ITER: Status of the Project and Lessons which can be Learned from the Magnets for Future Giant Science

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ITER Magnet Division and CTTA

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Manufacture of the ITER magnets started in 2008 with the superconducting strands, and over 10 years has progressed through the completion of the conductor supply to reach the stage of full scale industrial production of the final coils, with final first-of-kind items nearing delivery and the remainder soon following. Looking at the development since 1988 and the industrialisation post 2008 provides illustrative lessons on the extent to which novel materials and concepts could be rapidly brought into mainstream production in a cost effective way. Among the most critical materials have been:

Insulation Systems

Superconductors

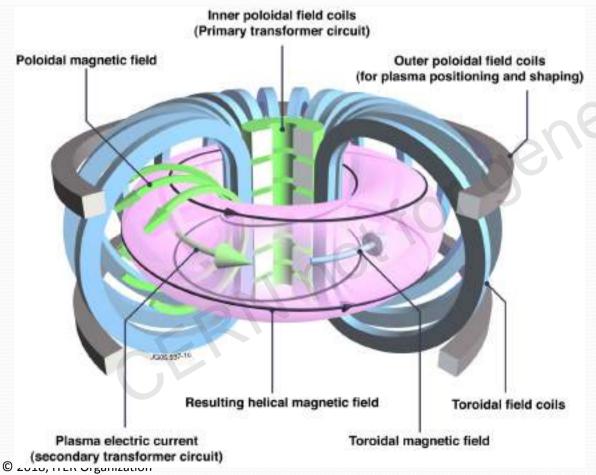
Structural Steels

High Strength Composites

The challenges of these materials have a common theme: choose what performance requirements to use for the baseline design, selecting the R&D targets at levels that are reasonable and promise a cost effective manufacturing route. In this seminar I will look at examples that show how to, and how not to, set these targets, and the results (of both).

1. What are the ITER Magnets

ITER is a superconducting Tokamak

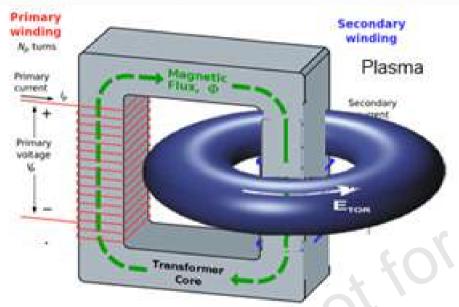


Designed to achieve 500MW fusion power

Plasma carrying a current up to 15MA confined by

- ☐ Toroidal Field Coils
- ☐ Central Solenoid Stack
- ☐ Poloidal Field Coils

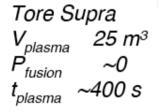
Creating the Plasma Current

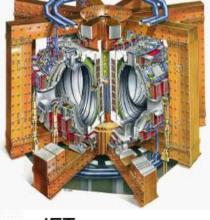


- Break down the plasma (applied electric field and/or ECRH) as a secondary 1 turn coil in a conventional transformer
- Primary winding is largely CS supported by PF
- As well as creating conditions to drive current, need a field configuration that allows plasma to form

Some tokamaks use an iron core to improve coupling to plasma







JET V_{plasma} 80 m³ P_{fusion} ~16 MW 2s t_{plasma} ~30 s

Plasma Shaping

Poloidal Coils

Radial position equilibrium of plasma

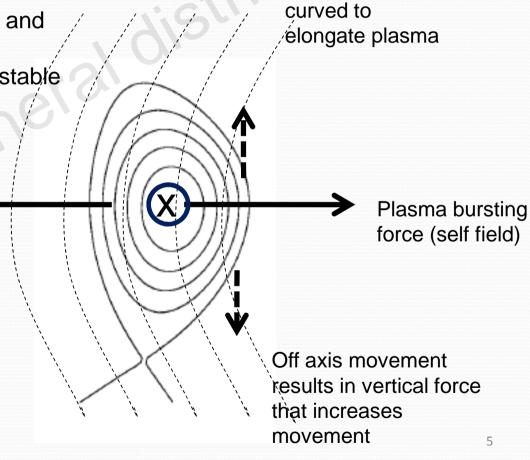
 Circular plasma current loop tends to expand as if under internal pressure. Has to be kept in position by field to push it back

 Diverter shape created by 'pulling' plasma from top and bottom

 BUT elongated tokamak plasmas are inherently unstable in the basic axisymmetric (n=0) solid body mode.

