

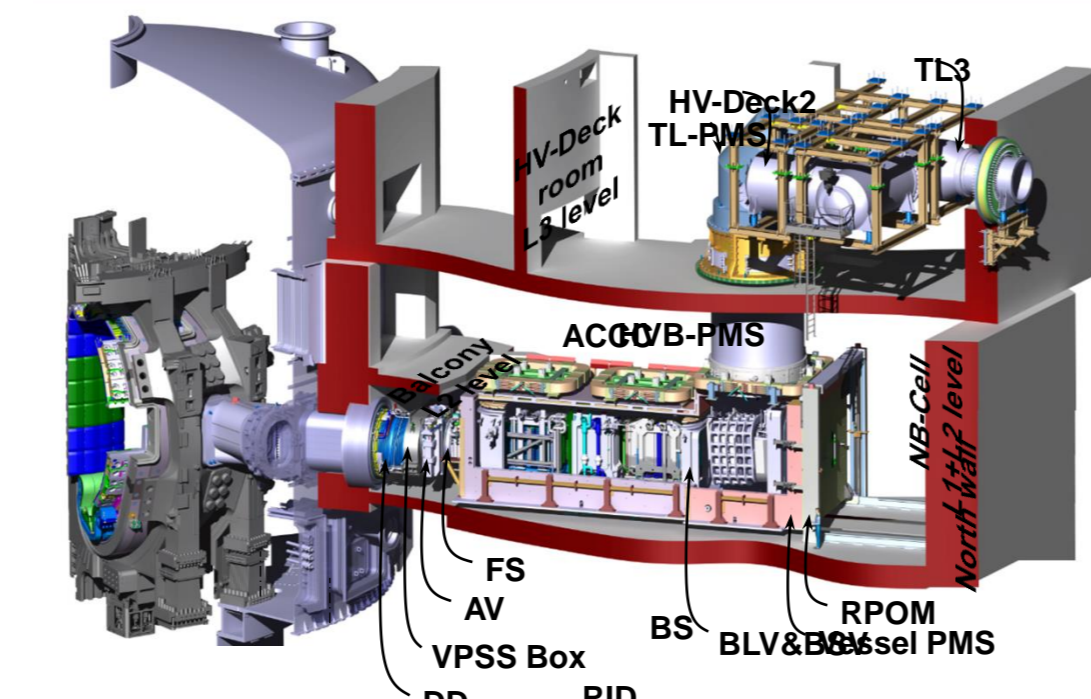
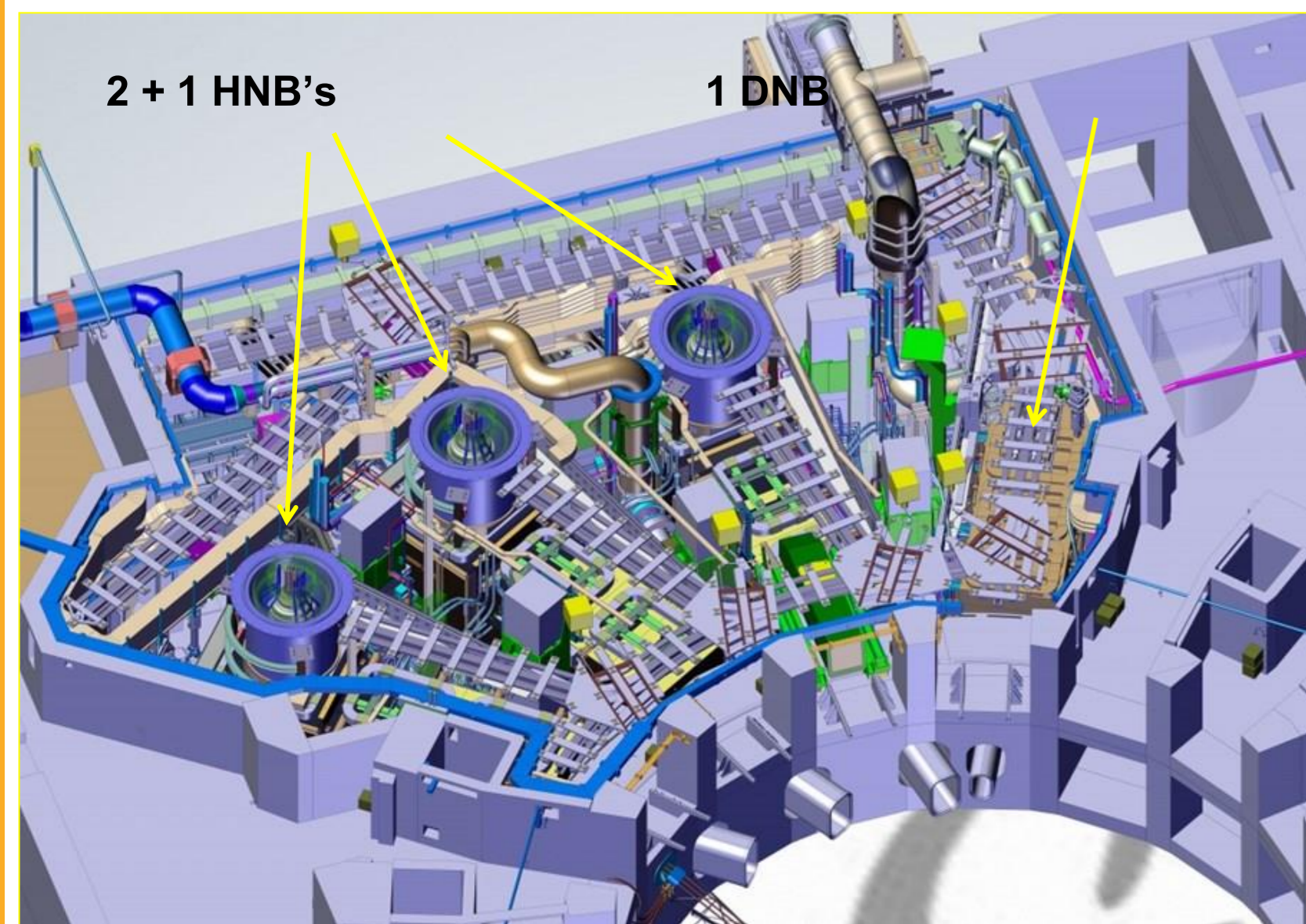
Update on the Negative Ion Based Neutral Beam Injectors for ITER

