

Polarized Fusion, its implications and plans for Direct Measurements in a Tokamak Plasma

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for

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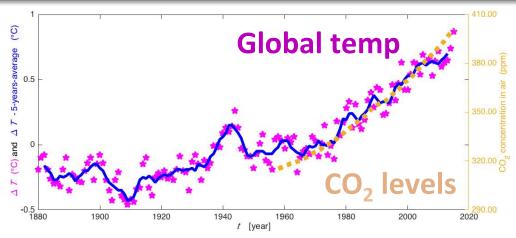




The need for fusion power

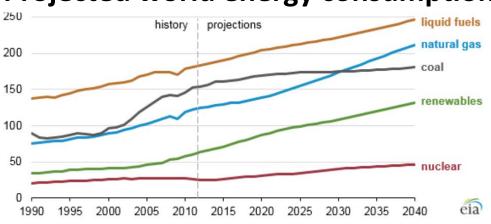
The world is burning!

- dramatic consequences cannot be avoided
- Q: is there anything that we have learned from spin which could mitigate them?
- A: not in the short term, but in the longer term, yes quite possibly, in the push towards fusion power



G. Ciullo, *Nuclear Fusion with Polarized Fuel*, Springer Proc. Phys., Springer-Verlag (2016)

Projected world energy consumption



US Energy Information Administration





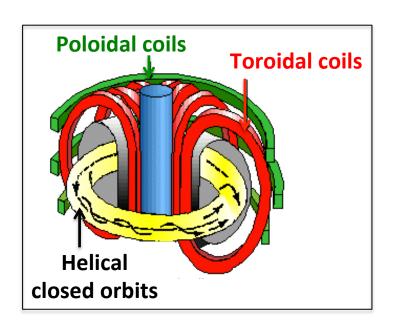




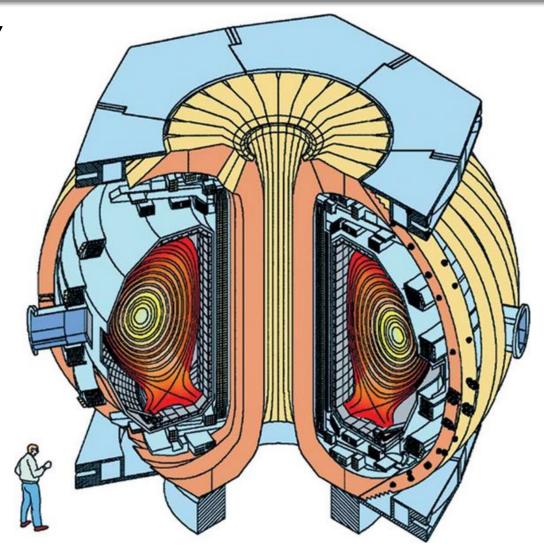
The road to fusion through magnetic confinement - preliminaries

$D+t \rightarrow \alpha + n$, Q = 17.6 MeV

 \Leftrightarrow though the low-energy tail of a J = 3/2 resonance in 5 He



- combination of Toroidal and Poloidal fields produces confined helical orbits
- ⇔ multiple chances for fusion



DIII-D tokamak (San Diego / USA)







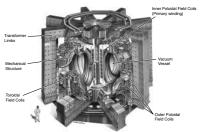


The road to fusion through magnetic confinement - preliminaries

- about 180 research tokamaks have been built; there are currently about 30 in operation
- mostly studying **D+D** reactions

quantum leap towards fusion power: Int. Thermonuclear Experimental Reactor

> JET (Oxfordshire / UK)



only machine that can run with tritium

A.M. Sandorfi – SPIN'16

1/2 GW reactor - under construction

ITER

(Cadarache / France)



best instrumented -most diagnostics

DIII-D

(San Diego / USA)









The road to fusion through magnetic confinement

	V(plasma)	B _c (tesla)	P(MW)	Q = P(fus)/P(in)	coils
DIII-D	20 m ³	2.1	_	<<1	normal
JET	90 m ³	3.8	16	~ 2/3	normal
ITER	700 m ³	5.3	500	~ 10	superconducting

- superconducting coils are needed to reach high field over a large volume
 concerns over potential field degradation from neutron flux
- ITER is to be a stepping-stone, requiring at least one more iteration to reach a viable fusion power plant
- Plant costs ~ V(plasma) x (B_c)²

