

Overview of the high-power CW RF systems of the WEST tokamak and new developments

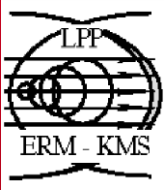
CWRF2018, 26/06/2018

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IRFM, nuclear fusion & WEST tokamak

**Particularities of the high power CWRF
systems in nuclear fusion**

WEST ICRF system

WEST LHRF system

IRFM new developments at LHRF

**Summary & potential common interest with
particle-accelerator community**



IRFM, nuclear fusion & WEST tokamak

Particularities of the high power CWRF systems in nuclear fusion

WEST ICRF system

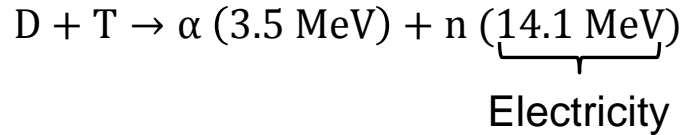
WEST LHRF system

IRFM new developments at LHRF

Summary & potential common interest with particle-accelerator community

- Institute at the Fundamental Research Division of the French Alternative Energies and Atomic Energy Commission (CEA).
- Localized at CEA-Cadarache site (close to Marseille, south of France).
- IRFM Staff: ~250 (~10% CEA-Cadarache staff), ~220 permanent, ~30 PhD postdocs and trainees.
- Works on physics & technology for nuclear fusion by magnetic confinement.
- Main experimental device: WEST tokamak.
- Also works for the International Thermonuclear Experimental Reactor (ITER) & other fusion devices worldwide (Europe, China, Korea, Japan, USA, India).

- Objective: nuclear fusion power plant.



- Nuclear fusion reactor very attractive:
 - No long-term nuclear waste.
 - Runaway reactions are impossible.
 - Reduced CO₂ emissions.
 - T bred from Li in-situ.
 - D and Li largely abundant.
- However: need to overcome Coulomb forces, this requires ~10-20 keV ($\equiv \sim 150 \times 10^6 \text{ }^\circ\text{C}$).

- ~10-20 keV → Plasma state. Plasma needs confinement.

- Tokamak:
 - Magnetic confinement.
 - Requires plasma current (I_p).

- I_p can be inductively generated.

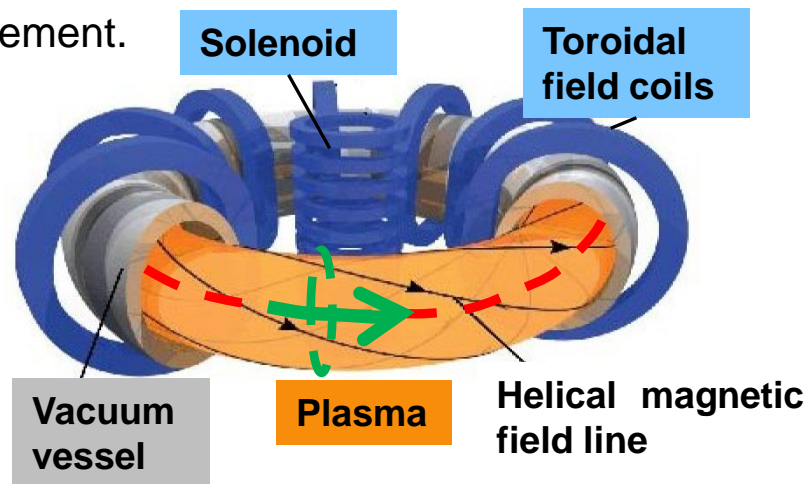
- I_p also heats the plasma by Joule effect.

- However:

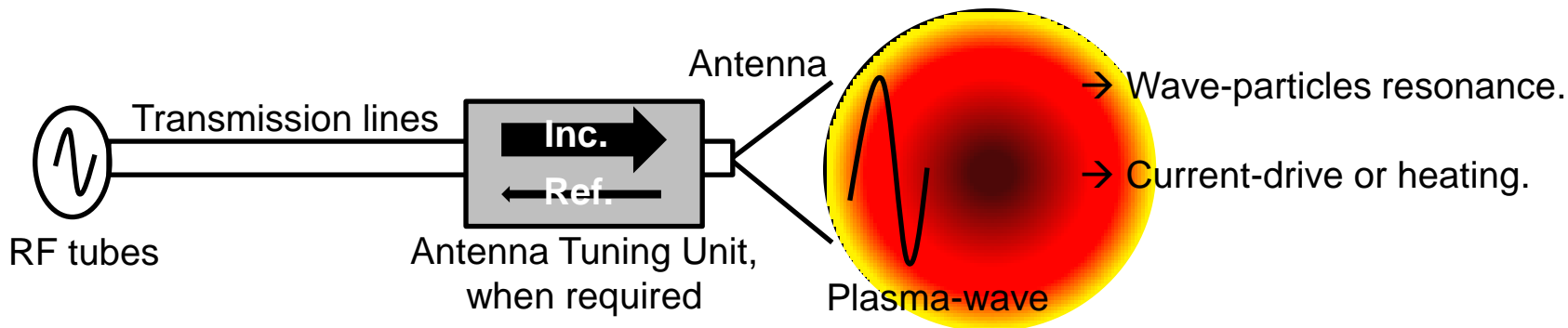
- Plasma resistivity \searrow with the temperature → I_p insufficient to reach ~10-20 keV.
- Inductive I_p has finite duration → Tokamak is intrinsically pulsed device.



Steady-state tokamak reactor requires auxiliary heating & current-drive (CD) systems (electromagnetic + other such as Neutral Beam Injectors).



■ General configuration:







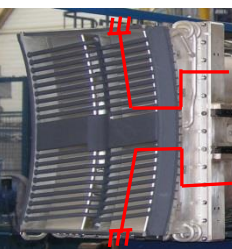




■ $Z_{in,antenna} \neq Z_0 \rightarrow$ Power reflection.

- $V_{max} \nearrow, I_{max} \nearrow$: arcing, overheating.
- Reflection + losses: efficiency \searrow .
- System's safety altered.

} Impedance-matching is crucial.