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1. Introduction

NB injectors at ITER – NB Cell



$D^0 \Rightarrow 1 \text{ MeV } 3600 \text{ s}$
 $H^0 \Rightarrow 870 \text{ keV } 1300 \text{ s}$
16.5 MW per HNB injector

$H^0 \Rightarrow 100 \text{ keV } 3600 \text{ s}$
2.4 MW DNB

The ITER baseline foresees 2 HNB's operating at 1 MeV 40 A D^0 , each capable of delivering 16.5 MW of deuterium ions to the plasma, with a 3rd HNB injector foreseen as an upgrade option that would bring up the total neutral beam power to 50MW [1]. In addition a dedicated DNB will be injecting 100 keV 60 A of H^0 for charge exchange recombination spectroscopy (CXRS) [2].

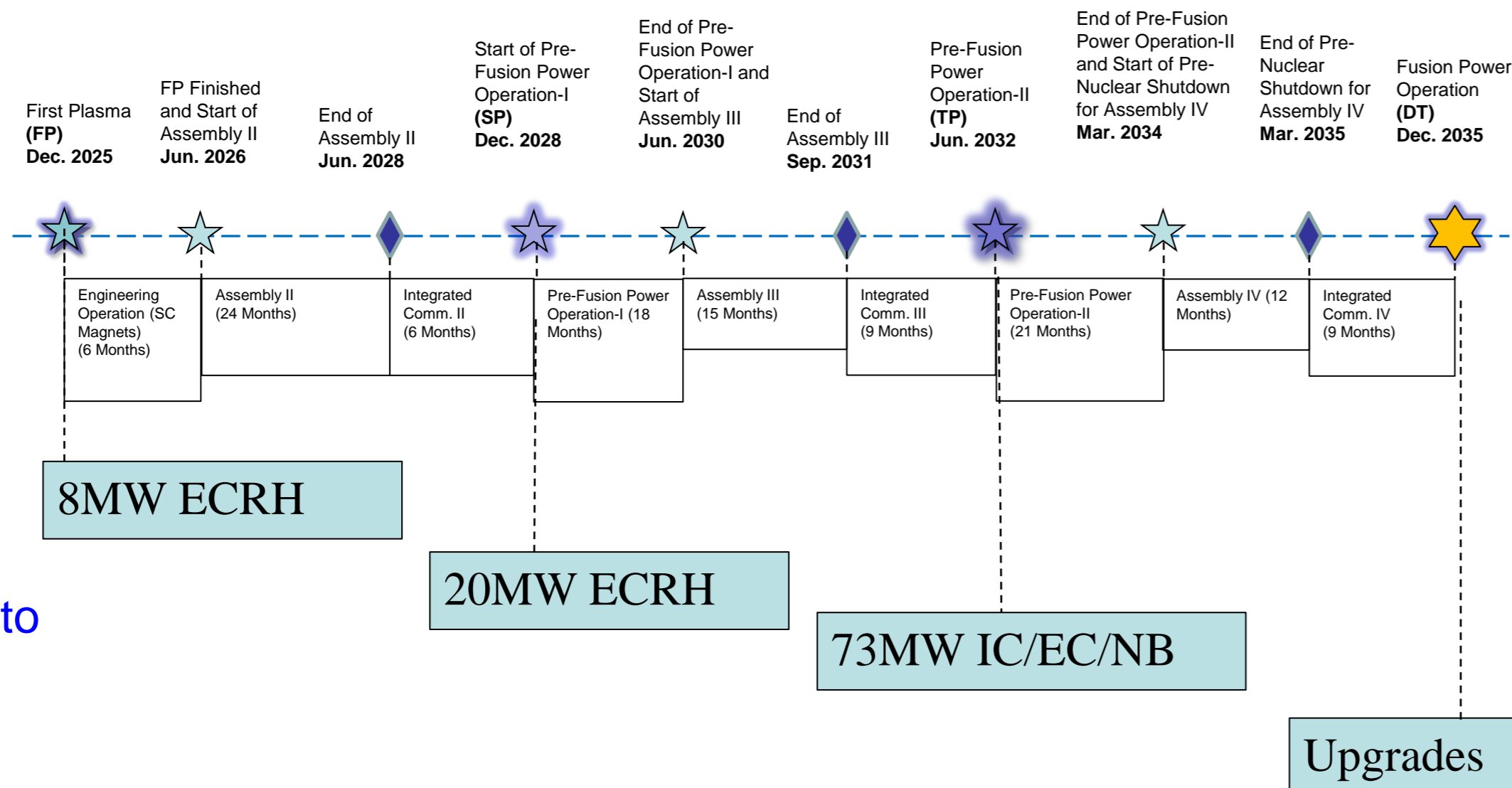
[1] R. Hemsworth, et. al., Nuclear Fusion, 49 (2009) 045006
[2] A. Chakraborty, et. al., IEEE Transactions on Plasma Science, 38 (2010) 2