⇔ *eq.* **20 – 40 B\$ for ITER**

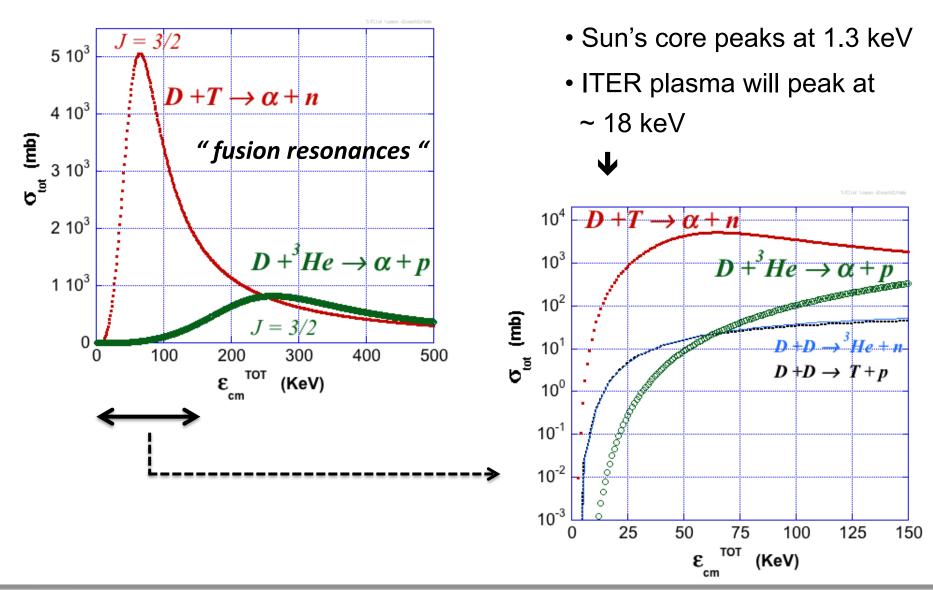








Reactions through the low energy tails of fusion resonances (in the Sun or in a tokamak)











The potential of SPIN

- fusion fuels: $D + T \Rightarrow \alpha + n$; (and $D + {}^{3}He \Rightarrow \alpha + p$)
 - \leftrightarrow dominated by J=3/2 resonance just above reaction threshold
 - ↔ ion temperatures < 10s of KeV ⇒ s-waves dominate
 - \leftrightarrow D (s=1) and T (s=\frac{1}{2}) preferentially fuse when spins are aligned

$$\boldsymbol{\sigma}_{cm} = \boldsymbol{\sigma}_0 \left\{ 1 + \frac{1}{2} \vec{P}_D^V \cdot \vec{P}_T \right\}$$

• polarized fuels ⇔ up to 50% enhancement in the cross section









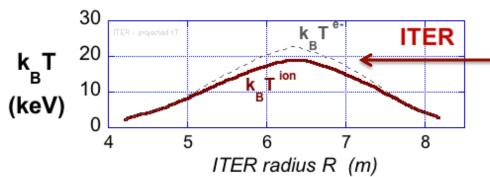
Reaction rates in a heated plasma

~ cross sections averaged over a Maxwellian velocity distribution

$$R = n_D n_T V_{pl} x \left\{ \left\langle \sigma v \right\rangle = \frac{4c}{\sqrt{2\pi M_r} (k_B T)^{3/2}} \int e^{-\varepsilon/k_B T} \varepsilon \sigma(\varepsilon) d\varepsilon \right\}$$

J.N. Bahcall, *Astrophys. J.* **143** (1966) 259

- D+T resonance peaks at 65 keV
- ITER plasma expected to peak at 18 keV
- most of the yield from low energies
- $<\sigma v>$ integral extends to higher energies but saturates by $\sim 50 \text{ keV}$



ITER Design Report, Plasma Physics and Controlled Fusion 44 (2002) 519



75

 $\sqrt{2\pi M_{\pi}} (k_B T)^{3/2}$

50

25

 $T + D \rightarrow \alpha + n$

 $^{-\varepsilon/k_BT}\varepsilon \sigma_o(\varepsilon) d\varepsilon$

100

(keV)

125



150

kT(ion) = 18 keV

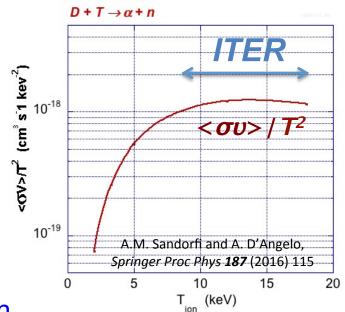
10⁻¹⁹

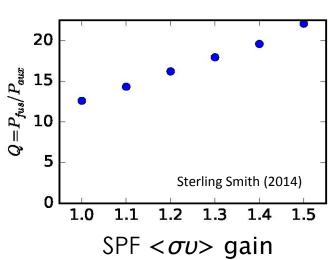
alpha heating \$\top non-linear enhancements from the resonance tails

Recast the fusion rate:

$$R = n_D n_T V \cdot \langle \sigma v \rangle = \frac{\beta^2}{4\mu_0^2} V \cdot \frac{\langle \sigma v \rangle}{T^2} \cdot B^4$$
plasma pressure ~ constant ~ constant for ITER

- new simulations for ITER show net 75% gain in power and Q = P(fusion)/P(in) with spin polarized fuel, from increased alpha heating
- ⇔ polarization equivalent to 15% change in B
- on the ITER scale, Q = 10 could be reached even with toroidal field degradation













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$$\boldsymbol{\sigma}_{cm} = \boldsymbol{\sigma}_0 \left\{ 1 + \frac{1}{2} \vec{P}_D^V \cdot \vec{P}_T \right\}$$

- polarized fuels ⇔ up to 50% enhancement in the cross section
 - ⇔ up to 75% enhancement in power and Q
 - ⇔ could compensate for 15% magnetic field degradation on the ITER scale, and maintain high Q
 - \Leftrightarrow costs savings of future fusion reactor plants ($\sim B^2$) as much as 30% \Leftrightarrow a potentially huge factor!









