6 \text{ s}$$

10.12.2015 1st He Plasma	2 MJ	3.2.2016 1st H Plasma	4 MJ
------------------------------------	-------------	---------------------------------	-------------

$$T_e = 8 \text{ keV}$$

$$T_i = 2 \text{ keV}$$

$$n_{e0} = 3 \cdot 10^{19} \text{ m}^{-3}$$

$$t_d = 500 \text{ ms}$$

$$T_e = 10 \text{ keV}, 7 \text{ keV}$$

$$T_i = 1 \text{ keV}, 2.1 \text{ keV}$$

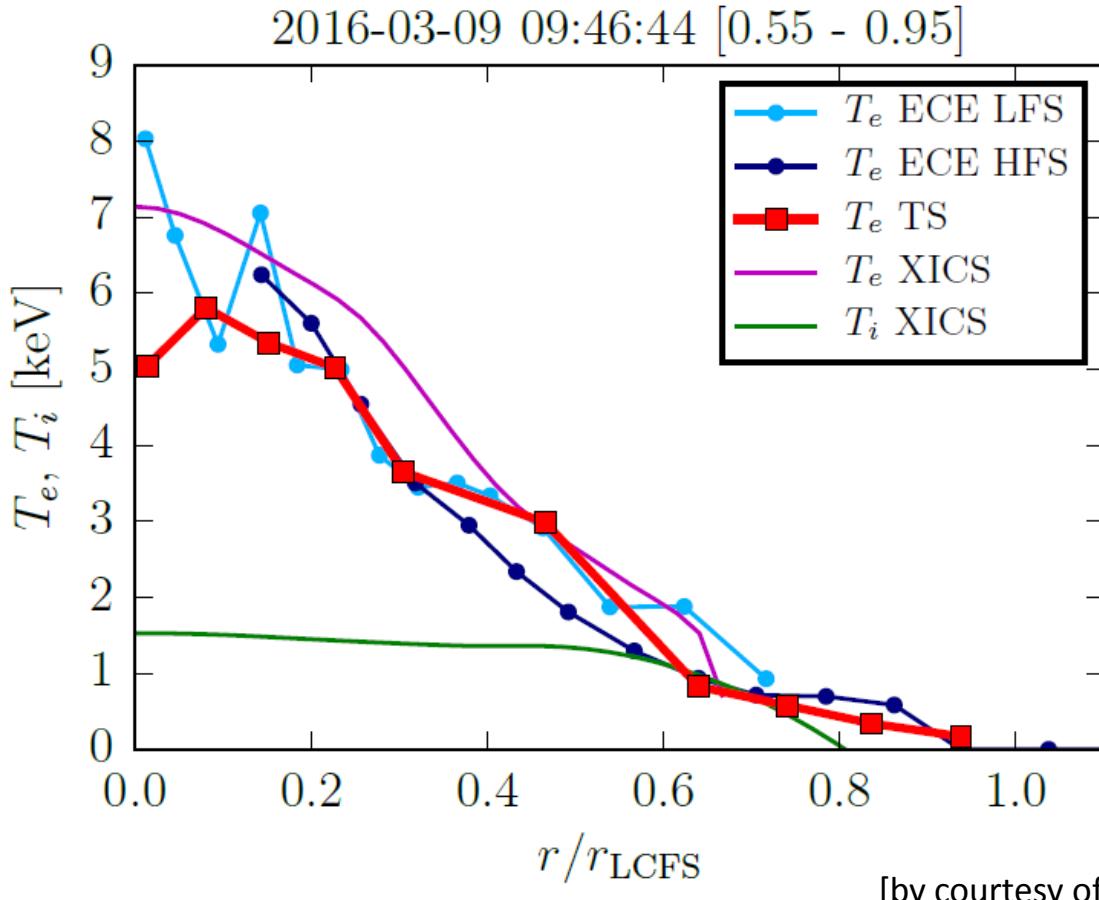
$$n_{e0} = 5 \cdot 10^{19} \text{ m}^{-3}$$

$$t_d = 1 \text{ s}$$

~ 1000
experiments

On the 10th of March 2016 the operation was suspended as planned.