2. Recent developments – staged approach & new schedule

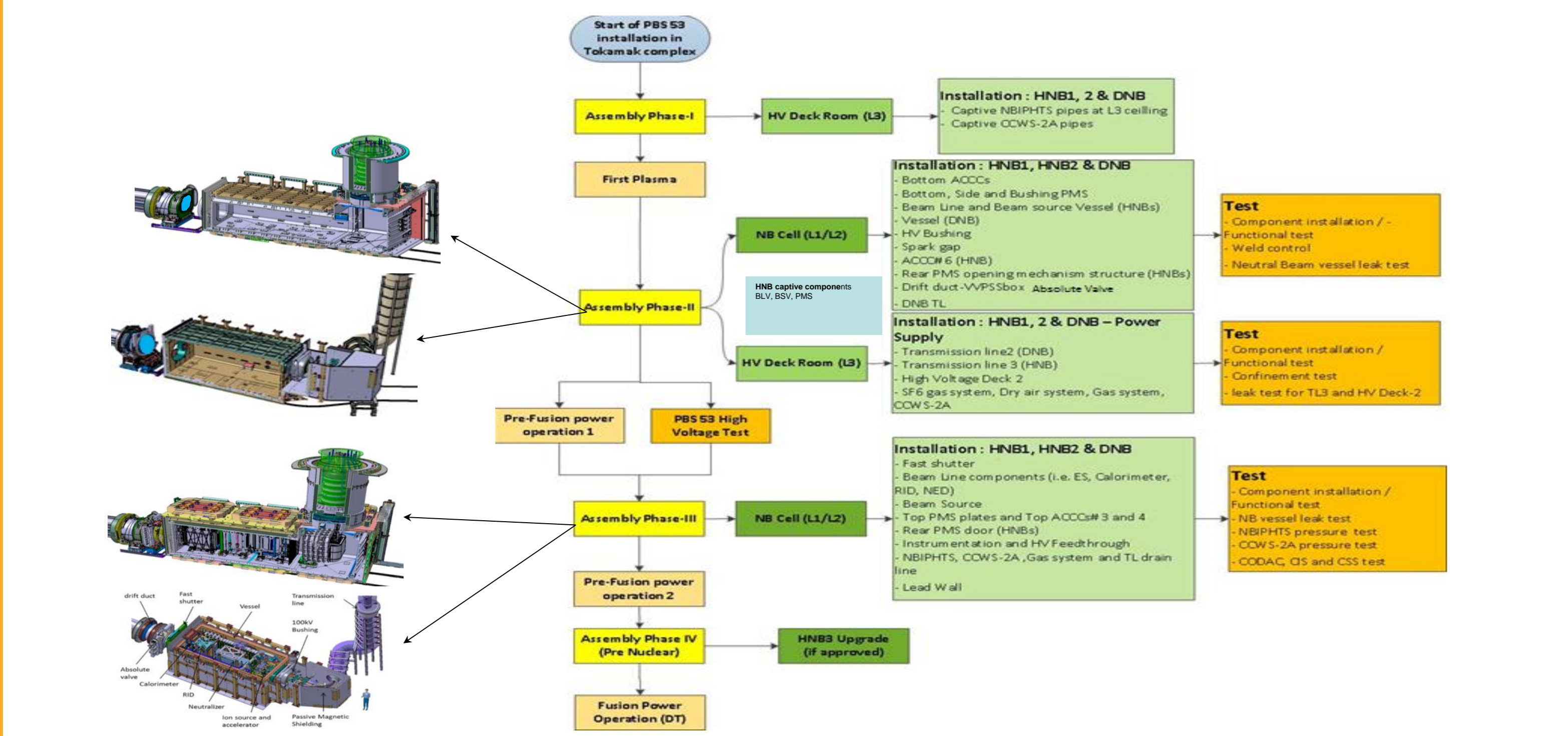
- The ITER Council has approved the Overall Project Schedule and – ad referendum – the Overall Project Cost: First Plasma in 2025, followed by a staged approach to DT Operation/full fusion power capability in 2035.
- There is strong progress in on-site construction and fabrication, matched by strong progress worldwide in component manufacturing.
- 19 of 20 milestones were successfully completed on schedule in 2016. The last milestone was delayed 3 months due to discovery of a flawed weld, but the welding was re-done with no impact to the overall schedule.

HCD configuration "staged approach"

- IC November 2016
 - RoD:.....
- On this basis an acceleration proposal is being developed in order to accelerate the ICRH.