MW-range electromagnetic heating and current-drive systems in nuclear fusion

Frequency	Ion Cyclotron Range of Freq. (ICRF, ~30-120 MHz)	Lower Hybrid Range of Freq. (LHRF, ~1-10 GHz)	Electron Cyclotron Range of Freq. (ECRF, ~50-200 GHz)
Tubes	 <p>Tetrodes</p>	 <p>Klystrons</p>	 <p>Gyrotrons</p>
Trans. lines	 <p>Rigid coaxial lines</p>	 <p>Rectangular/circular, mono-mode/oversized waveguides</p>	 <p>Circular oversized and corrugated waveguides</p>
Antennas	 <p>Phased arrays of electrically small loops</p>	 <p>Phased arrays of rectangular waveguides</p>	 <p>Quasi-optical mirrors</p>
	Generally, heating	Generally, current-drive	

This talk

- Tore Supra (1st plasma in 1988): a pioneer in long pulse operation.
 - Superconducting toroidal coils.
 - Actively cooled plasma facing units.
 - Pressurized water loops.
 - 15 MW of CWRP power.
- World record of injected/extracted energy in a tokamak (6 min, 1GJ in 2003).
- Reached coupled RF power:

ICRF	~10 MW / 1.5s / 3 antennas, ~4 MW / 1s / 1 antenna.
LHRF	3.8MW / 5s / 1 antenna, 2.7MW / 78s / 1 antenna.
ICRF+LHRF	6.2 MW / 150 s.

- Tore Supra upgraded to WEST (1st plasma in December 2016).



Torus major radius	2.5 m
Torus minor radius	0.5 m
Toroidal field	3.7 T @ R=2.5m
Plasma current	Up to 1 MA
ICRF power	Up to 9 MW
LHRF power	Up to 7 MW
Pulse duration	Up to 1000s



IRFM, nuclear fusion & WEST tokamak

Particularities of the high power CWRF systems in nuclear fusion

WEST ICRF system

WEST LHRF system

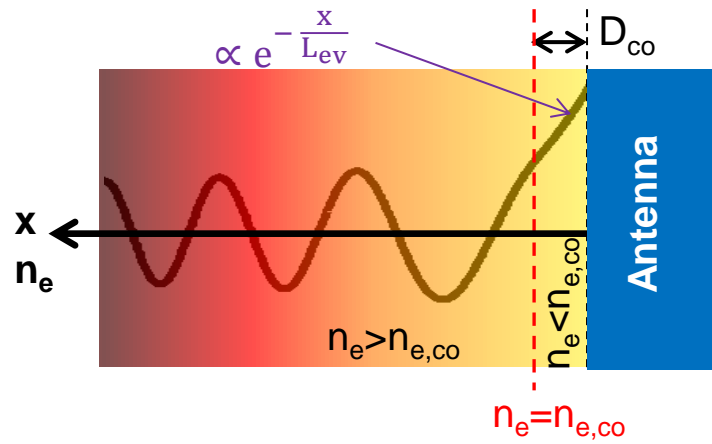
IRFM new developments at LHRF

Summary & potential common interest with particle-accelerator community

- Magneto-plasma: gyrotropic dielectric.
- Birefringence:
 - Slow-Wave.
 - Fast-Wave.
- Excite proper plasma-wave → Proper antenna polarization.
- Generally required: $|N_z| > 1$ → Particular phased-arrays, typically interspaces $\sim \lambda_0/10$.
- Challenge of antenna-plasma coupling:

Stix tensor:

$$\mathbf{K} = \begin{bmatrix} S & jD & 0 \\ -jD & S & 0 \\ 0 & 0 & P \end{bmatrix}$$



- High vacuum ($<10^{-5}$ Pa) & high temperature ($\sim 200^{\circ}\text{C}$) environment.
- Eddy currents \rightarrow Large forces & torques (~ 500 N.m in ms-time scale, ex: plasma transients).
- Large heat fluxes ($\sim \text{MW}/\text{m}^2$).
- Nuclear-safety constraints & compatibility with remote handling (ex. on ITER).
- Need to excite proper plasma-waves. Need for proper phased-arrays with proper polarization.
- Need to optimize edge electron density profile (ex. $n_e \nearrow$ by gas-puffing), optimize plasma shape, etc. in order to $V_{\perp} / I_{\perp} / |\Gamma|_{\perp}$, and optimize power coupling.

- Very non-stationary plasma → Harsh non-stationary RF loading. Real-time controlled &/or intrinsically immune (*aka* load-resilient) impedance matching required.
- High/CW RF currents (\sim kA / \sim 1000s). Thermomechanical considerations & active cooling.
- High electric fields (\sim MV/m). Need for arc detection systems (VSWR-based, optical, acoustic, ratio between RF signals, RADAR-based, S-matrix based *aka* SMAD, SHAD, etc.).



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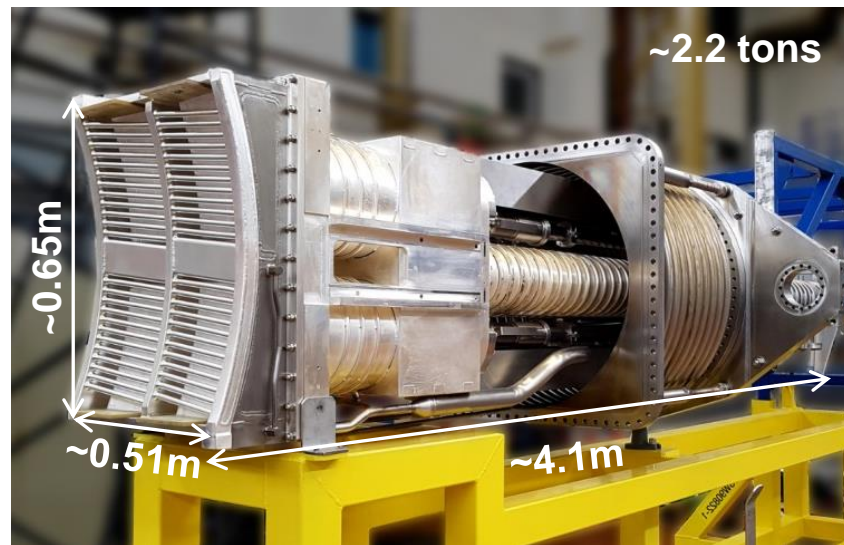
Summary & potential common interest with particle-accelerator community