Typical radial temperature profiles



[by courtesy of S. Bozhenkov]

X-ray imaging crystal spectrometer

$500 \text{ eV} < T_e < 10 \text{ keV}$

$500 \text{ eV} < T_i < 4 \text{ keV}$

Ar impurity seeding

plasma densities

$n_e < 4-5 \cdot 10^{19} \text{ m}^{-3}$

Thomson scattering system

$20 \text{ eV} < T_e < 10 \text{ keV}$, $5 \cdot 10^{18} \text{ m}^{-3} < n_e < 5 \cdot 10^{20} \text{ m}^{-3}$, $\Delta r_{\text{eff}} \approx 2-3 \text{ cm}$

two Nd-YAG lasers 2J/pulse @ $f=20 \text{ Hz}$, laser beam $\emptyset=5-7 \text{ mm}$

Numerous research topics addressed

radial electric field and
plasma rotation

heat wave experiments
for confinement studies

identification of
impurities and
radiation collaps

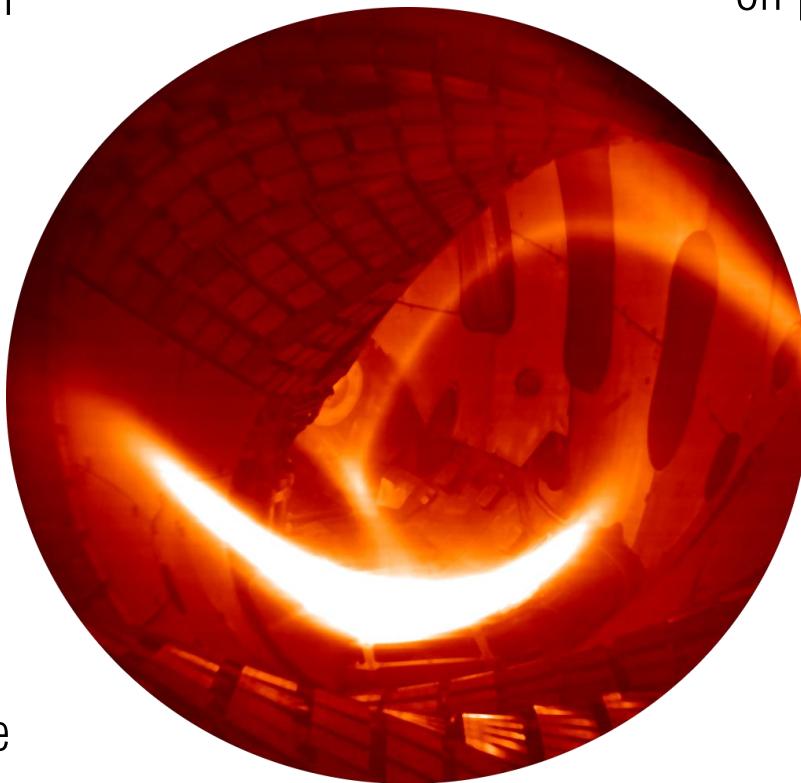
filamentation
in the plasma edge

magnetic configuration influence
on particle transport

electron cyclotron
current drive

advanced
heating schemes

limiter heat loads
and influence of
trim coils

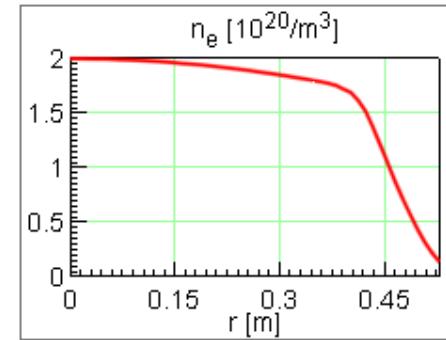
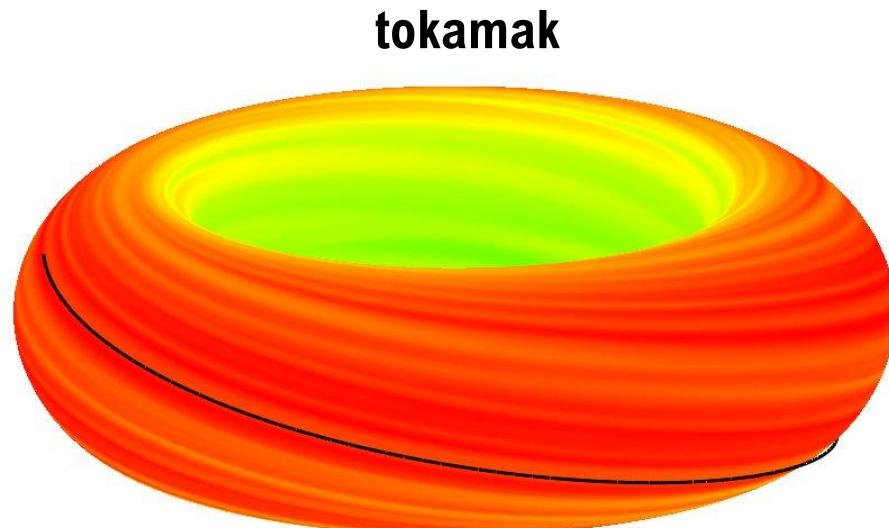


EUROfusion

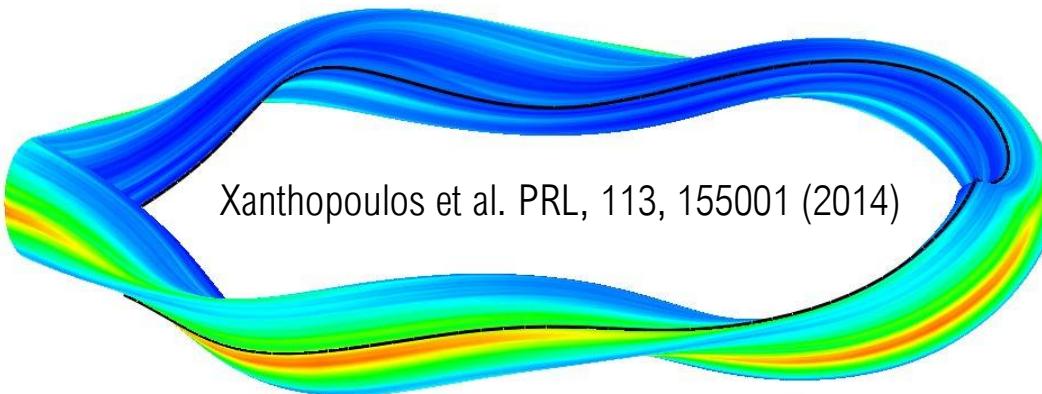


turbulence
amplitude

86.2
81.3
76.4
71.5
66.5
61.6
56.7
51.7
46.8
41.9
37.0
32.0
27.1
22.2
17.3

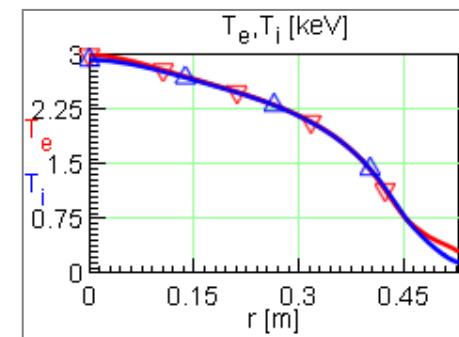


optimized stellarator



Xanthopoulos et al. PRL, 113, 155001 (2014)

∇T_i driven
turbulence



Wendelstein 7-X „by chance“ turbulence optimized?