Restoring force

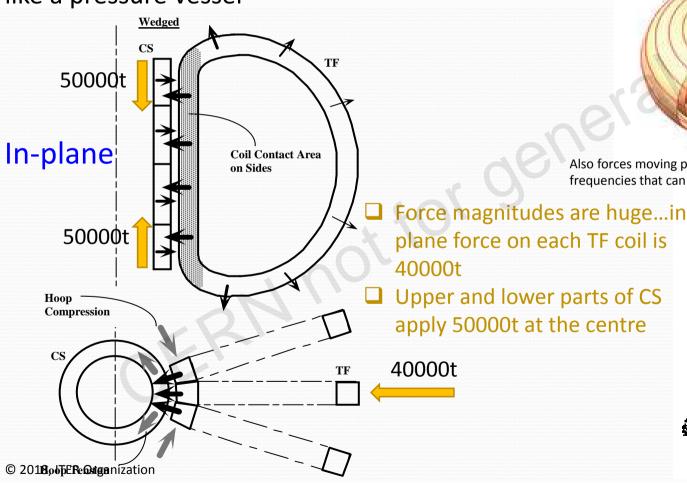
(external field)



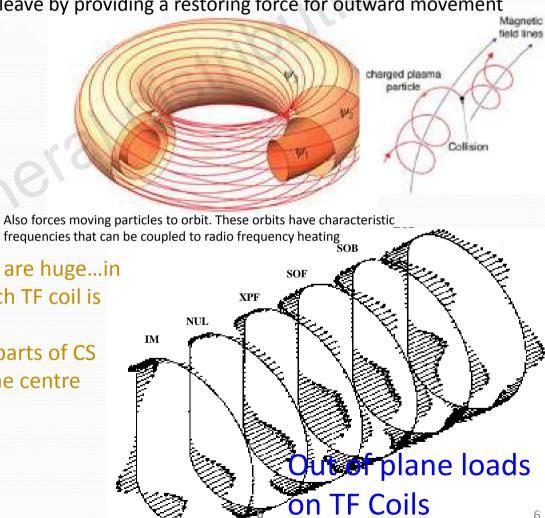
External field

Role of Toroidal Field Coils and Resulting Loads

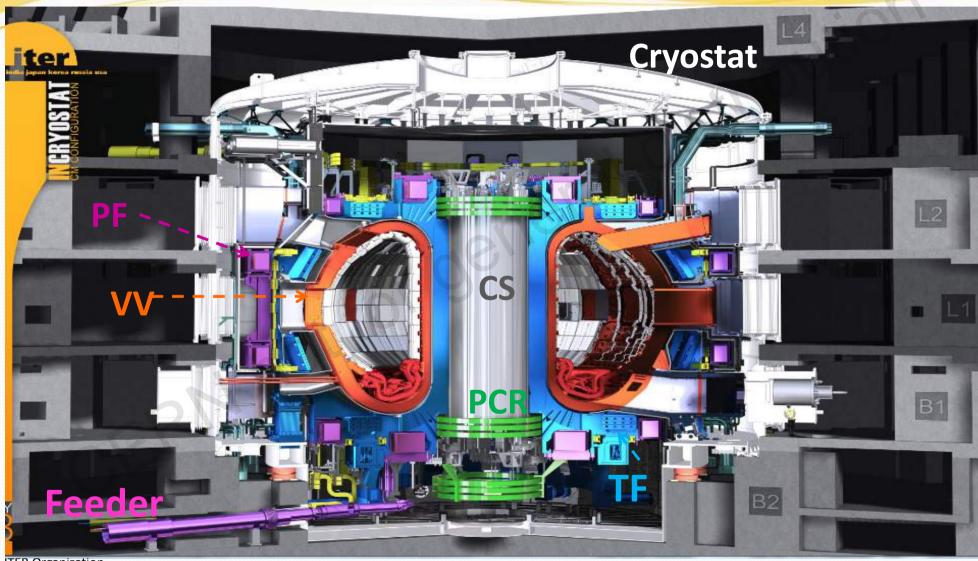
Because of poloidal fields, structures have to react a complex 3D force pattern....not at all like a pressure vessel