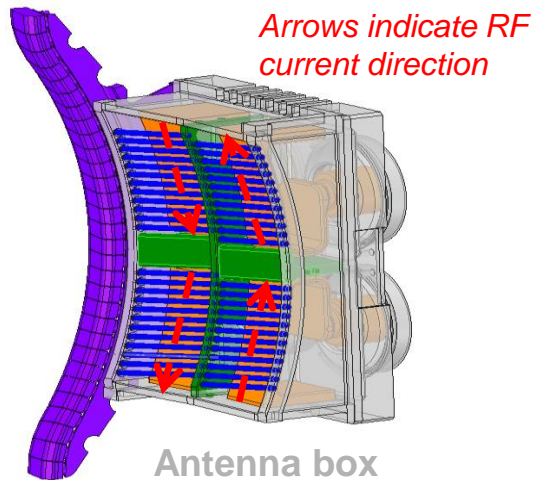
■ Features:

- Per antenna: 3MW/30s or 1MW/1000s, 2 power inputs.
- 48-60 MHz.
- Load-resilience with VSWR < 2:1.
- Symmetric spectrum & $|N_{z,M}| \sim 10$.
- Water actively cooled (70°C/30 bar).
- Vacuum compatible (10^{-5} Pa, $\sim 150^\circ\text{C}$).

■ RF measurements (amplitude & phase):

- Voltage probes at straps inputs (vacuum compatible).
- Directional couplers (P_i , P_r) at antennas inputs.

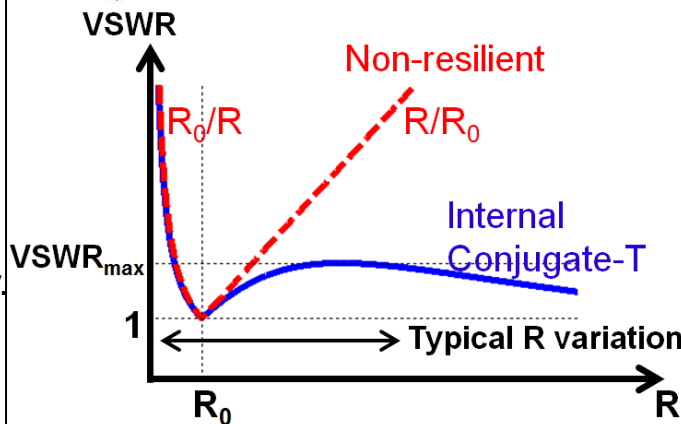
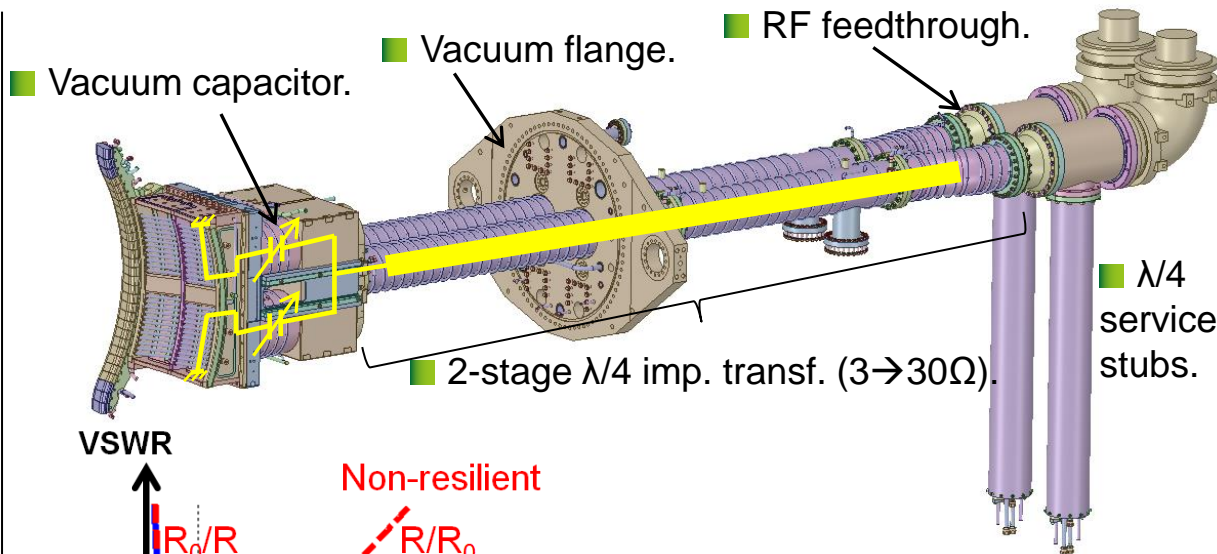




Antenna box

Straps or loops ($Z_s=R+jL\omega$)

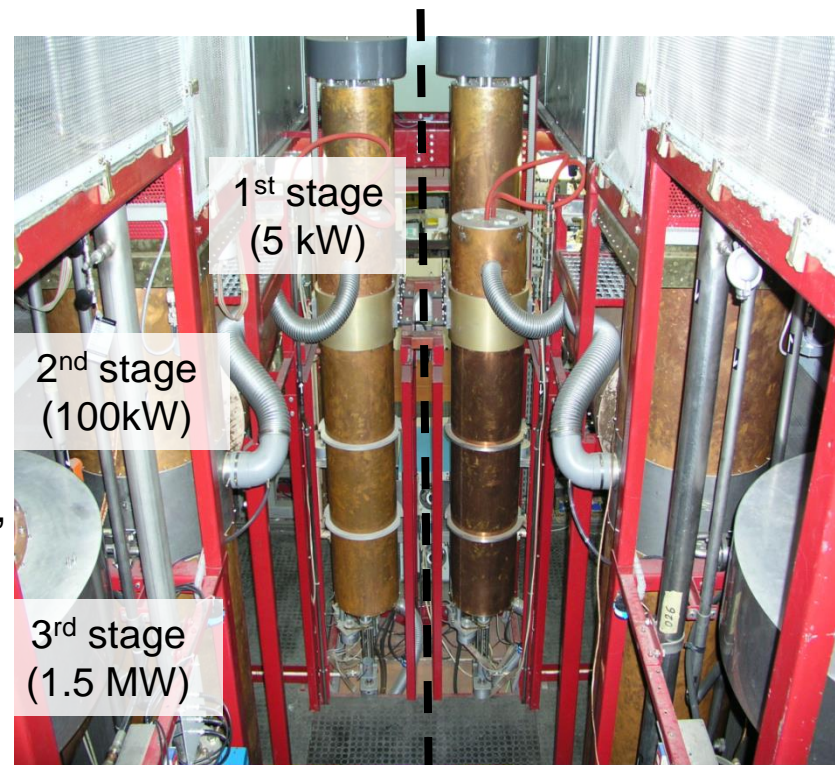
Septa, Faraday screen, Limiters



Internal Conjugate-T matching \rightarrow Load-resilience against RF loading variations (ms & sub-ms).

