

AIDA 2020

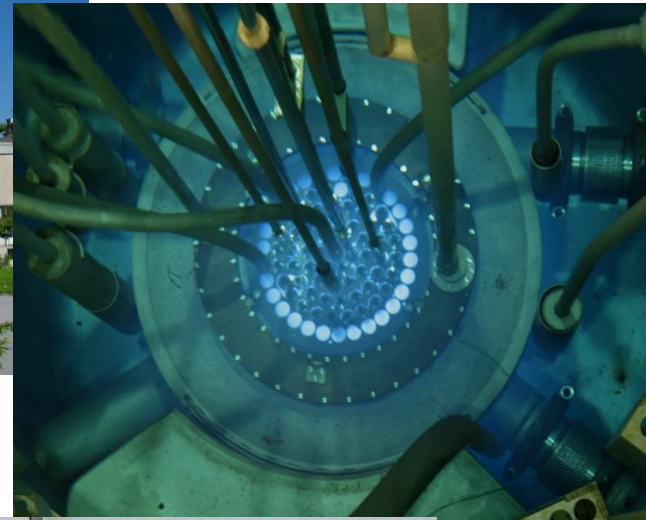
**Transport System for
Large objects at
Ljubljana JSI TRIGA Reactor
(part of WP15.5)**

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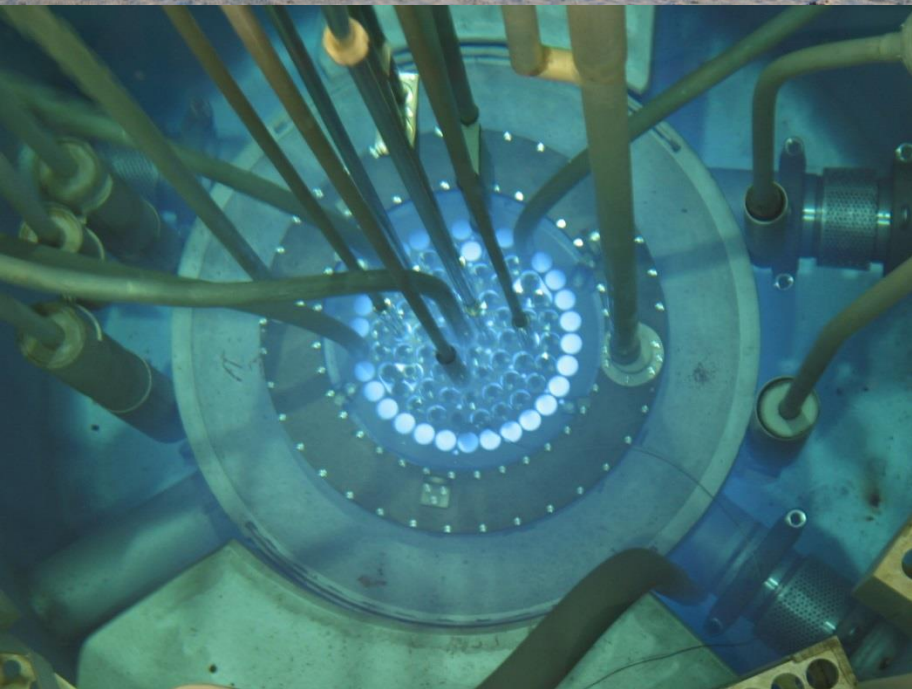
AIDA2020 2nd Annual Meeting, Paris, April 4, 2017