MAX-PLANCK-GESELLSCHAFT

Turbulence in a tokamak – gyrokinetic code

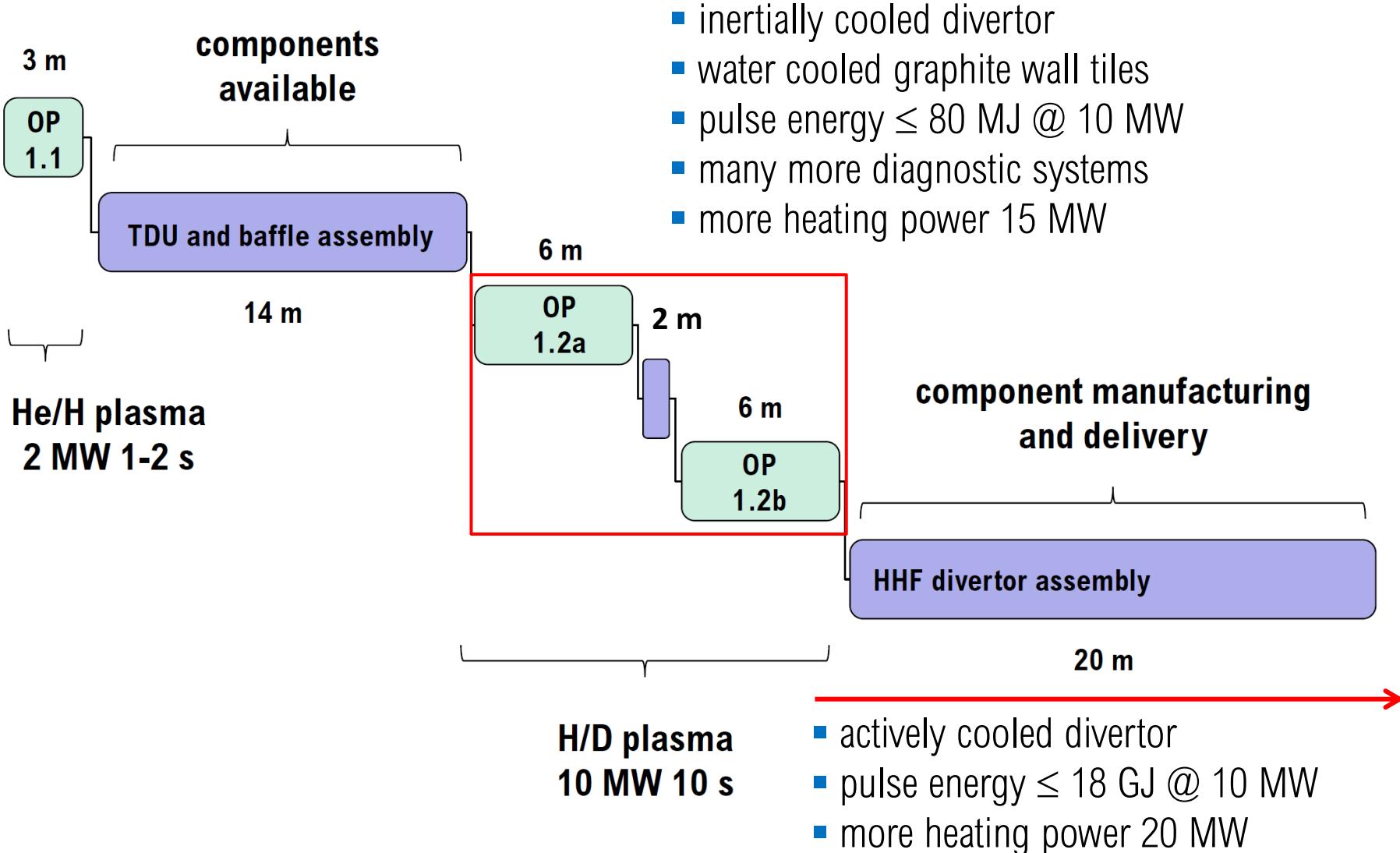
IPP

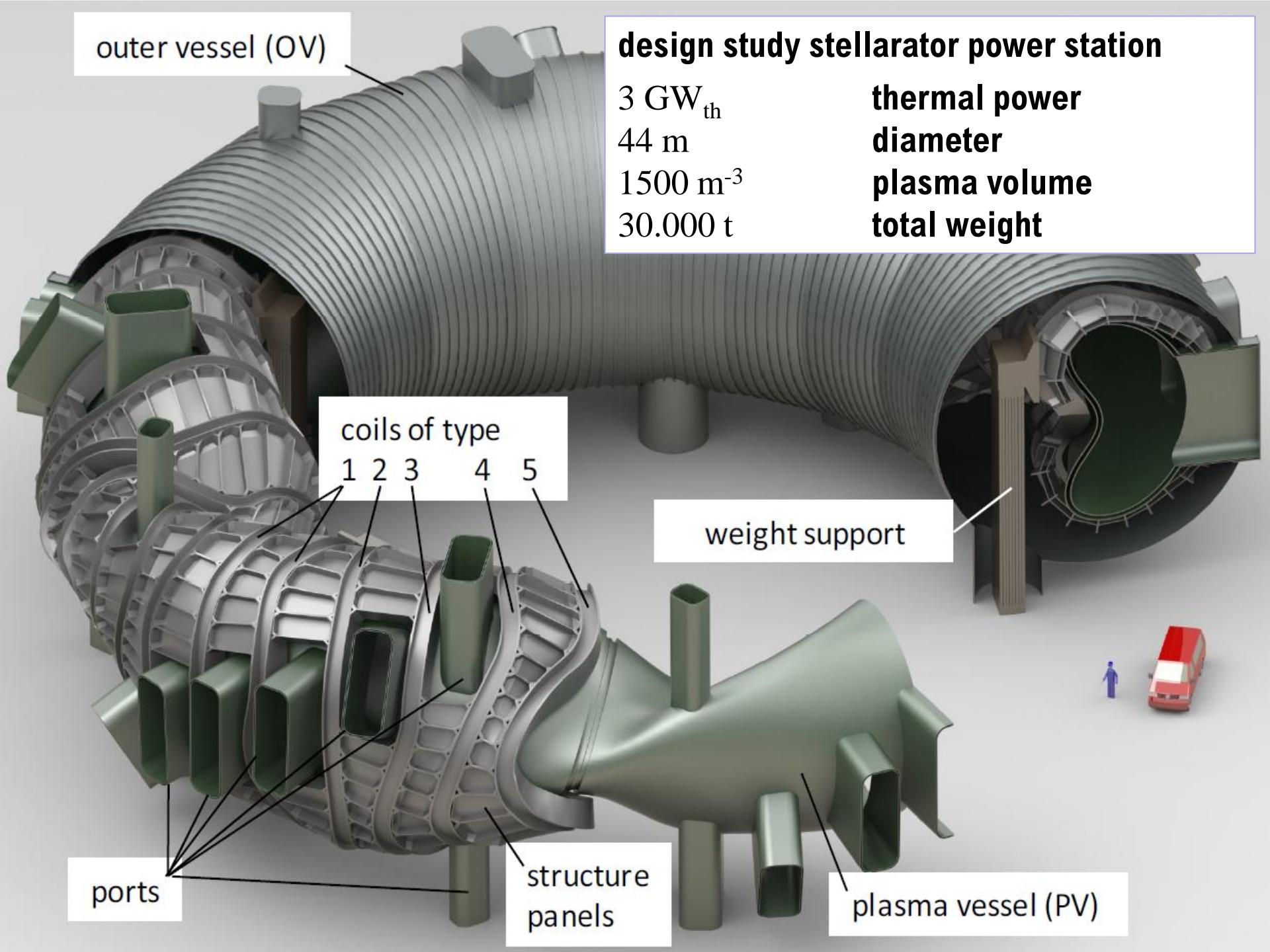
Global Gyrokinetic Simulation of
Turbulence in
ASDEX Upgrade



gene.rzg.mpg.de

The mid term plan for Wendelstein 7-X





outer vessel (OV)