- $R \propto P/I^2 \propto P/V^2 \rightarrow R \nearrow$ crucial.
- $|\Gamma_{strap}| > 0.9 \rightarrow$ Matching mandatory.
- $Q \sim L\omega/R \gg 1 \rightarrow$ Sensitive matching.
- R very non-stationary.

- 3 modules (1 module / antenna), 1 module \equiv 2 generators (1 generator / $\frac{1}{2}$ antenna).
- Each generator:
 - Solid state amplifier (200W).
 - 3-stage tetrodes.
- Control system with $\sim 10 \mu\text{s}$ time-scale:
 - FPGA calculations.
 - Inputs: dissipated power, grid & anode currents, P_i & P_r , arc detection, etc.
 - Outputs: power trips, regulation & limitation (ex. power is reduced if $P_r > 200\text{kW}$).



TITAN vacuum chamber:

- Vacuum leak tests (10^{-5} Pa, $\sim 150^{\circ}\text{C}$).
- High CW RF voltages/currents tests ($\sim 30\text{kV}/915\text{A}$ peak @ straps inputs).



■ Low-power dummy load ($\sim \text{mW}$, tests using VNA) :

Radially moveable (0-10cm) aquarium, hosting high $|\epsilon_r|$ media (BaTiO_3 mixtures, optimized salty water).

- Validate RF design.
- Check frequency range.
- Fill look-up tables with ATU settings.
- Assess load-resilience (sweep antenna/load distance).



Antenna validation before installation in the tokamak & accelerate commissioning on plasma

- Detection within $\sim 30 \mu\text{s}$. Power tripping during $\sim 30\text{ms}$ before reapplication.

- Arc detection systems:

- V_r/V_i at antenna & generators ($VSWR_{\text{threshold}} \sim 4$).

⚠ But do not protect the full system (ex. low-Z regions)



Undetected arcs

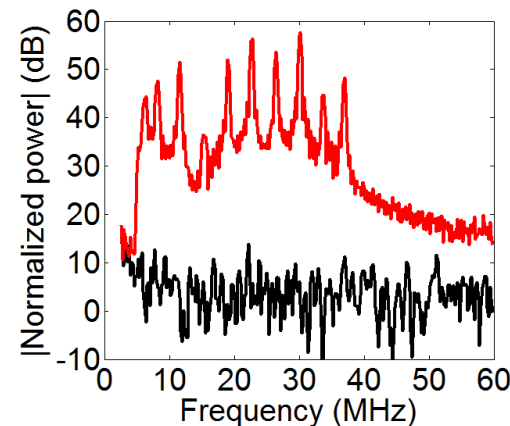
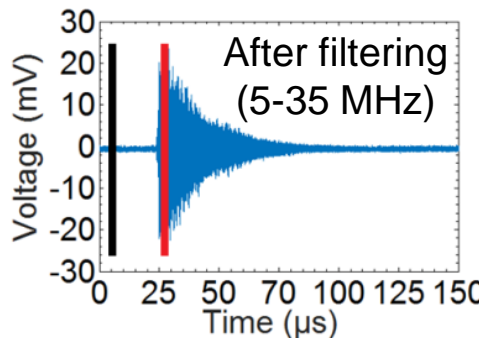


⚠ Complementary arc detection systems are required.

- Optical arc detection @ low-Z regions.

- Sub-Harmonic Arc Detection (SHAD):

+ Under development: FPGA-based SHAD for discrimination between arcs and spurious noise.





IRFM, nuclear fusion & WEST tokamak

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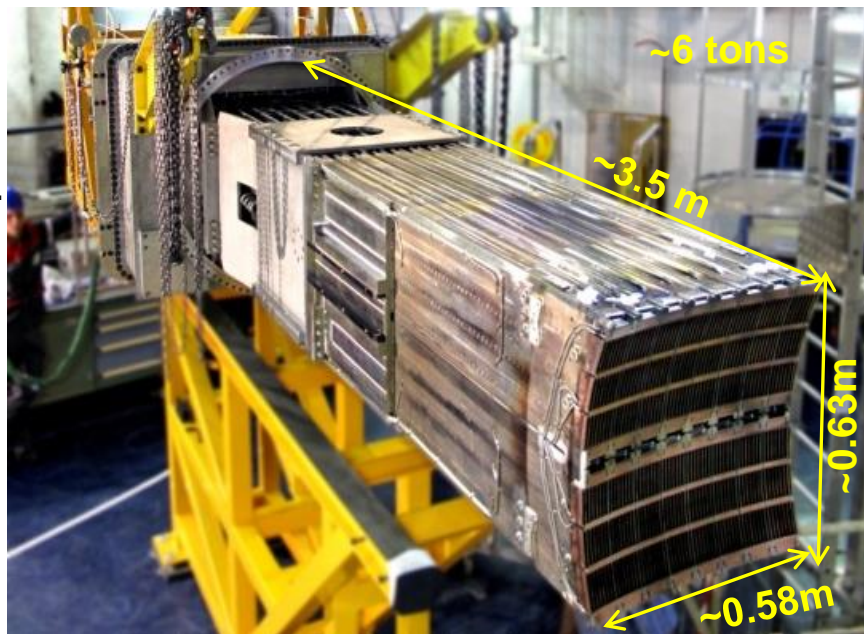
WEST LHRF system

IRFM new developments at LHRF

**Summary & potential common interest with
particle-accelerator community**

■ Features:

- 3.7 GHz. 3-4MW / 1000s.
- ~300 reduced-height waveguides ($\sim 70 \times 8 \text{ mm}^2$).
- Directional spectrum & $|N_{z,M}| \sim 2$.
- Water actively cooled (150°C / 30 bars).
- Vacuum compatible (10^{-5} Pa , $\sim 150^\circ\text{C}$).



■ 3dB splitters, directional couplers, DC blocks.

■ BeO pill-box RF window.

■ Step ϕ -shifter.