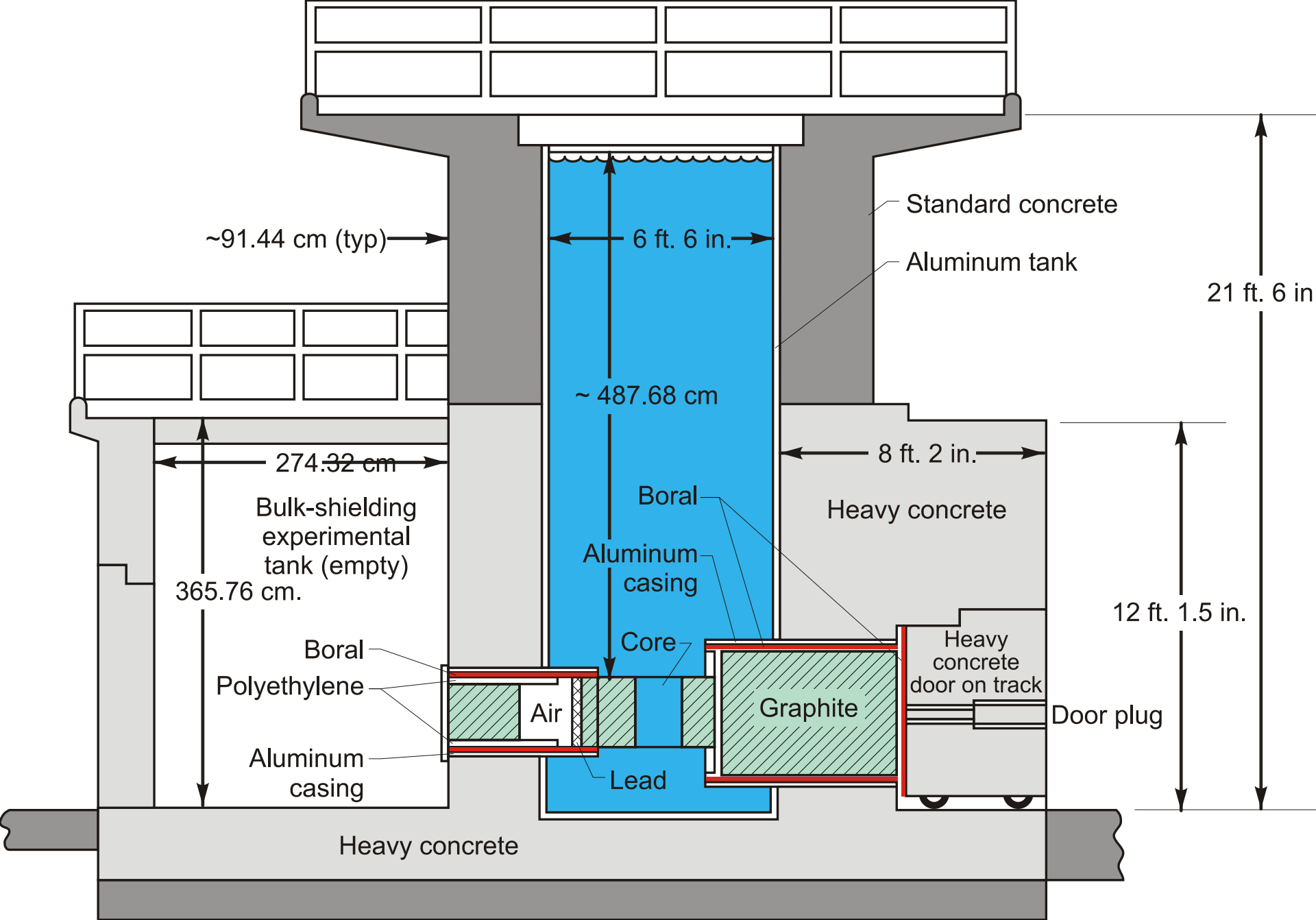
TRIGA Mark II Reactor

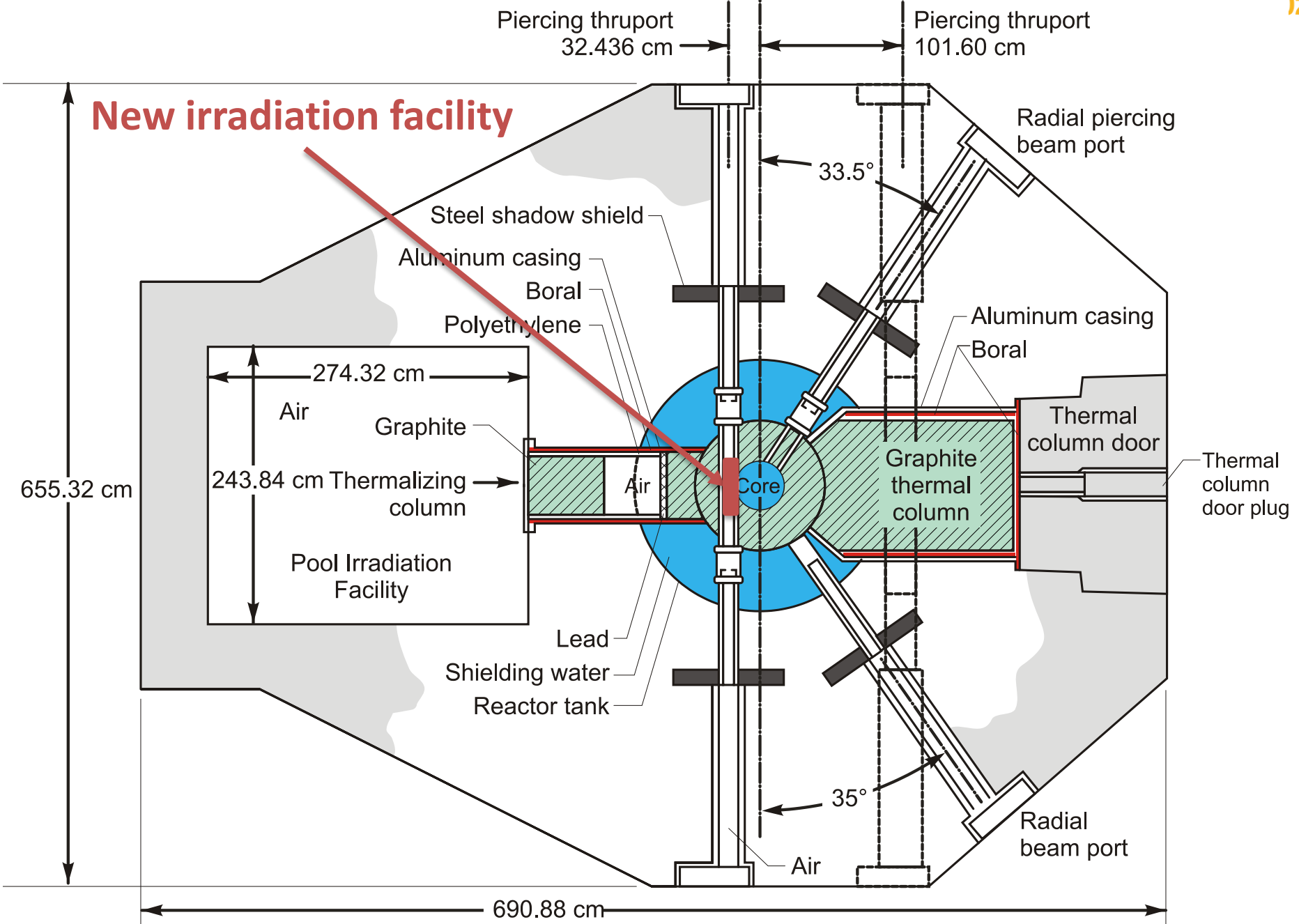


- 1st criticality:
 - 31st May, 1966
- P_{\max}
 - 250 kW (steady state)
 - 1 GW (pulse)
- Fuel
 - UZrH (12 wt. % U)
 - enrichment = 20 %