Toroidal Field makes it more difficult for charged particles to leave by providing a restoring force for outward movement

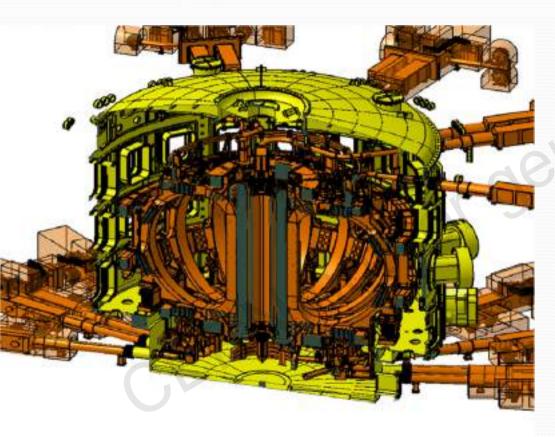


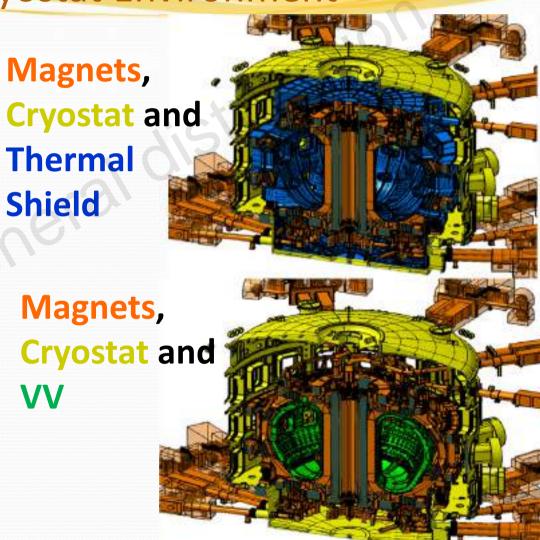
Overall Magnet System and Neighbours



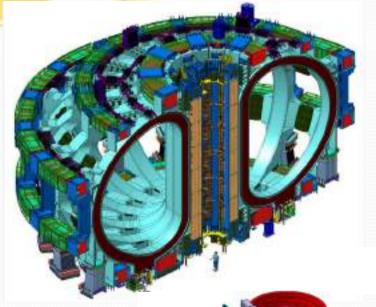
Superconducting Magnet In-Cryostat Environment