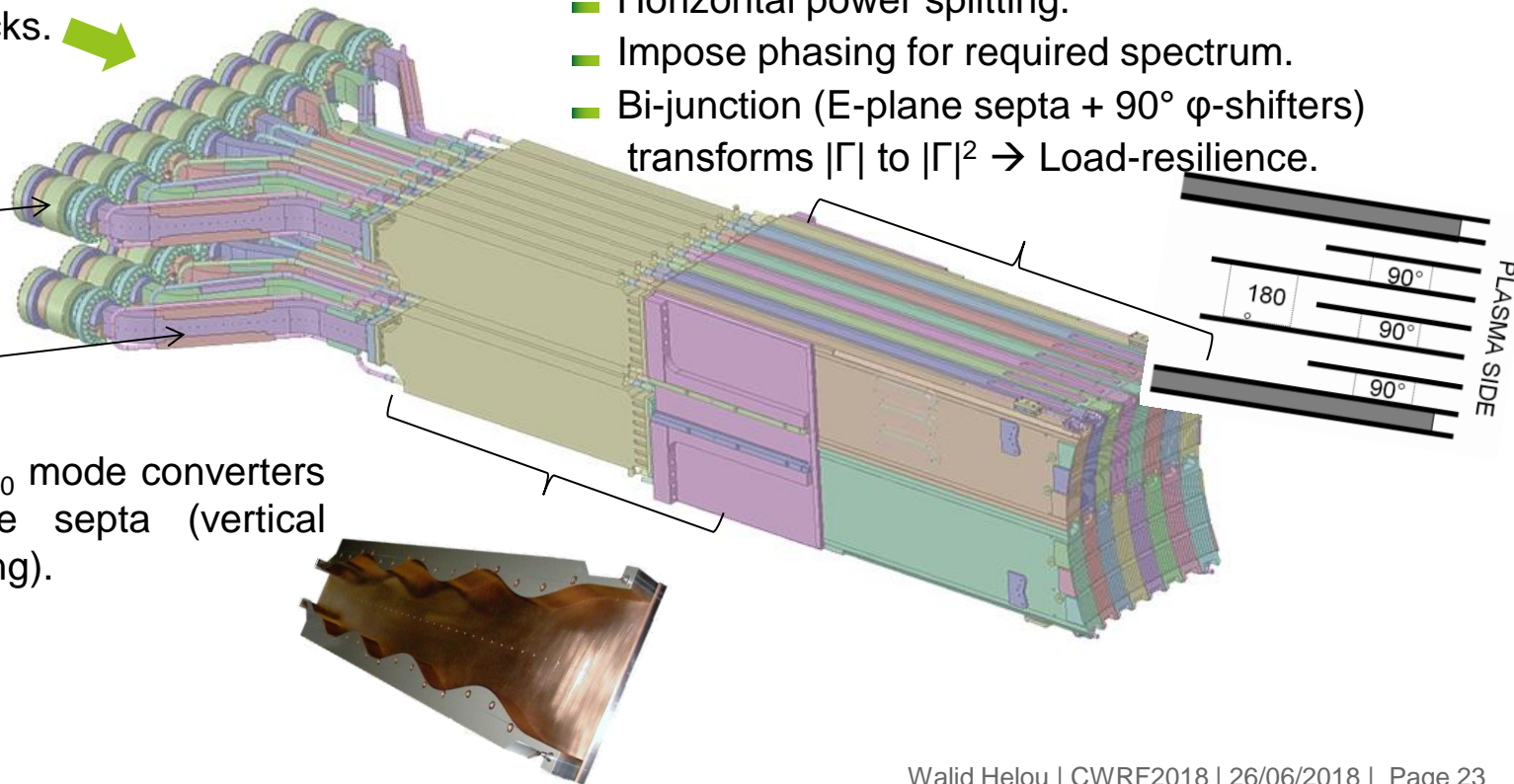
■ TE_{10} - TE_{30} mode converters and H-plane septa (vertical power splitting).

■ Multi-junctions (E-plane septa, step ϕ -shifters):

■ Horizontal power splitting.

■ Impose phasing for required spectrum.

■ Bi-junction (E-plane septa + 90° ϕ -shifters) transforms $|\Gamma|$ to $|\Gamma|^2 \rightarrow$ Load-resilience.



- 16 klystrons (8 per antenna).
- Specs for a klystron:

Frequency	3.7 GHz \pm 5 MHz
Power / 1000s	700 kW (VSWR<1.1:1) 600 kW (VSWR<1.4:1)
Efficiency	38-44%
Gain min	50 dB (5 cavities)

- Each klystron features a dual-output followed by BeO RF windows and a power combiner.



- Power tripped ($\sim 10\mu\text{s}$) and switched-off, if:
 - Vacuum in klystron $>$ threshold.
 - $I_{\text{beam}} >$ threshold.
 - P_r @ klystron $>$ 7 kW.
 - Arc (optical detection) @ klystrons RF windows. Maximum allowed trips: 1.

- Power tripped ($\sim 10\mu\text{s}$) and reapplied (after $\sim 10\text{ms}$), if:
 - Arcs (optical detection) @ antennas RF windows or splitters dummy loads. Maximum allowed trips: 7.
 - P_r/P_i @ antenna $>$ 0.2. Maximum allowed trips: 100.

- Power reduced, if:
 - Copper level increases in the tokamak.
 - Antennas front-face temperature $>$ threshold (infrared camera security).

Arc detection @ RF windows is essential





IRFM, nuclear fusion & WEST tokamak

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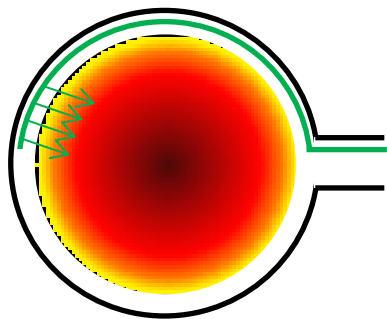
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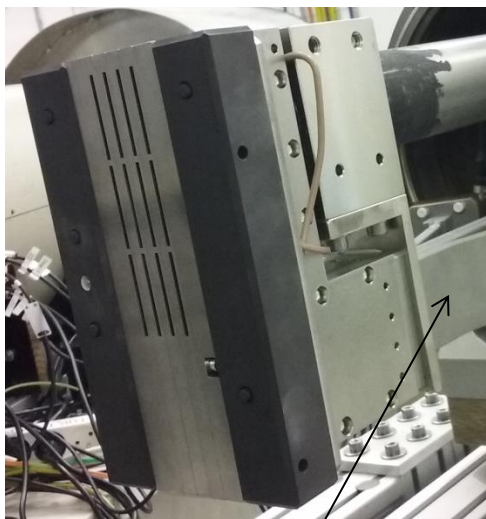
**Summary & potential common interest with
particle-accelerator community**