design study stellarator power station

3 GW_{th}

44 m

1500 m⁻³

30.000 t

thermal power
diameter
plasma volume
total weight

coils of type

1 2 3 4 5

ports

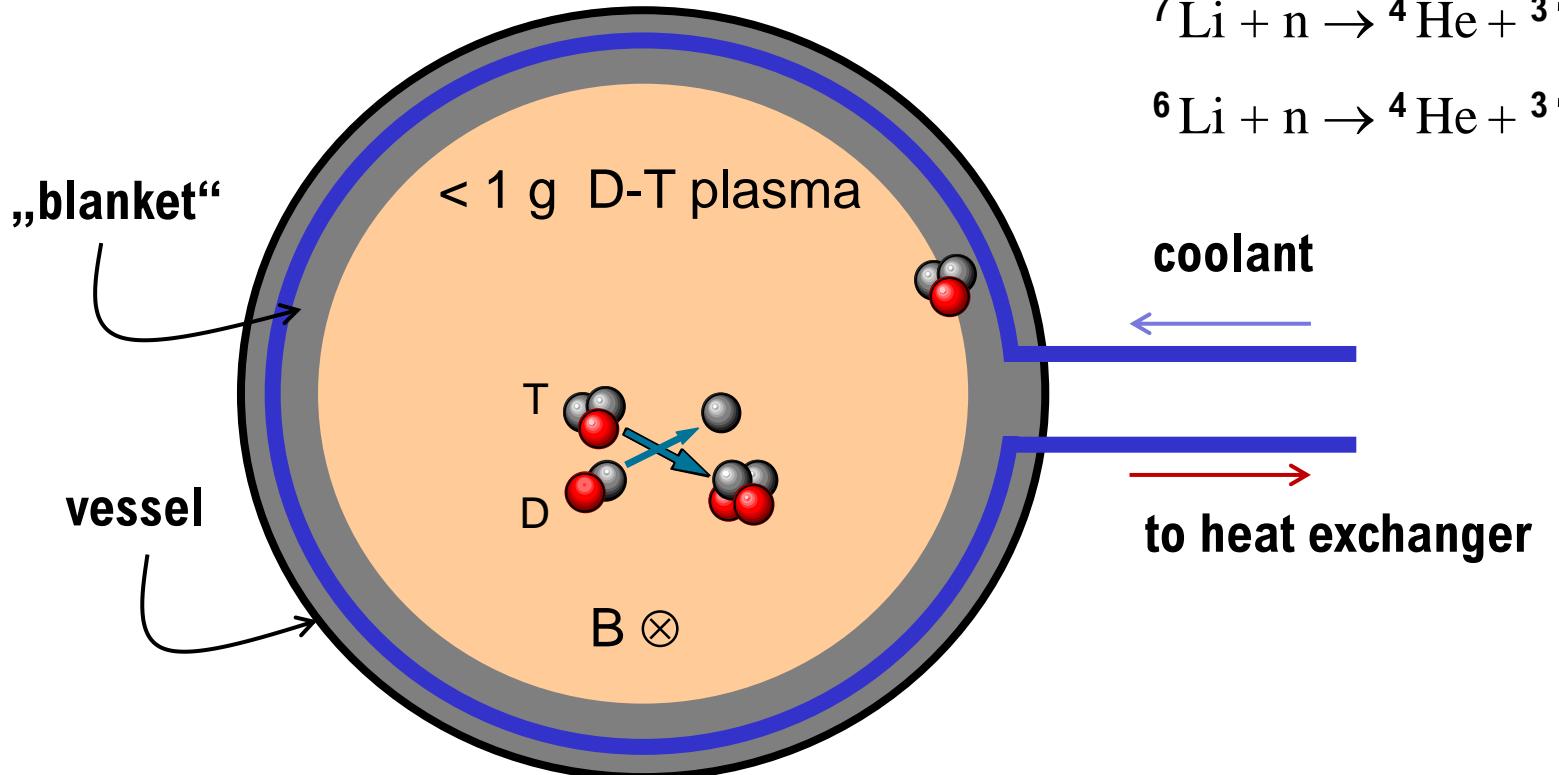
structure
panels

plasma vessel (PV)