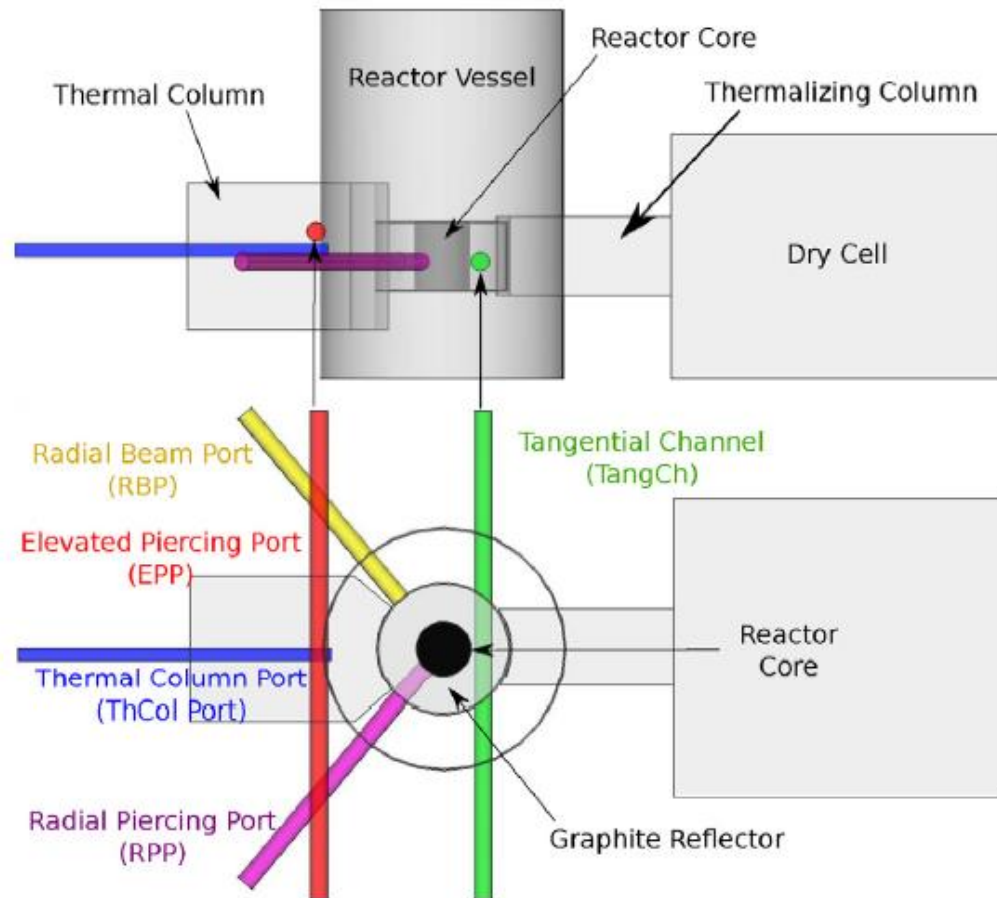








TRIGA Irradiation Channels



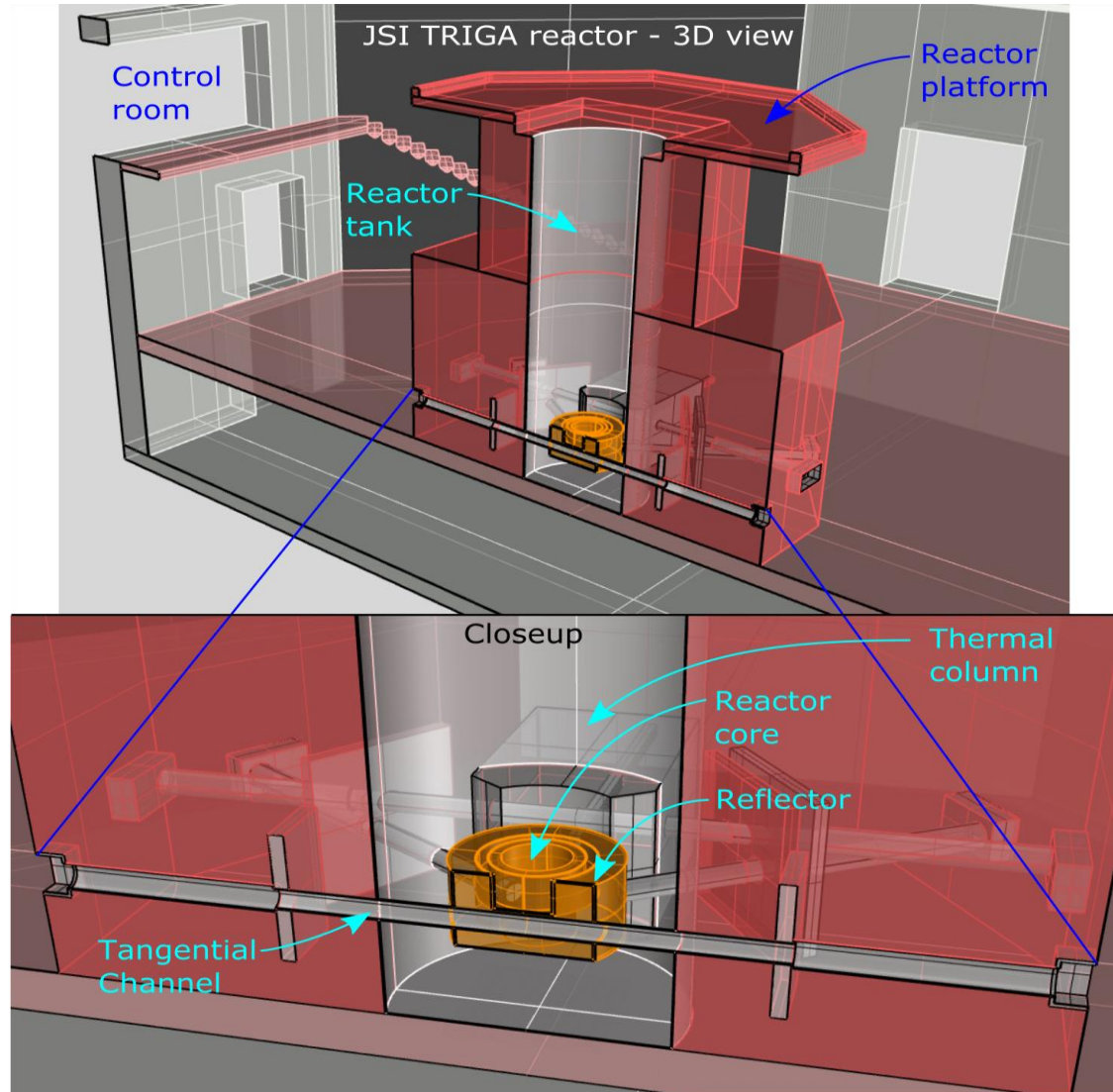
Goal

- Develop and install a large sample irradiation ($2R < 15$ cm) facility in the tangential channel of the JSI TRIGA reactor

Transport system for neutron irradiation of large objects at JSI:

- In the framework of the AIDA-2020 project, Work Package 15 - Upgrade of beam and irradiation test infrastructure, a new irradiation device / transport system which will enable the irradiation of larger samples, up to 12 cm in diameter, has been installed in the Tangential Channel of the JSI TRIGA reactor in Ljubljana, Slovenia. This report documents the design and installation of the irradiation device / transport system.*