■ LHRF slotted waveguide antenna array:

- Off-port extension.
- Wave-coupling from limited-access regions.



mW prototype tested @
COMPASS tokamak



3D printed (metal) waveguide feeder

■ LHRF metamaterial low-power loads:

- Pre-qualification of LHRF antennas





IRFM, nuclear fusion & WEST tokamak

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WEST ICRF system

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IRFM new developments at LHRF

**Summary & potential common interest with
particle-accelerator community**

- Very particular high power CW RF systems in nuclear fusion:
 - High vacuum / temperature environment.
 - Large heat and electromechanical loads.
 - Very particular phased-arrays.
 - Challenge of antenna-plasma coupling.
 - Harsh non-stationary RF loading.
 - High RF voltages/currents.
 - Arc detection aspects.

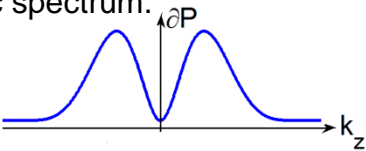
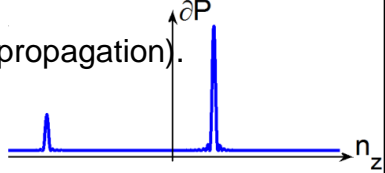
- Potential common interest with particle-accelerator community, ex:
 - Commissioning procedures & pre-qualification tests.
 - Impedance-matching problematics.
 - RF arc modeling, detection & discrimination from noise source.

**Thank
you for
your
attention**

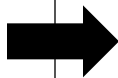
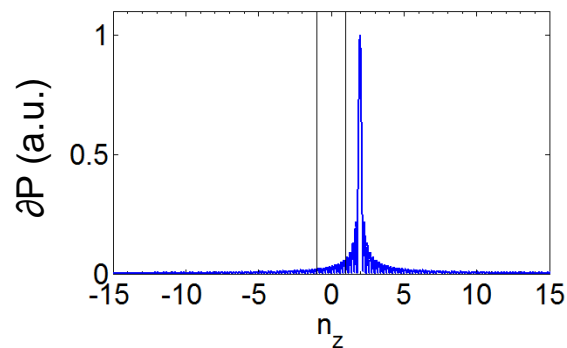
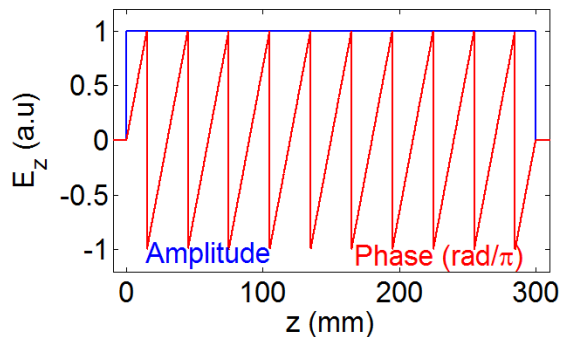
**Your
collaboration
is welcome**

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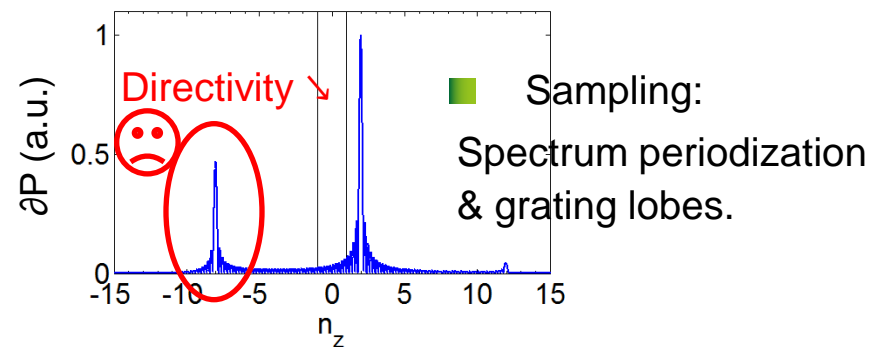
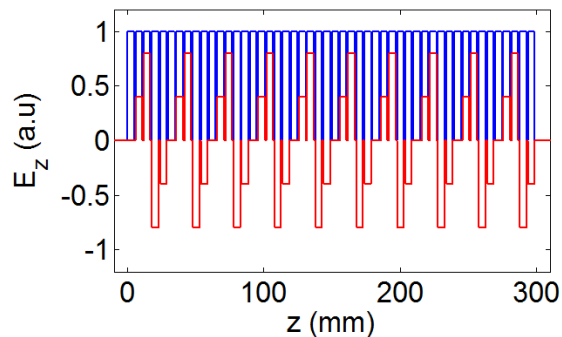
DRF
IRFM
SI2P

	Heating at ICRF (~ 50 MHz)	Current-Drive at LHRF (~ 5 GHz)
Wave-particle resonance	Ion cyclotron resonance: $\omega - N\omega_{c,i} - \underbrace{k_z v_{z,i}}_{\ll \omega} = 0$	Landau resonance: $\omega - \underbrace{N\omega_{c,e}}_{N=0} - k_z v_{z,e} = 0 \rightarrow \frac{\omega}{k_z} = v_{z,e}$
Plasma-wave	Fast Wave (FW)	Slow Wave (SW)
Frequency (f_0)	<ul style="list-style-type: none"> $\omega = N\omega_{c,i}$ at required resonance layer ($\omega_{c,i} \propto B_{static}$). $n_{e,co} \searrow$ when $f_0 \nearrow$ 	<ul style="list-style-type: none"> $n_{e,co}$ depends only on f_0: $n_{e,co} \searrow$ when $f_0 \searrow$
Polarization	$E // y$ -axis → FW: suitable for IC resonance.	$E // z$ -axis → SW: suitable for Landau resonance.
Spectrum	<ul style="list-style-type: none"> Typically, symmetric spectrum. $k_{zM} \sim 5\text{-}15 \text{ m}^{-1}$. $n_{e,co} \nearrow$ when $k_z \nearrow$. 	<ul style="list-style-type: none"> Asymmetric spectrum. $N_{zM} > 1$ (absorption & propagation). Typically: $N_{zM} \sim 2$. 
Typical $n_{e,co}$ & L_{ev}	<ul style="list-style-type: none"> $n_{e,co} \sim 10^{18}\text{-}10^{19} \text{ m}^{-3}$ ($D_{co} \sim 5\text{-}15 \text{ cm}$) $L_{ev} \sim 10 \text{ cm}$. 	<ul style="list-style-type: none"> At 5GHz: $n_{e,co} = 3.1 \times 10^{17} \text{ m}^{-3}$. $L_{ev} \sim 5 \text{ mm}$.