Polarization survival - history

• Potentially large benefits require fuel polarization to survive a 10^8 K plasma for the energy containment time \sim a few sec

• History

- Kulsrud, Furth, Valeo & Goldhaber, Phys Rev Lett 49 (82) 1248
- Lodder, Phys. Lett. A98 (83) 179
- Greenside, Budny and Post, J. Vac. Sci Technol. A 2(2), (84) 619
- Coppi, Cowley, Kulsrud, Detragiache & Pegoraro, Phys Fluids 29 (86) 4060
- Kulsrud, Valeo & Cowley, Nucl Fusion 26 (86) 1443
- Cowley, Kulsrud, Valeo, E.J. Phys. Fluids 29 (86) 1443

•••

• Depolarization mechanisms

- a great many mechanisms were investigated in the '80s; two survive scrutiny
- both hinge on wall recycling

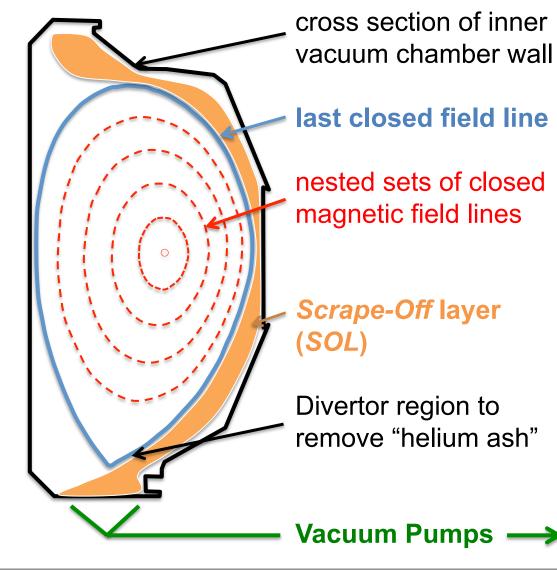








Wall Recycling through the scrape-off layer





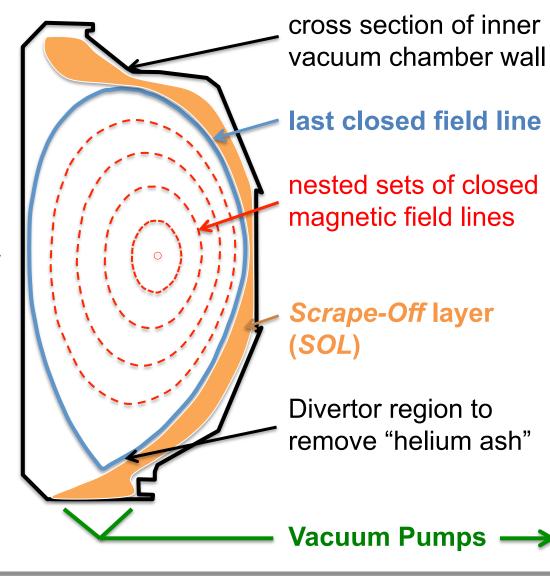




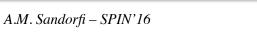


Wall Recycling through the scrape-off layer

- after injection, some few % of the fuel undergoes fusion; the rest escapes the plasma
- escaping ions strike outer walls and are neutralized
- depending on wall conditions, ions could depolarize
- if these reenter the plasma, they could dilute polarization of the core
- fuel leaving the plasma will eventually diffuse through the SOL and be pumped away







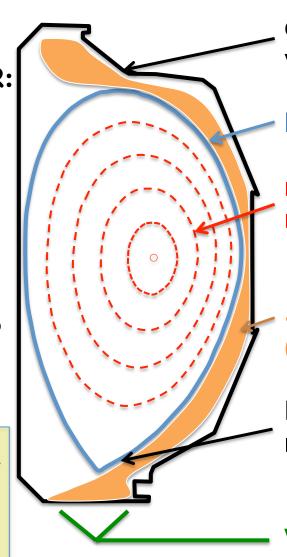


the ITER scrape-off layer

What's new?

- Plasma Simulations for ITER:
 - Pacher et al.,
 Nucl. Fus. 48 (2008) 105003
 - Garzotti et al.,
 Nucl. Fus. 52 (12) 013002
- ⇔ at ½ GW, the SOL is opaque to neutrals, which are swept to the divertor by convection

⇔ Wall recycling will be insignificant in ITER scale reactors



cross section of inner vacuum chamber wall

last closed field line

nested sets of closed magnetic field lines

Scrape-Off layer (SOL)

Divertor region to remove "helium ash"

Vacuum Pumps











Q: how to produce polarized fuel for a ~ GW reactor?

- every characteristic of polarized material comes at a cost; eg. NP techniques have emphasized lifetime (T_1) of 10^6 –to- 10^8 sec, (which is useless for fusion where ~ 10 sec would be more than adequate)
- ITER will require 2000 moles/day, much more than consumed in NP exps
- ⇔ significant R&D, tailored to fusion requirements, will be required
 - eg. 0th order speculation:
 - spin-exchange optical pumping (SEOP) of molecular DT gas with 2 lasers; alternate: separate SEOP of HD and HT (since H is benign in a plasma)
 - condense polarized gas to solid pellets for injection with *Pellet Injectors*, modified to maintain continuous magnetic holding fields
- Crucial to first verify expectation of polarization survival in plasma









next challenge: demonstrate polarization survival in a plasma

Focused efforts by two groups:

I. SPF (Spin-Polarized-Fusion) Collaboration:



- Jefferson Lab
- DIII-D National Fusion Facility:
 - General Atomics, (parallel VIII, Applications)
 - ORNL,
 - UC-Irvine
- University of Virginia (parallel VIII, Applications)
- University of Connecticut