3-d Modelling

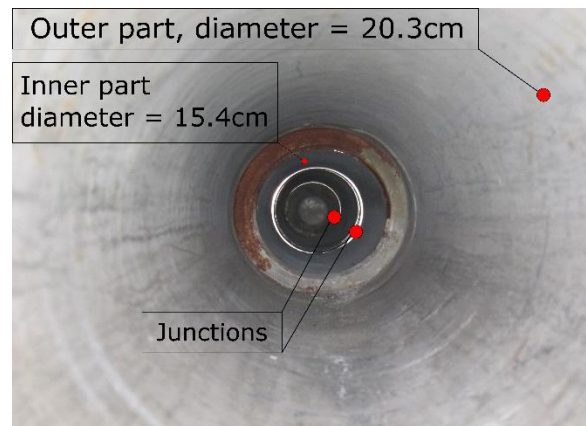
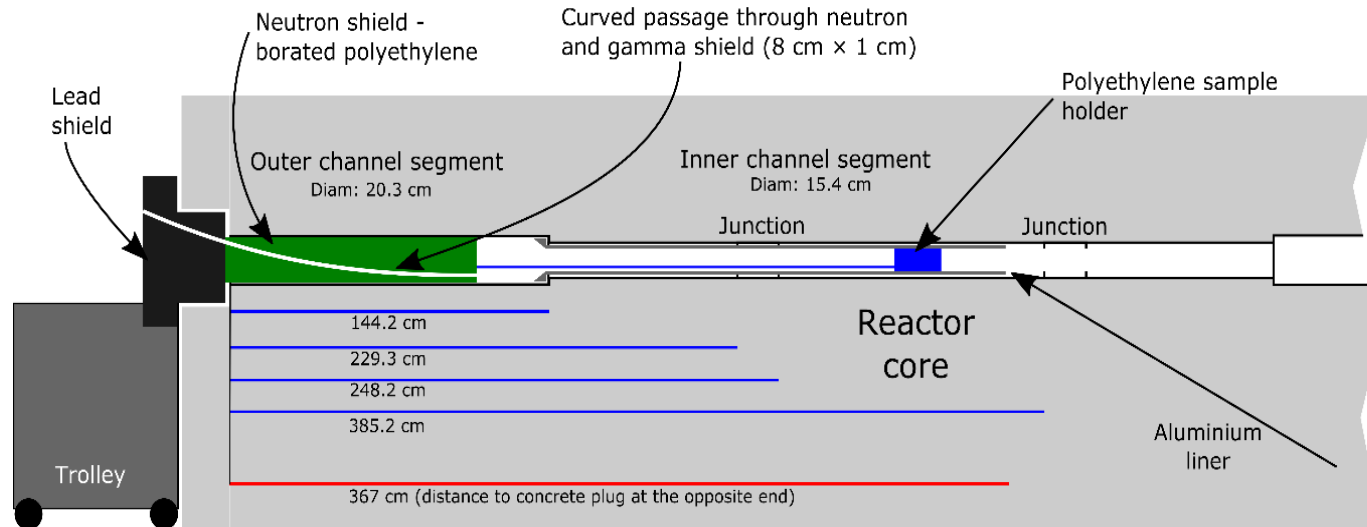


Chronology

- Extraction of cold neutron source used in the past for neutron diffractometry
- Decontamination and inspection of channel interior
- Documentation required to obtain the authorization for the installation of the irradiation device from TRIGA Reactor Safety Committee and Slovenian Nuclear Safety Administration
- Authorization granted in August 2016
- Channel completed in October 2016

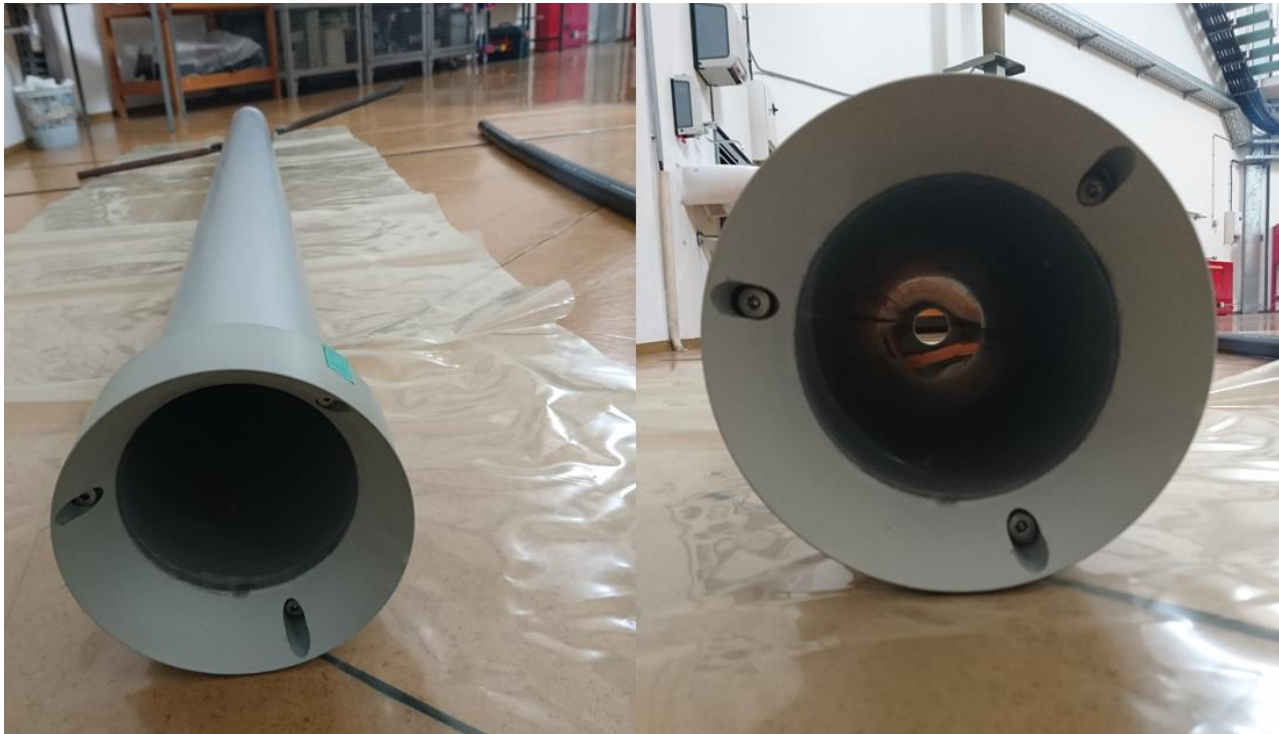
Detailed View

TRIGA reactor Tangential channel fitted with irradiation device



Insertion Tube

- Aluminium liner with 14.6 cm inside diameter
- Protection of internal components
- Facilitates insertion and withdrawal of samples