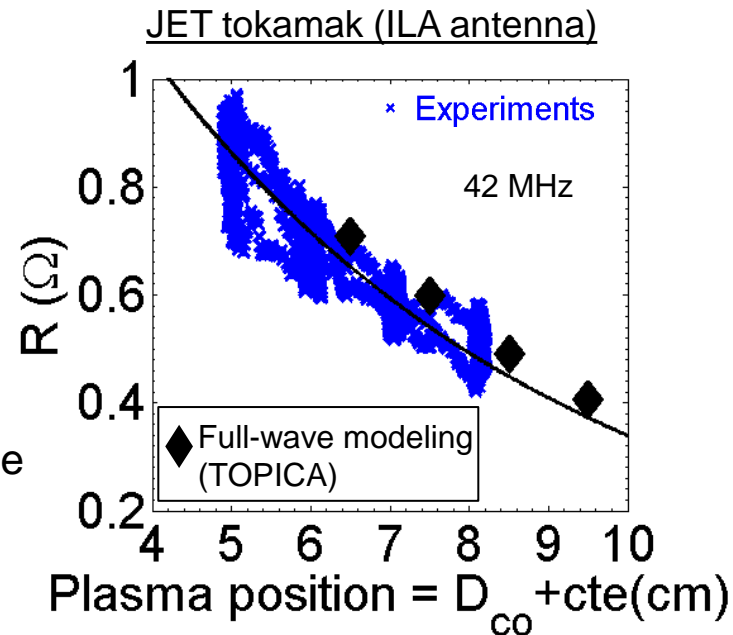
■ Use a single radiator \rightarrow
Technically non-trivial.



■ Spatial sampling \rightarrow Phased array antennas.



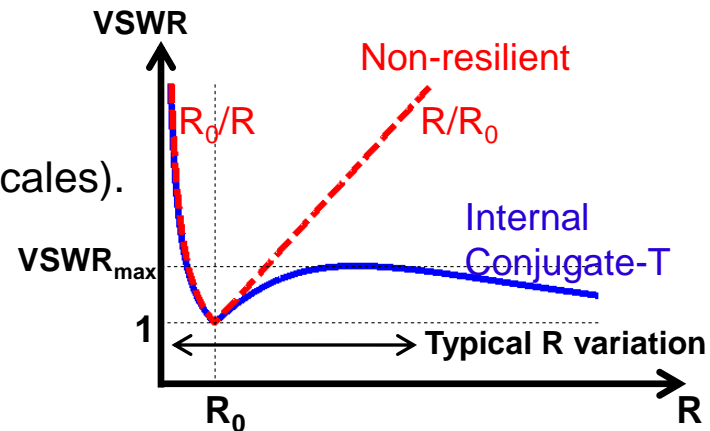
- R very much dependent on edge density profile (D_{co} & ∇n_e).
- Edge density profile hardly controllable
→ Same for R.
- Transitions of plasma confinement modes & plasma edge instabilities impact much edge density profile & R.



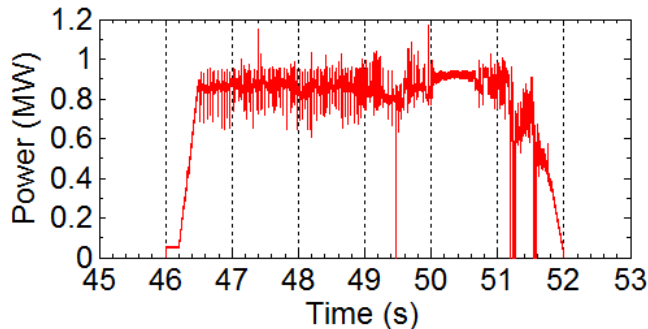
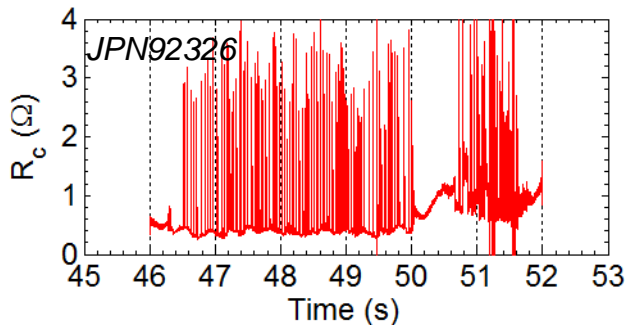
Straps matched separately for R_0 with fixed 2-port lossless ATU: $VSWR=R_0/R$, or R/R_0 .

Need for:

- Load-resilient ATU (inherently: $VSWR < VSWR_{max} \sim 2$):
→ Cope with fast loading variations (ms & sub-ms time-scales).
- Real-time control:
→ Automatically set working point for load-resilient ATU.
→ Relax constraints on load-resilience.



Example: JET tokamak (ILA antenna)