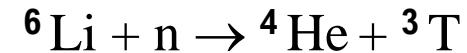
weight support

The reactor scheme

D concentration in water 0.015%



in situ T generation

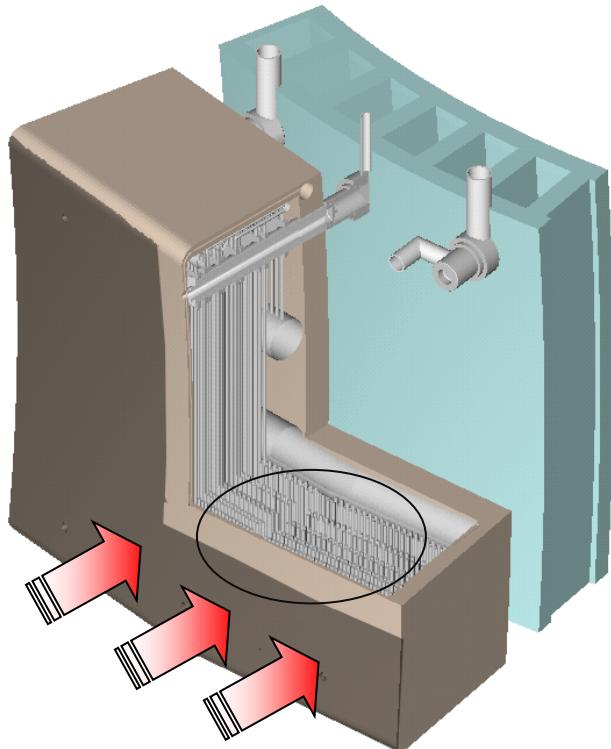


coolant

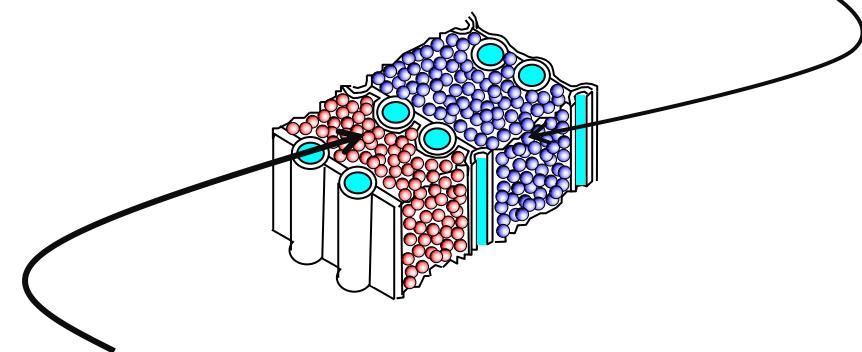
to heat exchanger

a fully ionized D-T-plasma at 10 keV ion temperature with an average particle density $n_d \sim n_T \sim 10^{20} \text{ m}^{-3}$ yields a fusion power density $P/V \sim 2 \text{ MW/m}^3$

The blanket – a critical component



neutron multiplier Be, Be_{12}Ti



Tritium breeder
 Li_2TiO_3 , Li_2O

critical is the T self-sufficiency

a comprehensive research program is needed and must be intensified – first test blanket modules will be investigated in the ITER tokamak (under construction)

<https://www.iter.org/mach/Blanket>

<https://www.fusion.kit.edu>

<http://www-fusion-magnetique.cea.fr>



- Wendelstein 7-X is an optimized SC stellarator
- completed after 15 years of construction
- the machine works perfectly fine
- the first plasma operation with He and H has started
- physics program based on a staged approach to steady-state

Why fusion at all and why stellarators?

- a new primary energy source – probably needed in future
- backbone power stations of the GW class
- basic research on magnetized high-temperature plasmas needed
- the stellarator promises stable steady-state operation
- Wendelstein 7-X is the key experiment for reactor extrapolation