II. PolFusion Collaboration:

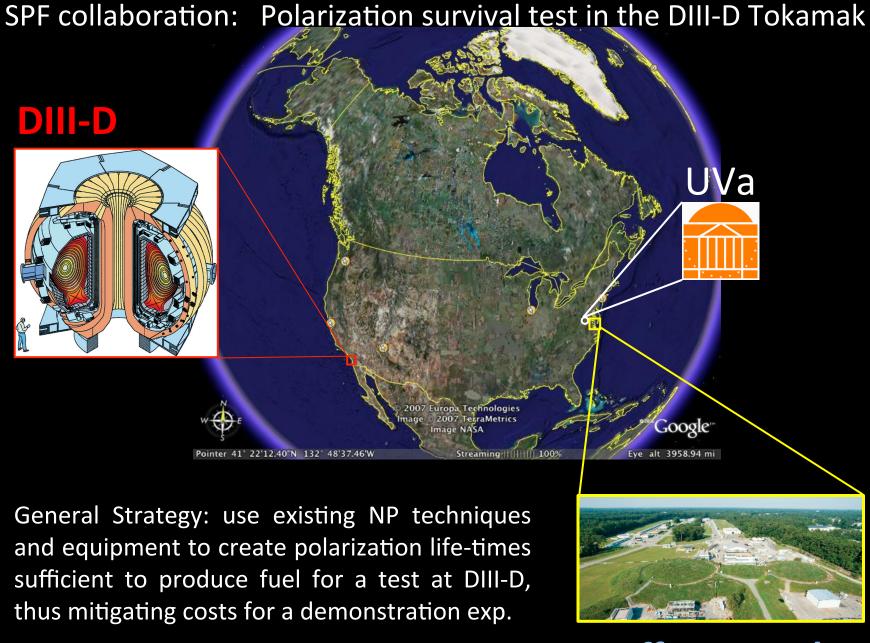
- Forschungszentrum Jülich
- University Düsseldorf
- TU Darmstadt
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- PNPI Gatchina
- Budker Institute Novosibirsk (parallel VIII, Applications)











Jefferson Lab

D + ³He as a test-bed for Spin-Polarized-Fusion

In nuclear reactions, isospin is a <u>very good</u> quantum number, particularly at low energies

- ⇔ ⁵He and ⁵Li are *mirror* nuclei with virtually identical low-energy structure
- \Leftrightarrow D+T \Rightarrow ⁵He \Rightarrow α +n and D+³He \Rightarrow ⁵Li \Rightarrow α +p are mirror reactions, with the same spins, incorporating the same nuclear physics
- \Leftrightarrow Polarization survival can be tested with D + 3 He $\rightarrow \alpha$ + p and lessons learned can be directly applied to D + T $\rightarrow \alpha$ + n









Strategy for testing polarization survival in DIII-D

- test reaction: $\vec{D} + {}^{3}\vec{H}e \rightarrow \alpha + p$ {mirror reaction to $D + t \rightarrow \alpha + n$ }
- use existing JLab facilities to create solid \overrightarrow{HD} ; ship to DIII-D
 - → diffuse 200-400 atm HD into ICF shells; cool to solid; polarize H and D; H ⇒ D spin transfer to maximize D spin; transport polarized pellets to DIII-D; load into 2 K cryo-gun



- use existing UVa facilities to develop polarized ${}^3ar{H}e$ gas-filled ICF shells
 - → diffuse ~20 atm polarized ${}^{3}\vec{H}e$ into ICF shells; cool to seal; move polarizer to DIII-D; fill shells; load into 77 K cryo-gun
- generate *H* plasma in the DIII-D Tokamak
 - → inject polarized fuel into plasma, alternating spin alignment:

parallel:
$$H\vec{D} \uparrow + {}^{3}\vec{H}e \uparrow \uparrow$$
 anti-parallel: $H\vec{D} \downarrow + {}^{3}\vec{H}e \uparrow \uparrow$ \Rightarrow compare proton yields









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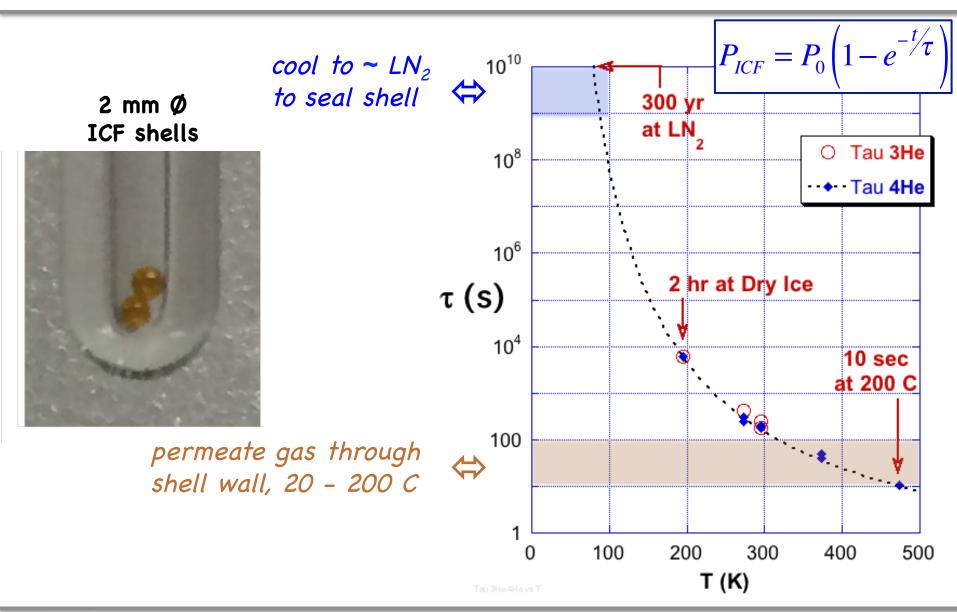








fuel delivery via Inertial-Confinement (ICF) polymer shells







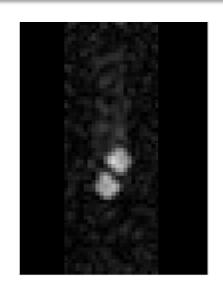




Imaging ICF pellets filled with polarized ${}^3ar{H}e$ at UVa









- 2 mm Ø ICF shells in a glass tube
- MRI scan of ICF shells filled with polarized ³He, cooled to 77K to seal, ³He outside removed