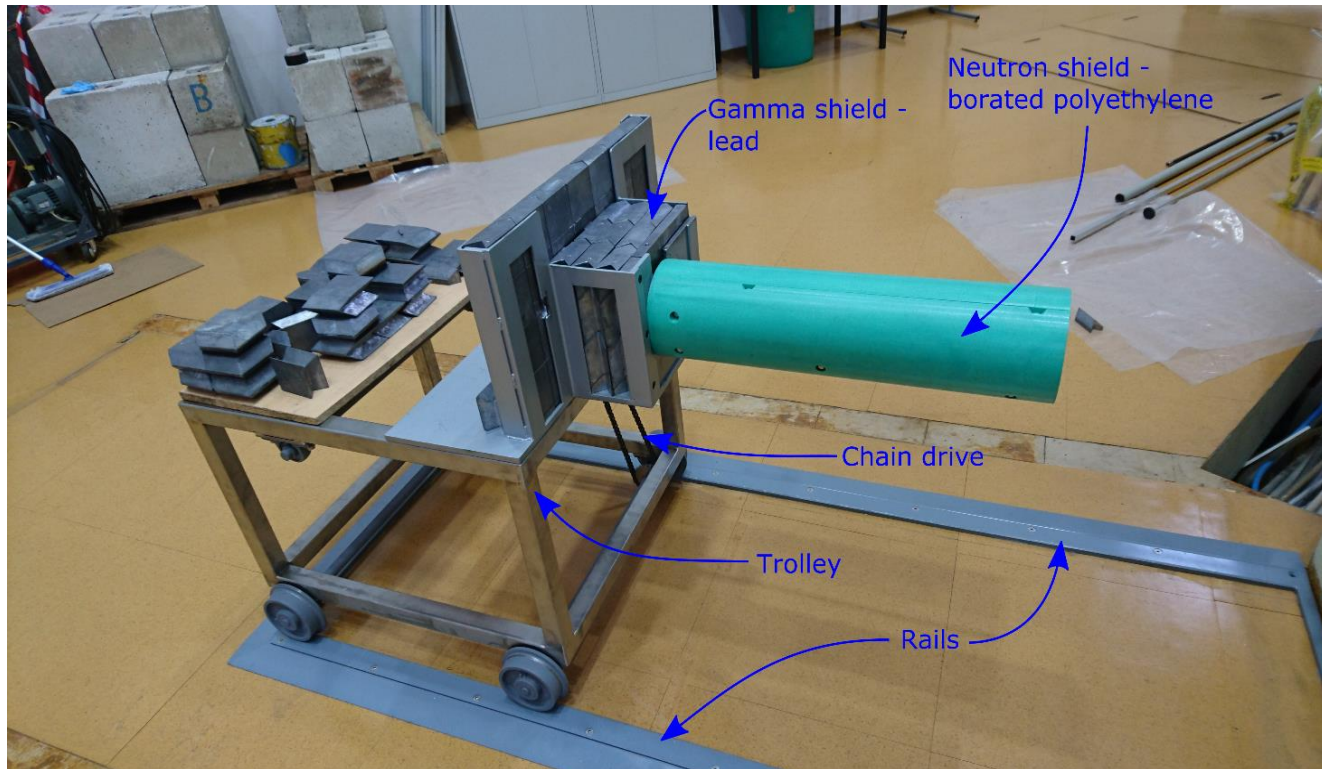
Sample Support Structure

- Sample support structure made from PE100
- Support jig for sample should be custom made !
- Allows routing of cables to the sample