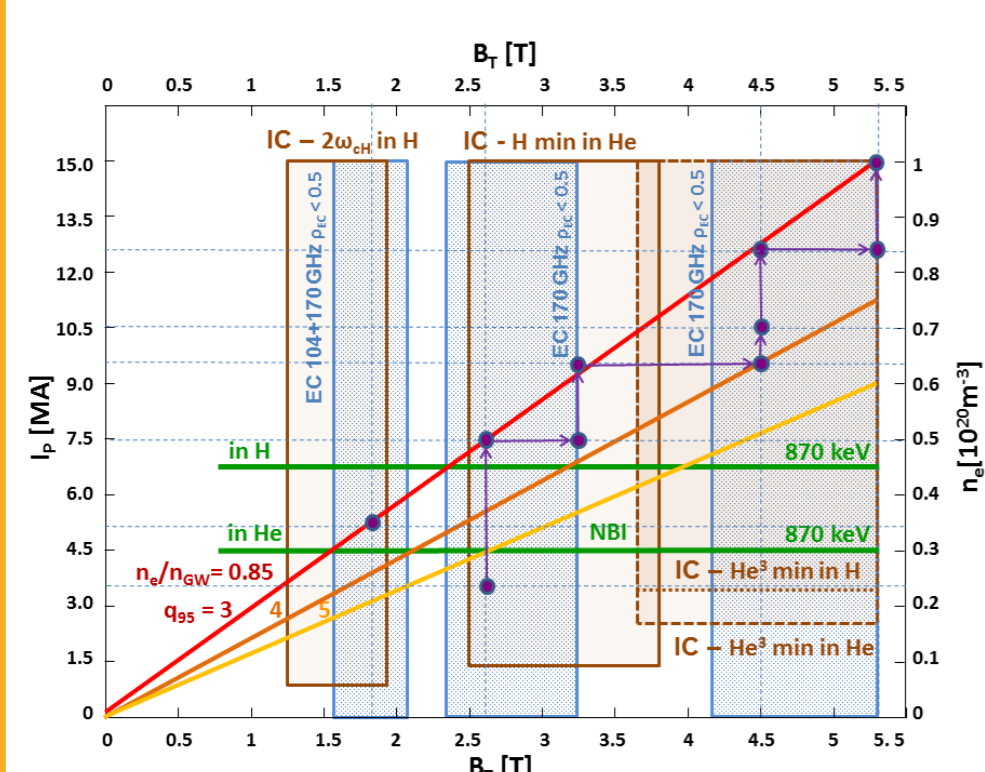


3. Staged approach: NB System Assembly Installation Plan



4. Third field operation and impact on NB heating system

Heating Neutral beams : PFPO-2 phase 2031 onwards; no hardware changes.
Physics analysis : by SCOD and HCD ongoing.



Diagnostic Neutral beam : No changes envisaged; but shine-through has to be checked

- Shine through limits :**
- Lower beam energies will allow operation at 1.8T respecting the shine-through density of the lower plasma density.
 - Under discussion
 - Beam energies of 870keV several seconds pulse lengths, to limit damage to the blanket shield block 16 DS.
 - Beam energies of 650keV for helium plasmas and 600keV for hydrogen plasmas at pulse lengths > 100 s.
- Operation at lower powers :**
- Still considered interesting and as synergy with the IC coupling mechanisms at second harmonic Hydrogen in hydrogen plasmas.
- The design of the front end components and duct:**
- Designed for both half and full field operation. Full power load analysis in the duct pending to confirm compliance with maximum power loads.

5. Overall Design Status and R&D Strategy

Front-End-Components: EUDA
- Drift duct – Final design phase
- Fast Shutter – Preliminary design phase
- Absolute valve – Preliminary design phase
- VVPS box: INDA – Final design phase
- VVPS – Vacuum Vessel Pressure Suppression System

Active Correction & Compensation Coils: EUDA
- Final design phase

HV Bushing: JADA
- Final design phase. MITICA HV Bushing in testing phase.