Magnets and Cryostat

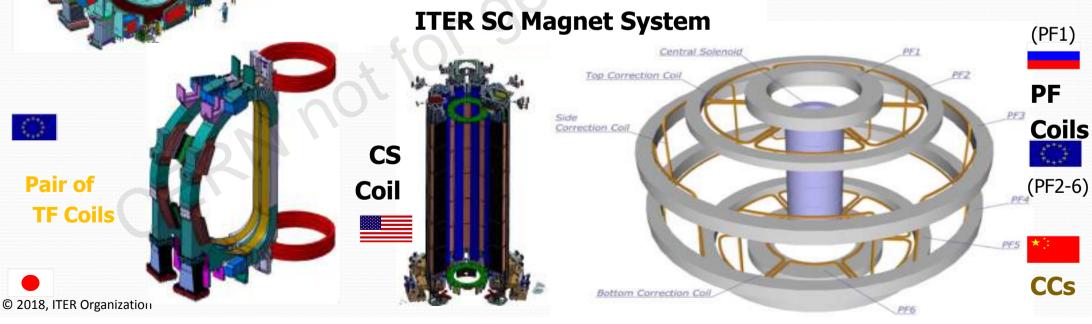




ITER Magnet System - Superconducting Coils

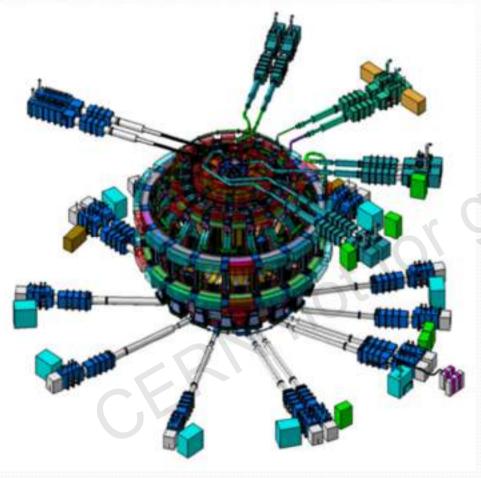


- The ITER sc magnet system is made up of
- 18 Nb₃Sn Toroidal Field (TF) Coils,
- a 6-module Nb₃Sn Central Solenoid (CS),
- 6 Nb-Ti Poloidal Field (PF) Coils,
- 9 Nb-Ti pairs of Correction Coils (CCs).

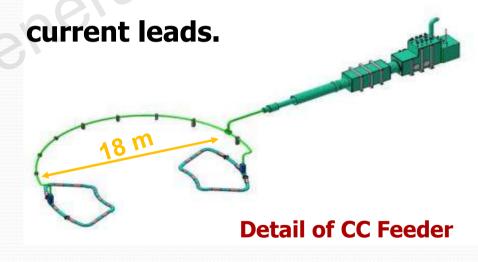


ITER Magnet System – Superconducting Feeders

ITER magnets are supplied with current/cryogenic fluids by 31 Feeders.



- The magnet Feeders include
 - Nb-Ti CICC busbars (MB & CB),
 - Ag-Au(5.4%) BiSCCO 2223 HTS



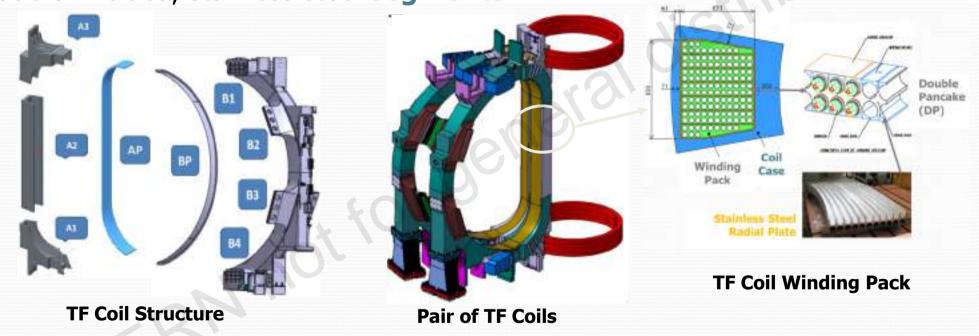






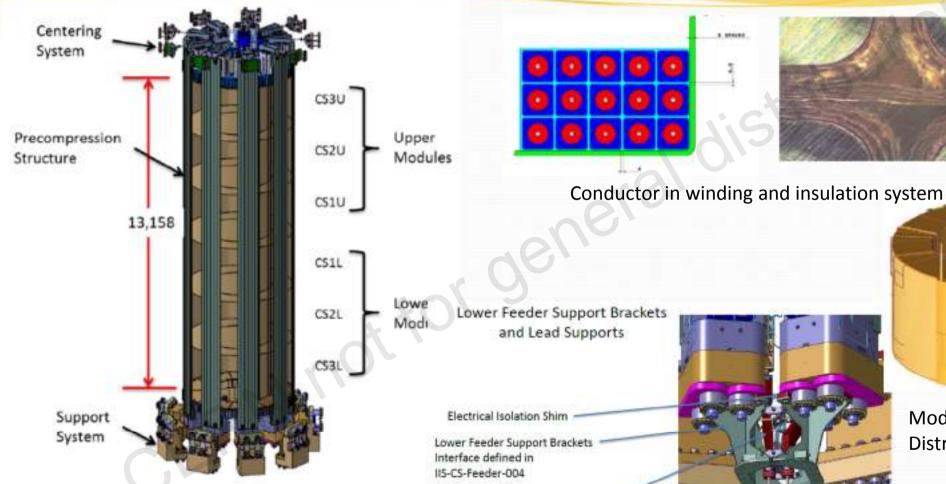
Main Features of ITER TF Coils

• The TF coil is made up of a winding pack **(WP)** inserted inside a thick **coil case** made of welded, stainless steel **segments**.