³He polarization inside ICF shells can be maintained for ~ 10 hr at 77 K

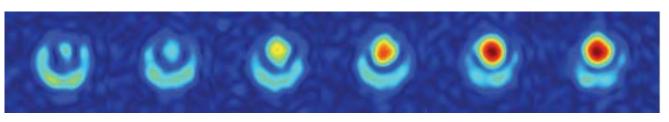








Options for ICF shells – polarization loss during permeation



Time sequenced MRI of polarized ³He filling a GDP pellet

ICF shell material	Known issues	Fractional loss of ³ He polarization during permeation
GDP (glow discharge polymer)	free radicals	~ 0.3 at 295K (*)
PAMS (Poly-alpha-methylstyrene)	no free radicals	tbd
Polystyrene	no free radicals	tbd

G.W. Miller, Parallel VIII - Applications

Likely options to reduce loss:

- Permeate at higher temperature where time spent in polymer wall is less
- Different materials, which have no free radicals









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 \Rightarrow compare proton yields anti-parallel: $H\vec{D} \downarrow + \vec{H}e \uparrow \uparrow$

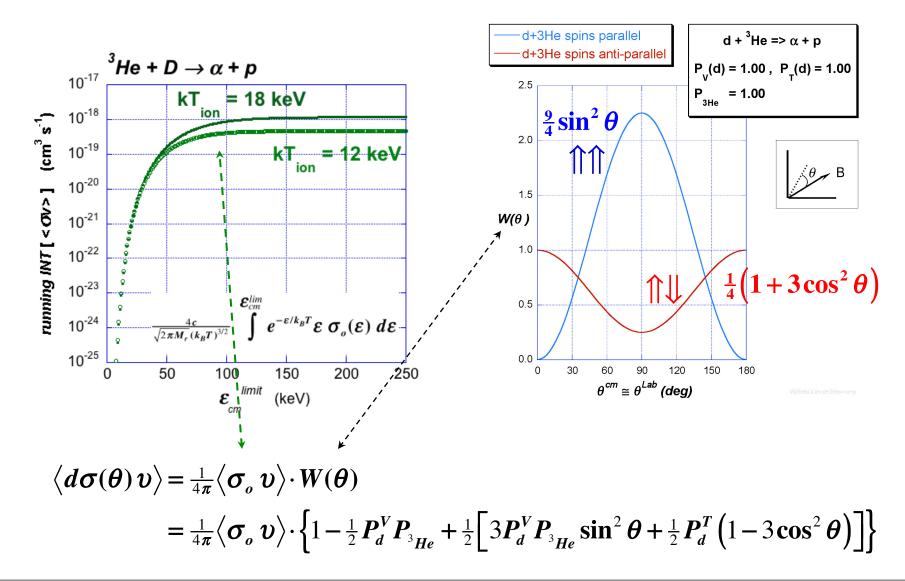








$D + {}^{3}He \rightarrow \alpha + p$ distributions wrt torus field



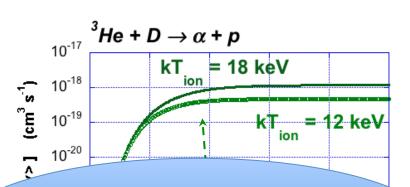






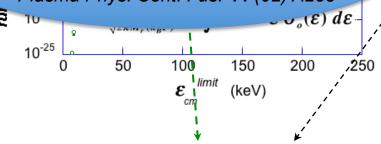


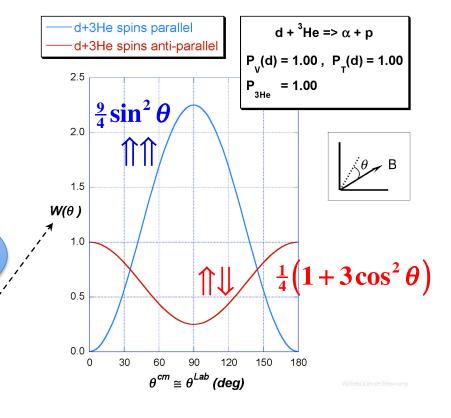
$D + {}^{3}He \rightarrow \alpha + p$ distributions wrt torus field