DNB INDA
- Beam Source, Residual Ion Dump, Neutralizer under manufacturing
- Acceleration Grid Power Supply installed in Indian Test Facility
- All components in final design phase except PS which are functional specifications.

Beam source: EUDA/JADA
- MITICA Beamsource (BS) in tender phase. HNB BS final design similar to MITICA.

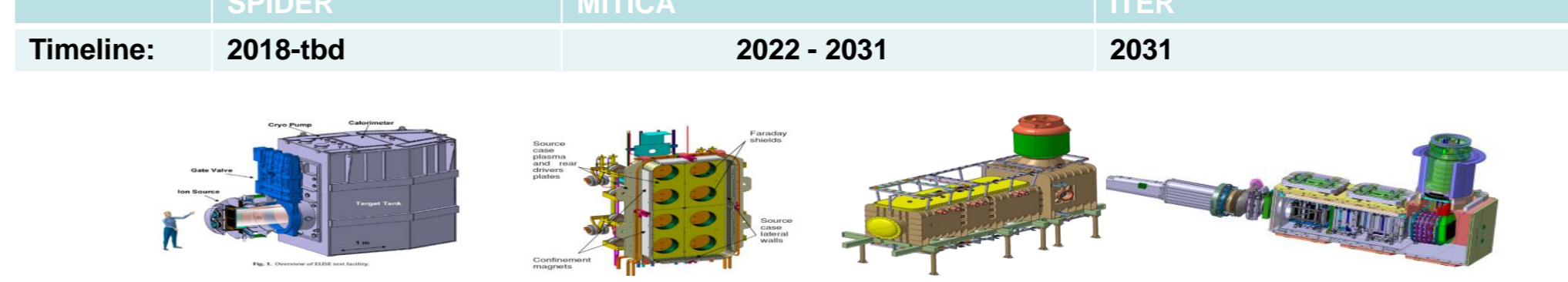
Vessel, Passive Magnetic Shield: EUDA
- Final design phase.

Neutralizer, Residual Ion Dump, Calorimeter (BLC's): EUDA
- MITICA BLCs in preparation for tender phase. HNB BLC final design similar.

Assembly Tools: EUDA
- Preliminary Design Phase.

Timeline:

SPIDER	MITICA	ITER
2018-tbd	2022 - 2031	2031



Indian Test Facility INTF: ROBIN & Twin Source, Demonstration of DNB Performance including far-field divergence
[3] A. Chakraborty, et. al., 1-6. 10.1109/SOFE.2015.7482367

6. Design updates and integration Issues

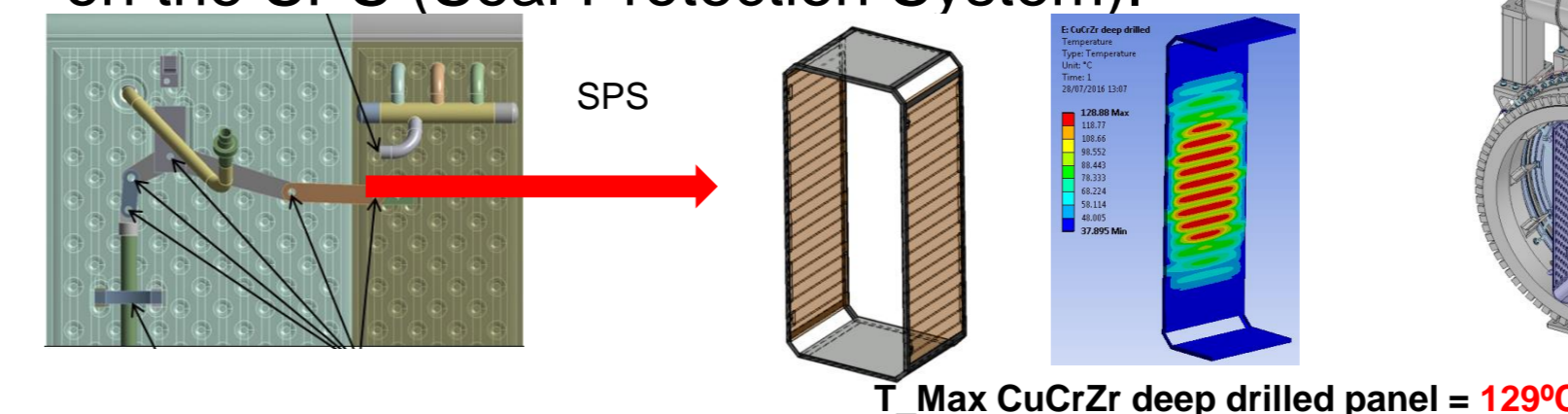
- Update of reference beam source design based on ELISE results and manufacturing experience**
 - Electromagnetic shield for RF drivers
 - Increased RF Line size to 3 1/8"
 - Plasma Grid (PG) Modification (shine-through prevention by cover plate between grid segments)
- Implemented in SPIDER & DNB sources currently being manufactured and in reference source design (MITICA & HNB sources).