Block diagram of a WEST ICRF launcher as used for SIDON calculations

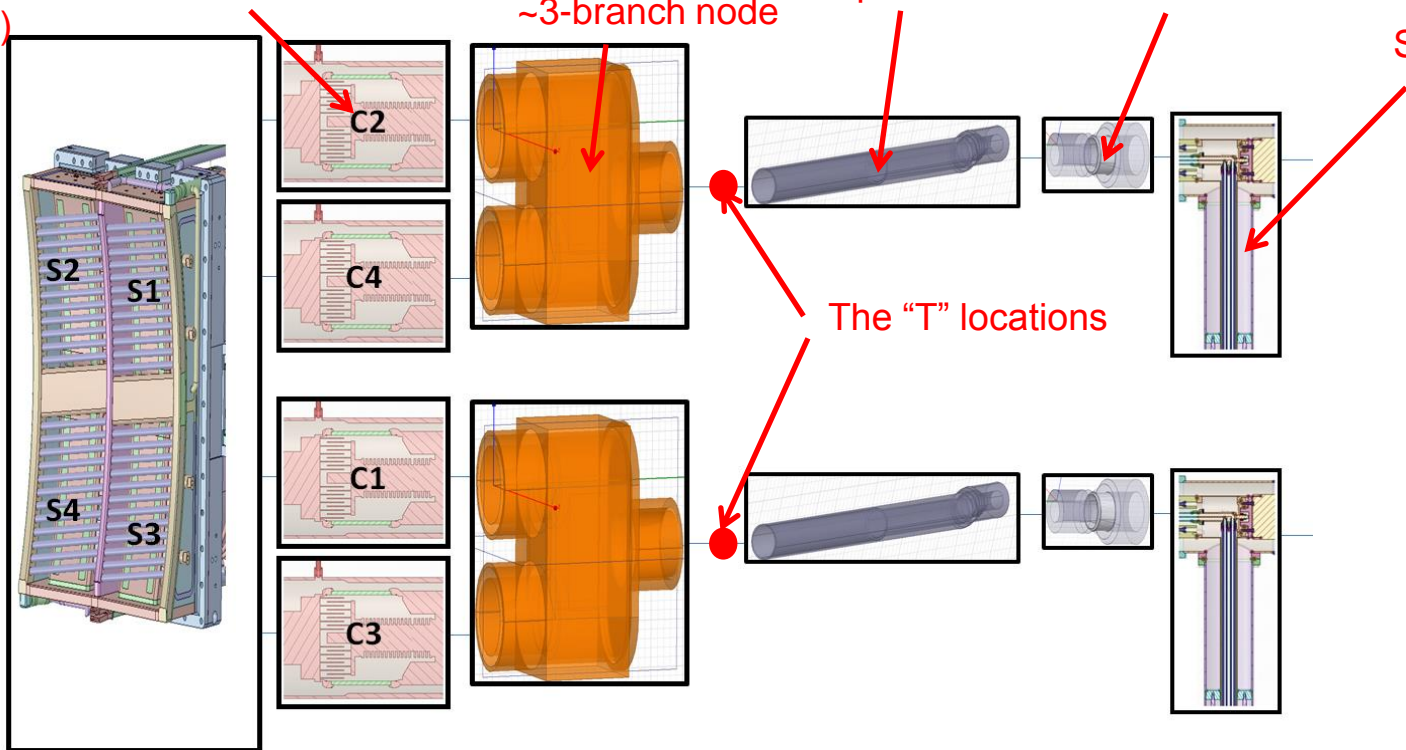
Capacitor: cylindrical concentric electrodes
(with V-probes)

Bridge: ~3-branch node

Imp. transformer

RF window

Service-stub

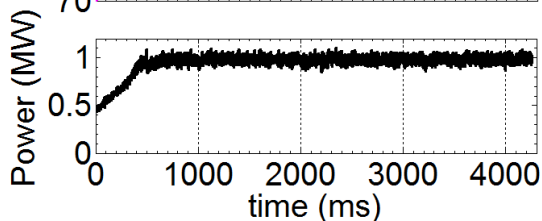
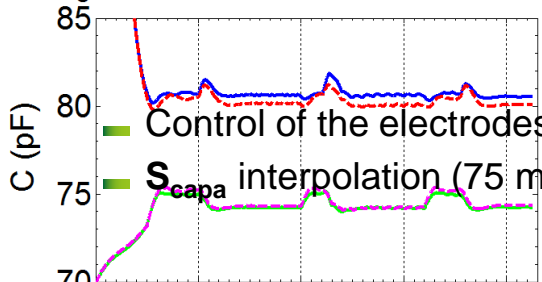
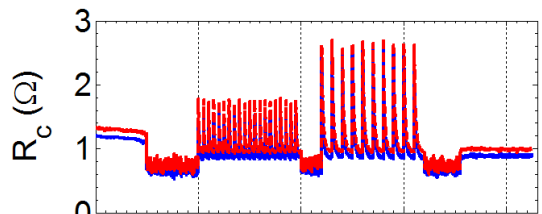


TOPICA/HFSS

HFSS

Simulations of operation scenarios for WEST ICRF launchers using SIDON

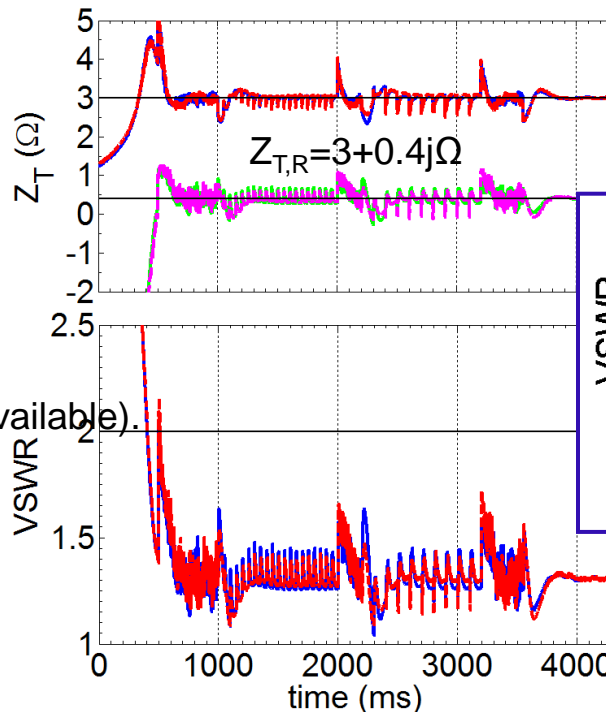
- Scenario is built.
- $S_{\text{front-face}}$ interpolation.



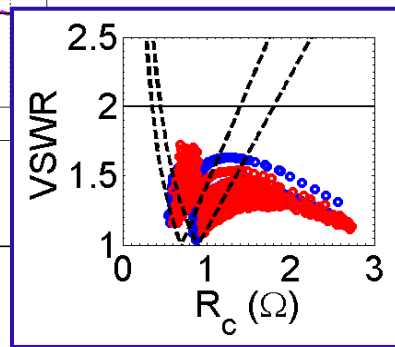
Control of the electrodes overlap.

S_{scapa} interpolation (75 matrices available).

- SIDON solves the RF network every $\Delta t=1\text{ms}$.



- Feedback control can be activated.



✓ Load-resilience