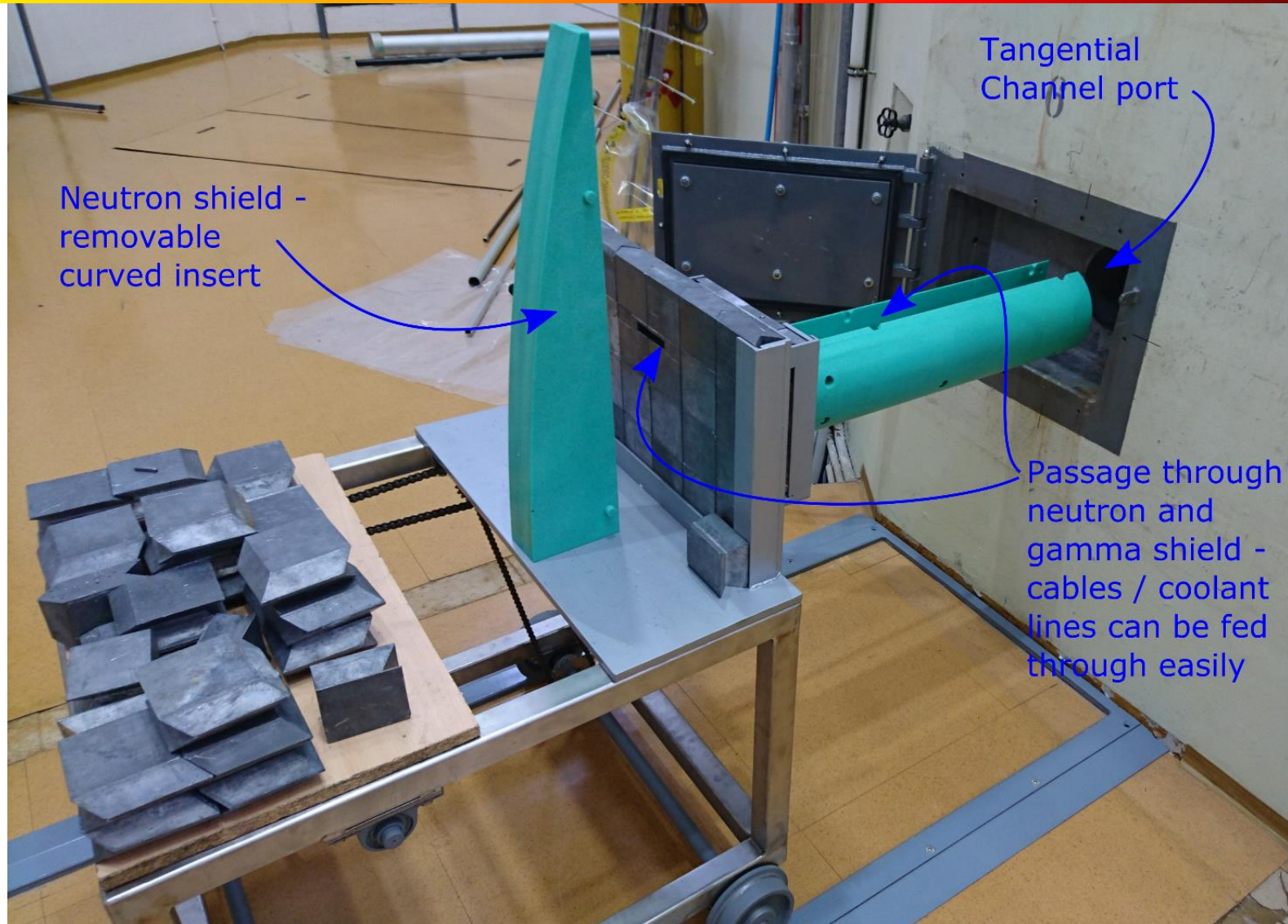


Shielding Plug

- Neutron shield made from borated polyethylene
- Gamma dose rate at surface of Pb \approx mSv/h
- Concrete bricks



Plug Insertion

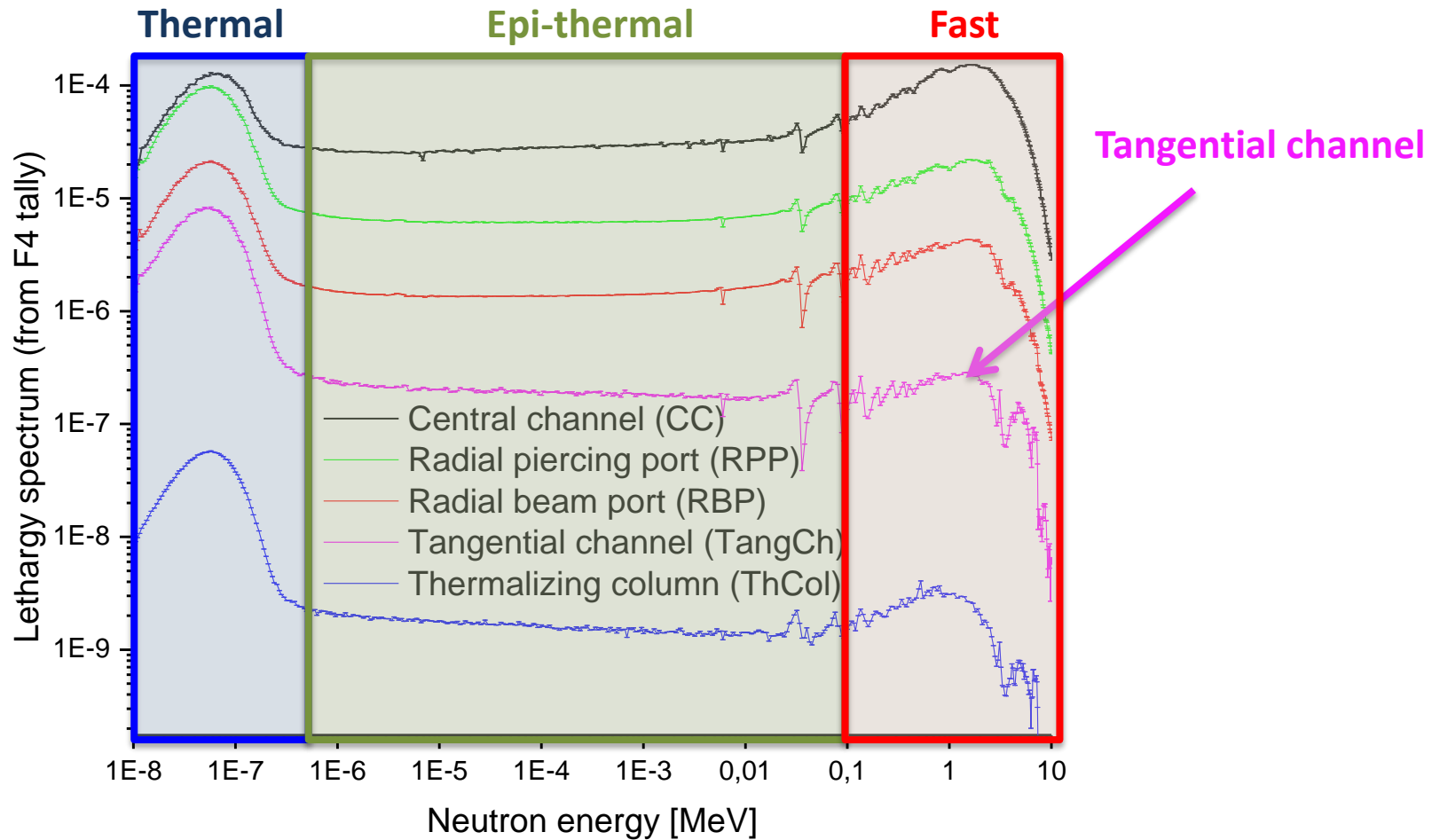


Channel Closed



- Inner diameter: **14 cm**
- Neutron flux characterisation:
 - L. Snoj et al., Appl. Rad. Isot. 70 (2012) 483–488
- Predicted neutron flux: $1.3e12$ n/cm²s
 - Thermal ($E < 0.625$ eV): 58 %
 - Epithermal (0.625 eV $< E < 100$ keV): 25 %
 - Fast ($E > 100$ keV): 17 % -> $2.3e11$ n/cm²s
 - **$1e15$ n_{eq}/cm² in 1 ½ hours**