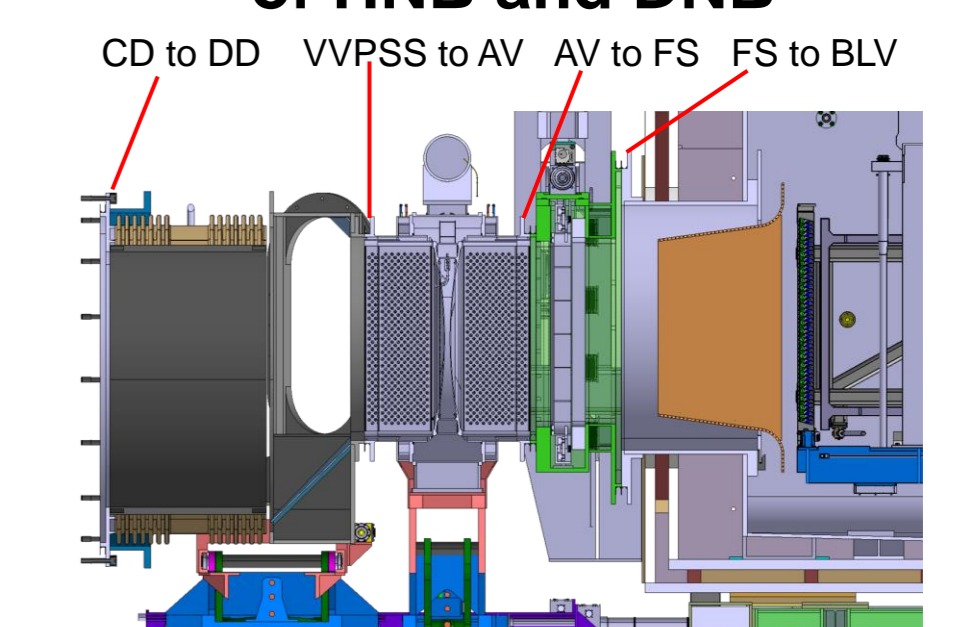
SPIDER source assembly being finalized, delivery to NBTF scheduled on 16/10/2017. DNB beam source expected to be delivered to INTF in 2018.