Q H-Mode DIII-D can reach ITER temperatures of 12-18 keV

Plasma Phys. Cont. Fus. 44 (02) A253





$$\langle d\sigma(\theta) v \rangle = \frac{1}{4\pi} \langle \sigma_o v \rangle \cdot W(\theta)$$

$$= \frac{1}{4\pi} \langle \boldsymbol{\sigma}_{o} \boldsymbol{v} \rangle \cdot \left\{ 1 - \frac{1}{2} \boldsymbol{P}_{d}^{V} \boldsymbol{P}_{_{^{3}\boldsymbol{He}}} + \frac{1}{2} \left[3 \boldsymbol{P}_{d}^{V} \boldsymbol{P}_{_{^{3}\boldsymbol{He}}} \sin^{2} \boldsymbol{\theta} + \frac{1}{2} \boldsymbol{P}_{d}^{T} \left(1 - 3 \cos^{2} \boldsymbol{\theta} \right) \right] \right\}$$









expected d+ 3 He $\rightarrow \alpha$ +p signal with existing NP material

Jlab:
$$P_V(\vec{D}) = 0.40$$
 UVa: $P({}^3\vec{H}e) = 0.65$

$$\langle \sigma^{par} V \rangle = \langle \sigma_o V \rangle \{1 + \frac{1}{2}(0.26)\}$$
$$\langle \sigma^{anti} V \rangle = \langle \sigma_o V \rangle \{1 - \frac{1}{2}(0.26)\}$$

UVa:
$$P(^{\circ}He) = 0.65$$

Signal from comparing shots
$$\Leftrightarrow$$
 $\Rightarrow \frac{\langle \sigma^{par} V \rangle}{\langle \sigma^{anti} V \rangle} = 1.30$









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"signal" comes from comparing different plasma shots \(\Rightarrow \text{requires good shot reproducibility} \)

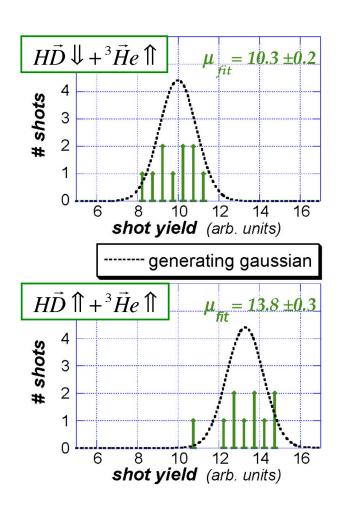






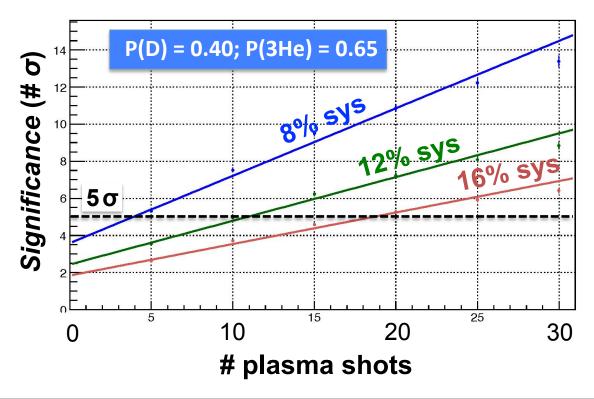


Systematic variations btw plasma shots determines # shots needed for a definitive experiment



How many shots in each spin alignment to reach 5 σ confidence ⇔ Monte Carlo

- 8% plasma variation ⇔ 4 shots
- 16% plasma variation ⇔ 18 shots



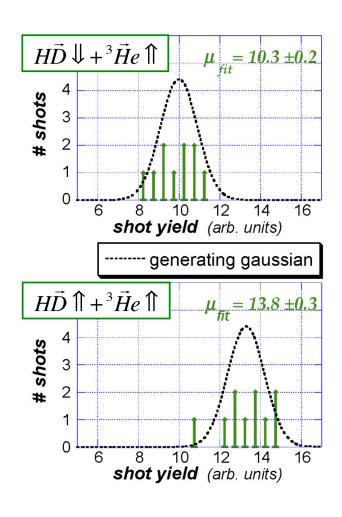






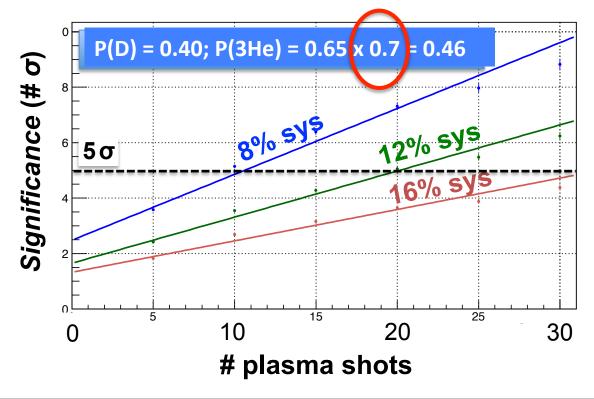


Systematic variations btw plasma shots determines # shots needed for a definitive experiment



How many shots in each spin alignment to reach 50 confidence ⇔ Monte Carlo

- 8% plasma variation ⇔ 10 shots
- 16% plasma variation ⇔ 30 shots







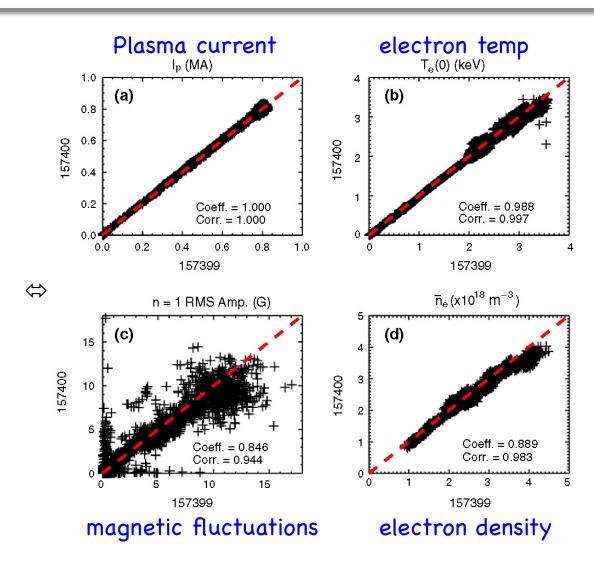




DIII-D shot reproducibility

DIII-D plasma shots:

- 3 s ramp up to 2.1 tesla
- 10 s flat top,
 with 80 keV neutral-beam
 heating
- 7 s ramp down
- 15 min btw shots
- parameters of repeated shots are high correlated, to ~ 10%
- time requested to study reproducibility of highperformance Quiescent Hmode for polarized fusion



Pace, Lanctot, Jackson, Sandorfi, Smith, Wei, J. Fus. Energy 35 (2016) 54



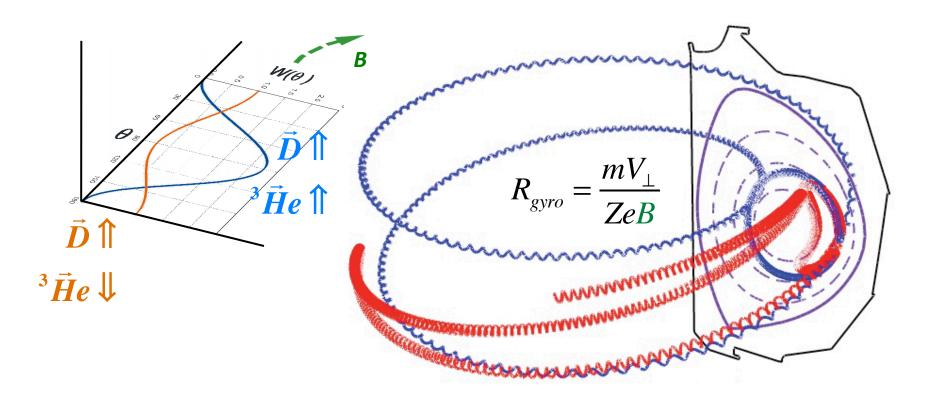






Tracking fusion products in DIII-D: Spin Alignment and Orbit Losses

- parallel spins ightharpoonup large $V_{\scriptscriptstyle \parallel}$ ightharpoonup large gyroradii ightharpoonup protons hit the wall in a few orbits
- anti-parallel spins ightharpoonup large $V_{_{||}}$ ightharpoonup small gyroradii ightharpoonup better confined



 α and ρ loss-locations on Tokamak wall depend on initial polarizations





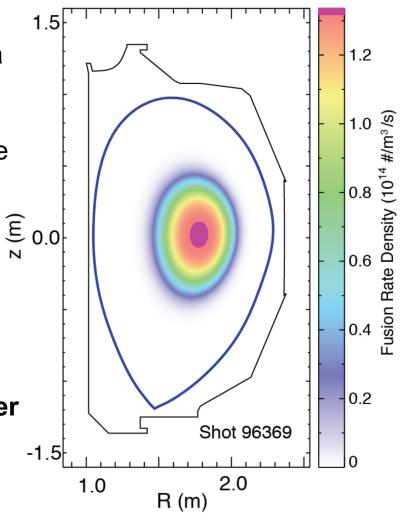




Tracking D + 3 He $\rightarrow \alpha$ + p products in DIII-D

Tracking Simulations:

- fusion rate density taken from data with a solid D₂ pellet (shot 96369)
- cross sections scaled from D+D to D+3He
- T_{ion} energy scaled to 15 keV (as expected for Quiescent H-Mode)
- fusion profile discretized; $\alpha + p$ generated along different polar (pitch) θ and azimuthal (gyrophase) ϕ , relative to the local field, weighting the relative number by the polarized angular distributions
- particles are tracked until striking a wall



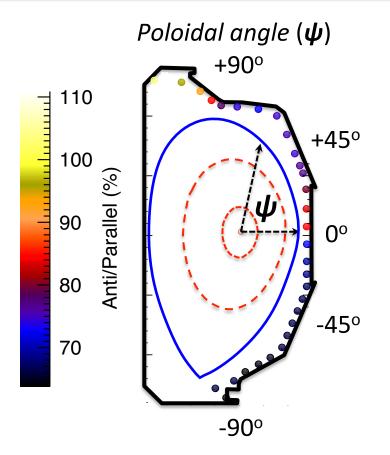




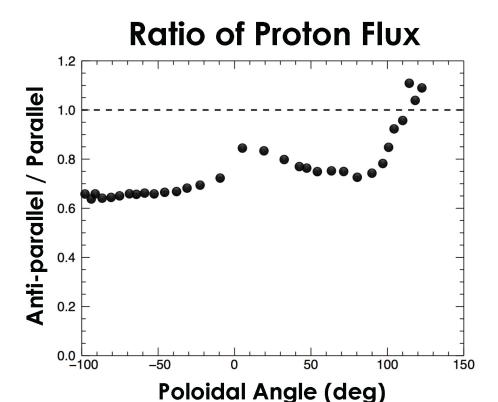




Predicted ratio of protons from anti-parallel & parallel spins



$$P(^{3}He) = 65\%$$
; $P(D) = 40\%$



Definitive Signal:

- ~30% change expected at several wall locations
- distinctive dependence on poloidal angle (ψ)









SPF (Spin-Polarized-Fusion) Collaboration

Jefferson Lab

 $H\vec{D}$ & pellet filling

A. Deur, C. Hanretty, M. Lowry, A.M. Sandorfi, X. Wei

University of Virginia

 $^{3}\vec{H}e$

J. Liu, G.W. Miller, X. Zheng

University of Connecticut

 $^{3}\vec{H}e$ & pellet filling

K. Wei

General Atomics/Fusion Energy Research

GA-DIII-D:

plasma, orbit & transport simulation

N. Eidietis, A. Hyatt, G. Jackson, M. Lanctot, D. Pace, S. Smith, H. St-John

GA-ICF Pellet Division:

ICF pellets for HD, ³He

M. Farrell, M. Hoppe, M. Schoff, N. Alexander

Oak Ridge National Lab

cryo-injection guns

L.R. Baylor

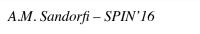
UC-Irvine

fast particle detection

W.W. Heidbrink









The next steps

- complete study of ICF shell materials to maximize ³He polarization
 UVa, JLab, U Conn
- study of variations in high-T_{ion} Quiescent H-mode plasma shots
 DIII-D/GA, JLab
- preliminary design of polarized pellet injectors (to define costs)
 - ORNL, JLab









next challenge: demonstrate polarization survival in a plasma

Focused efforts by two groups:

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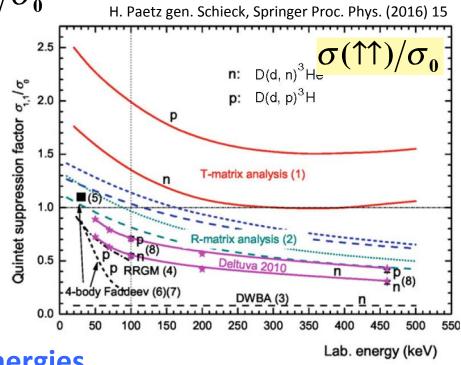






Direct measurements of low-energy D+D asymmetries

- could $\vec{D} + \vec{D} \implies {}^{4}He \rightarrow {}^{3}He + n$ or T + p test pol survival?
- to 1st order, two parallel spin 1 deuterons cannot form spin 0 ⁴He
 but, this is complicated by NP effects (deuteron D-state, ...)
- NP theory predictions for $\sigma(\uparrow\uparrow)/\sigma_0$ range from, suppression by 10 $_2$ to enhancement by 2.5
- 1st direct measurement with polarized beams and targets in preparation at PNPI-Gatchina (St Petersburg, RU)
 - ⇔ potentially high impact if they can reach very low energies











extras

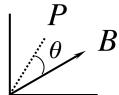








spin-dependent 3 He+D $\rightarrow \alpha$ +p (or T+D $\rightarrow \alpha$ +n) angular distributions



- polar (pitch) angles relative to local magnetic field direction
 - neglecting interference terms (good to ~ 2-3 %)

$$\frac{d\sigma}{d\Omega_{om}} = \left(\frac{d\sigma}{d\Omega}\right)_0 \left\{1 - \frac{1}{2}P_D^V P_{3_{He}} + \frac{1}{2}\left[3P_D^V P_{3_{He}}\sin^2\theta + \frac{1}{2}P_D^T \left(1 - 3\cos^2\theta\right)\right]\right\}$$

•
$$P_D^V = \mathbf{n}_D^{+1} - \mathbf{n}_D^{-1} \in [-1, +1]$$

•
$$P_D^T = n_D^{+1} + n_D^{-1} - 2n_D^0 \in [-2, +1]$$

•
$$P_{_{^{3}He}} = n_{He}^{+\frac{1}{2}} - n_{He}^{-\frac{1}{2}} \in [-1, +1]$$

→ angle integrated cross section :

$$\boldsymbol{\sigma}_{cm} = \boldsymbol{\sigma}_0 \left\{ 1 + \frac{1}{2} \vec{P}_D^V \cdot \vec{P}_{^3He} \right\}$$









Polarization dilution from HFS in partially ionized states at injection

- g.s. of all fuels (DT, HD, 3 He, ...) have 2 electrons paired to 1s \Leftrightarrow no nuclear int.
- after injection, a partially-ionized state with 1 electron will exist for ~ 10 ms,
 during which there will be level mixing and a degree of dilution of nuclear pol
- net fractional polarization loss: $\frac{\Delta P}{P} = \frac{1}{2} \left[1 + \frac{4(\mu_e \mu_I)^2 B^2}{A_{HFS}^2} \right]^{-1}$
 - \Leftrightarrow ³He has the largest hyperfine splitting, A_{HFS} = -8.66565 GHz
 - ⇔ mean ΔP/P for ³He, averaged over the DIII-D plasma field region and weighted by particle density = 1 %
 - ⇔ HFS ~ 1/B² ⇔ irrelevant in ITER, due to higher magnetic fields







Secondary Reactions

- use H-plasma heated with H neutral beams
- simulations follow secondary reactions to estimate background yields:

$$\rightarrow$$
 D + T \Rightarrow α + n (Q = +17.6 MeV)

- 15 MeV protons from 3 He + D \Rightarrow α + p provide a unique signature that is easily separated
- 2-step (D + D \Rightarrow ³He) + D wrt primary ³He + D is suppressed by $n(D) / [n(D)^2 \times n(D)]$, which is negligible