Neutron Spectra

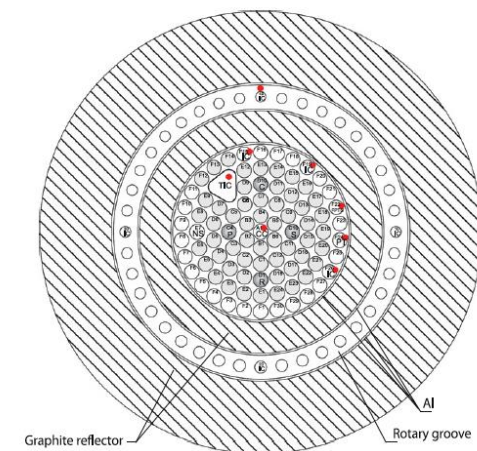


Predicted Fluxes

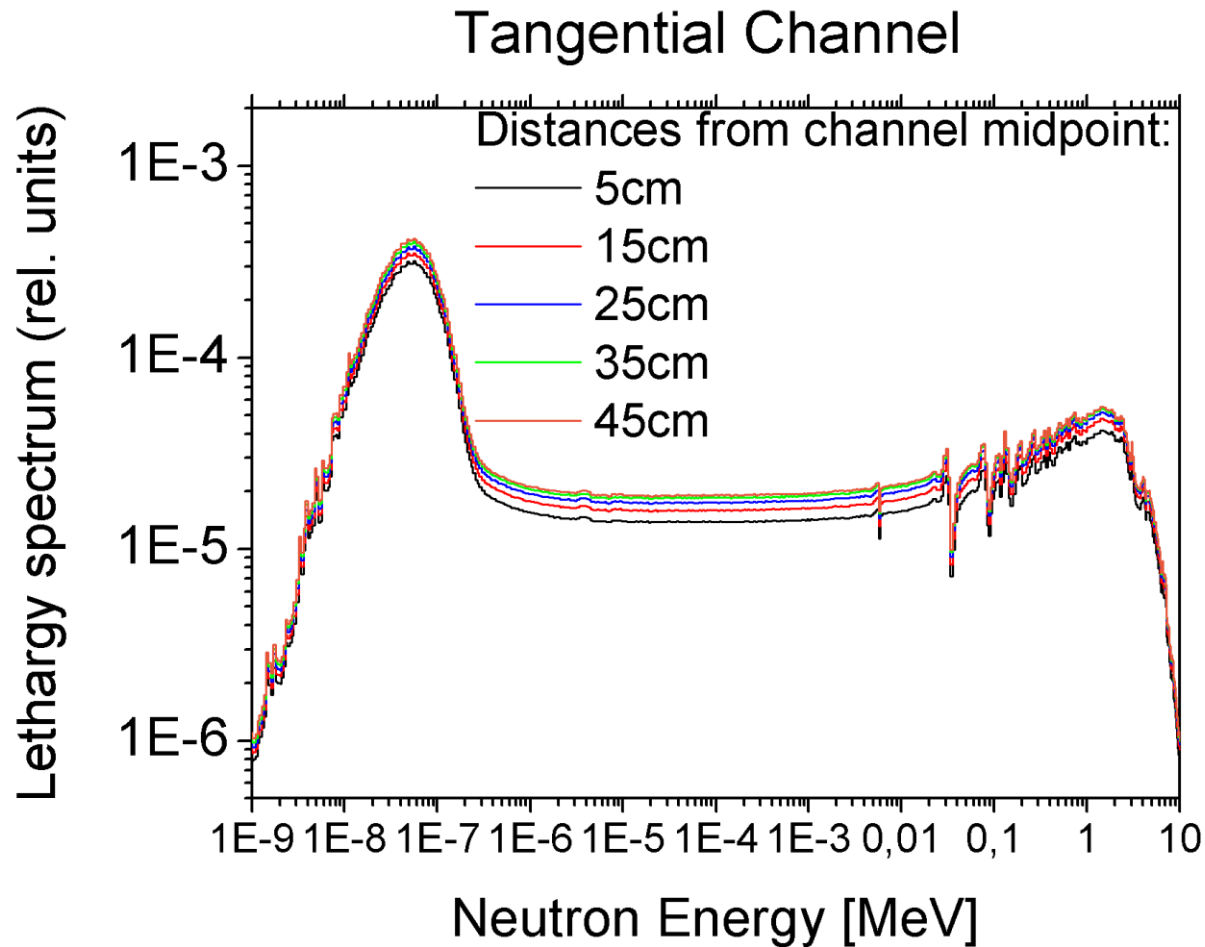
- Thermal neutrons <0.625 eV
- Epithermal neutrons 0.625 eV – 0.1 MeV ← Core simulation
- Fast neutrons > 0.1 MeV

	$\Phi_{th}(10^{12} \text{ cm}^{-2} \text{ s}^{-1})$	$\Phi_{epi}(10^{12} \text{ cm}^{-2} \text{ s}^{-1})$	$\Phi_{fast}(10^{12} \text{ cm}^{-2} \text{ s}^{-1})$	$\Phi_{tot}(10^{12} \text{ cm}^{-2} \text{ s}^{-1})$
small channel (F19)	3.66	1.86	1.81	7.32
large channel (TIC)	4.46	3.45	3.85	11.7
Tang. Ch.	0.75	0.33	0.23	1.31

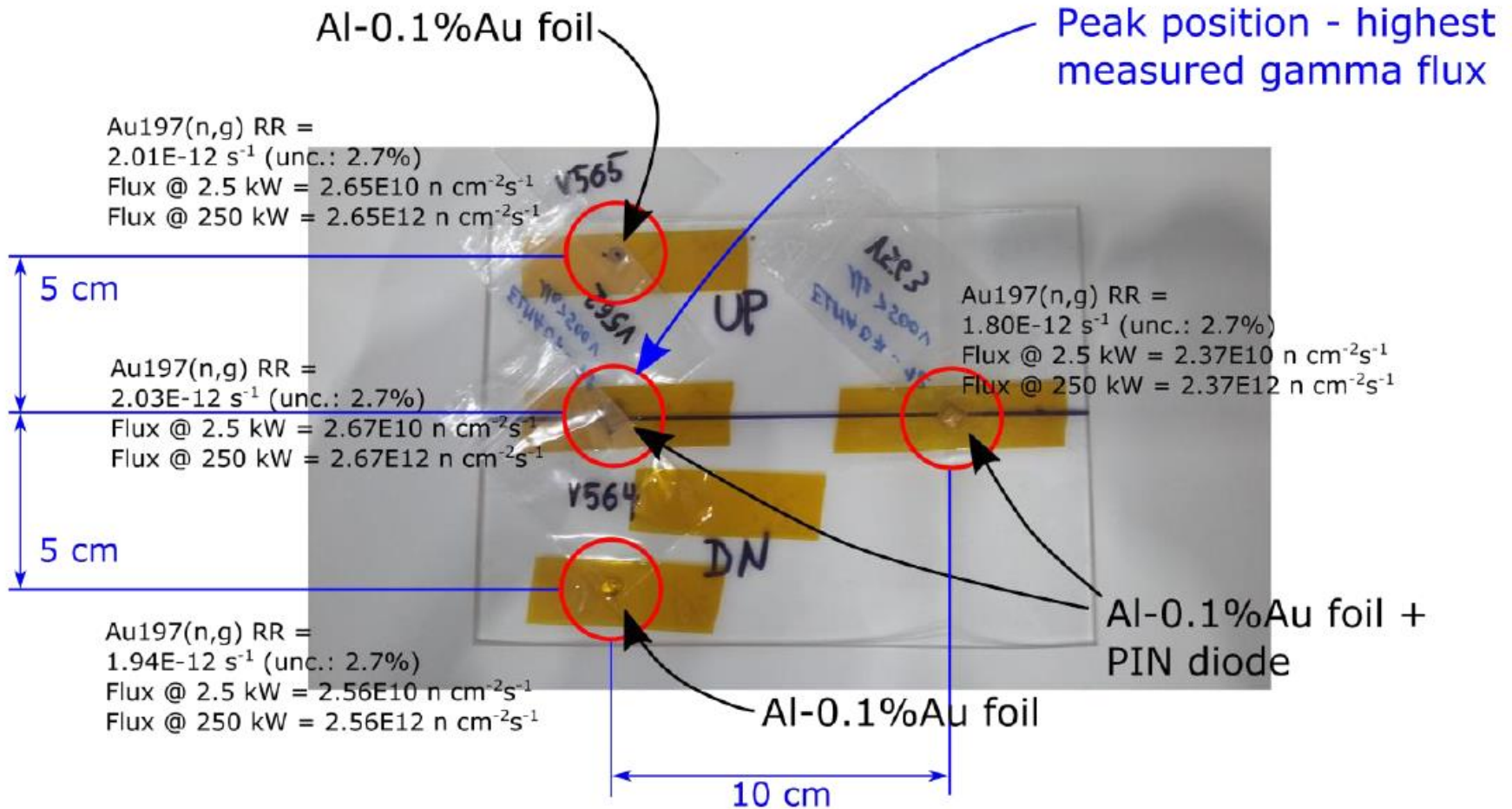
Agreement with measurements in small channel within 10 %