Absolute valve

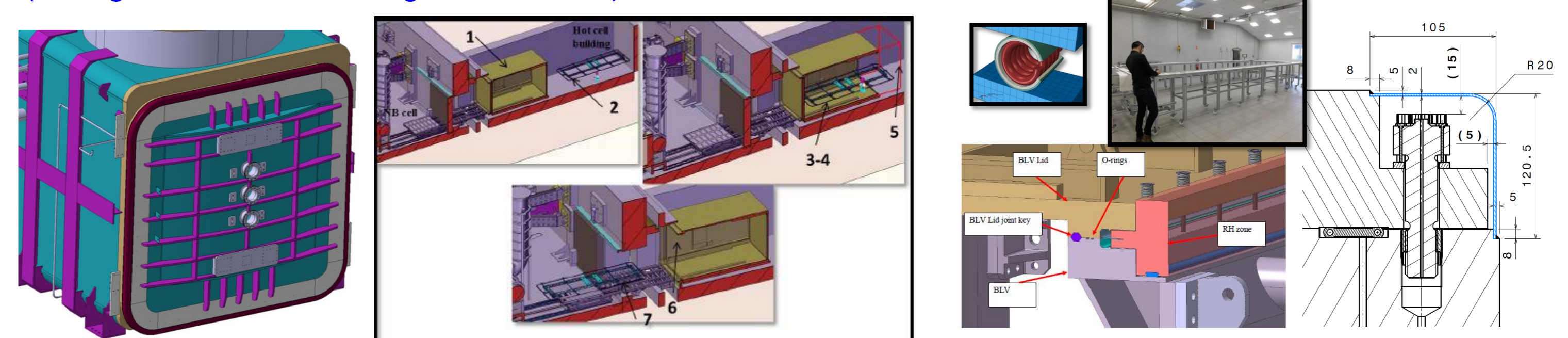
CuCrZr deep drilled panels are replacing the hydro-formed panels on the SPS (Seal Protection System).



Double metallic seal for FEC's of HNB and DNB

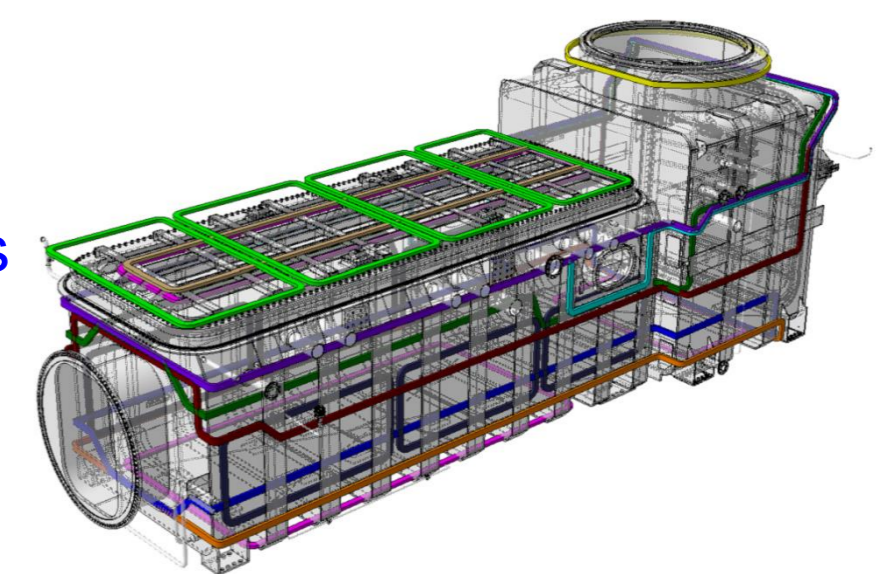


- Hybrid sealing solution for vessel** Double elastomer/metallic seal + welded solution
- Concerns about baseline lip solution – RH for welding and inspection not demonstrated, limited number of openings, no recovery in case of damage to lip.
- Double elastomer for testing, double metallic seal for pre-nuclear and welded cap for nuclear operation (configuration under design finalisation).



MFRS re-design after dynamic analysis

Currently ongoing
Dynamic analysis performed with updated building model to define the controller for the Magnetic Field reduction system => required coil currents required to meet the stringent residual field requirements exceed PS dimensions.
Redesign of coils under way. Options under investigation – additional active side coils and / or active coils inside the Passive Magnetic Shield Work ongoing.



Preclusion of counter injection and Duct liner water connectors

Detailed calculations of gas density and re-ionised power load profiles in the HNB duct have been carried out because of new information on the atomic flow from the Tokamak plasma into the duct.
HNB power deposition $PD_{max} = 2.4 \text{ MW/m}^2$ with a total integrated power of **0.3 MW**.

- The introduction of the VV connecting duct liner
- The preclusion of counter injection as protection not symmetrical.