Flux Along Channel



Neutron Flux Measurement

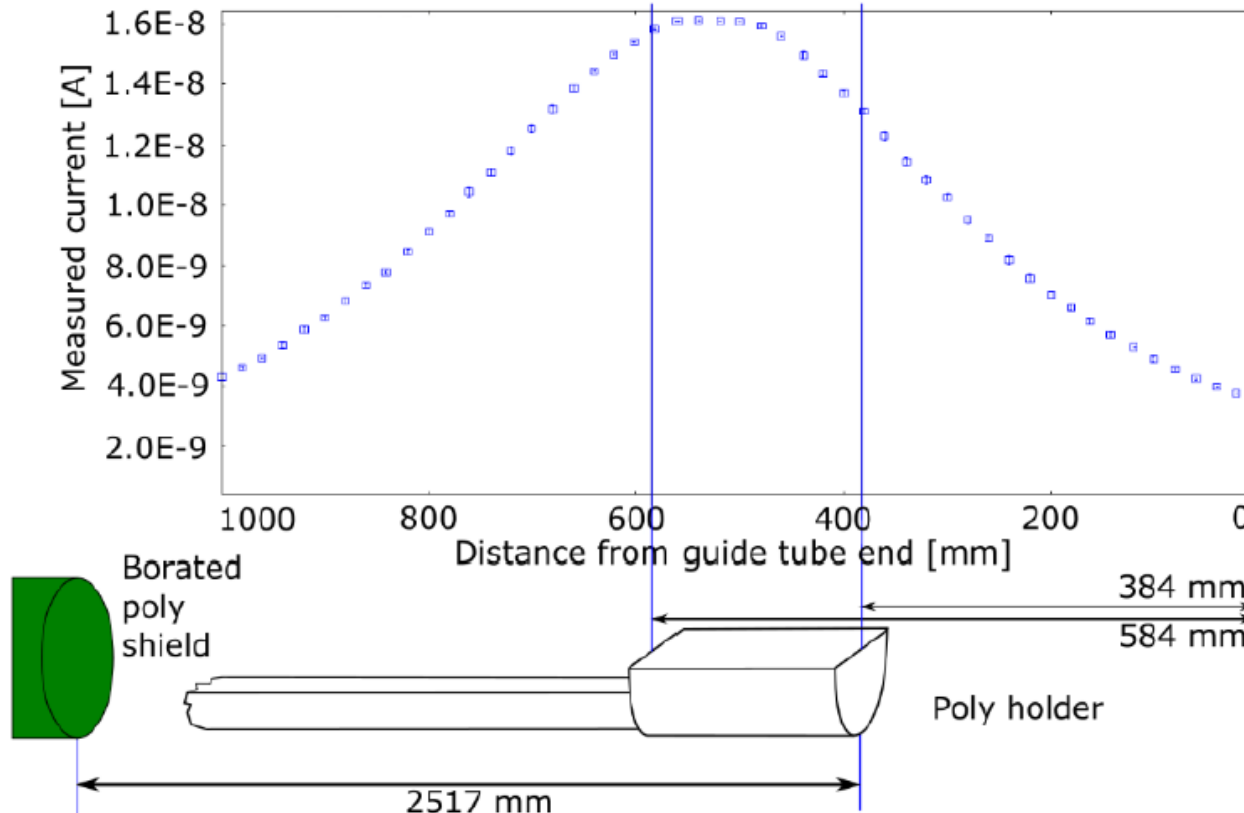


Neutron Flux

- Flux measured by Au 197(n, γ)
 - measures (mostly) thermal flux
 - scaled by simulated spectrum to total flux
- Au - measured total flux $2.67e12$ n/cm²s
 - uniformity < 10 % on 10 cm x 10 cm
- PIN - measured NIEL flux $3.9e11$ n/cm²s
 - NIEL hardness factor for total spectrum 0.146
 - hardness factor for $E_n > 0.1$ MeV: 0.83
- Twice the predicted value !

Gamma Flux

Gamma flux profile measurement in Tangential Channel, 3mm IC @ 25 kW



- Measured gamma flux profile
- Dose rate several 10 kGy/h (very preliminary!!) –
- Resulting in several kGy for $10^{14} n_{eq} cm^{-2}$

Summary

- New irradiation facility at JSI reactor installed and being commissioned
 - characterization in progress
 - first measurements indicate 2x predicted flux
- Allows irradiation of ~12 x 20 cm samples
 - services possible
- $10^{15} n_{eq} cm^{-2}$ in less than one hour
- Ready to receive samples!