• Each winding pack (WP) comprises 7 double pancakes (DPs), made up of a radial plate with precisely machined grooves into which the CICC is transferred upon heat treatment completion.

CS Coil Features

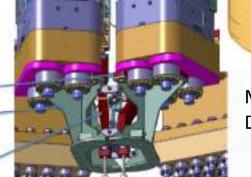


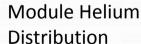
6 hexapancake wound coils using Nb3Sn conductor

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Bus Bar Extension supports

Lower Twin Box Joints Interface defined in IIS-CS-Feeder-001



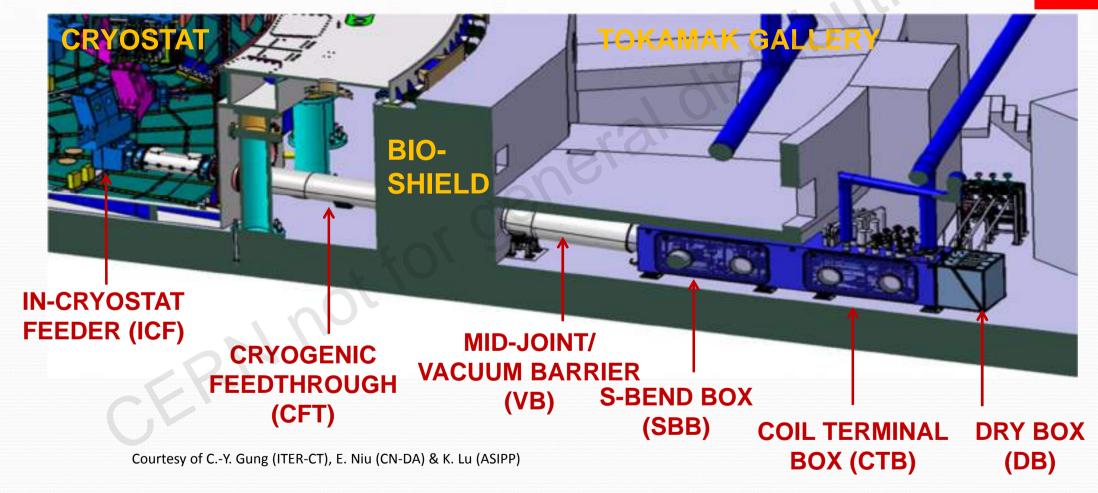


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ITER Magnet Feeder Layout

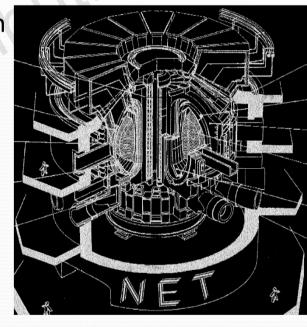
The magnet feeders are deeply integrated into to the tokamak.

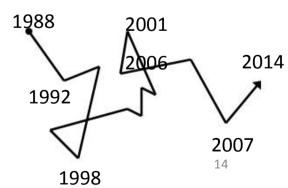




2. ITER Magnet History and Innovations

- ☐ The design roots go back to 1988, the start of the CDA and the NET machine. Present design mostly from 2001 which itself was based on the CDA final design report (1992) which had commonality with the NET project 1988 report (Fusion Technology July 1988--right)
- ☐ Changes, sometimes significant, to surroundings and requirements have created something of a random walk over the last 30 years. We have a design that meets our needs but cannot be said to be optimised. We are where we are....even if sometimes we don't like it
- ☐ The magnet parameters (field & volume) act as the primary drivers for the overall machine size. One of the lessons learned from the history of ITER is that giant oscillations can be created between adventurous (but perhaps unrealistic) innovations that produce large promised cost reductions and more sober (but on-paper more expensive) design realism. Key is to get the right balance





ITER Project Timeline

Lessons Learned?

1982 1988/89 2006-7 1988 1991 1993 1990s 2001-2006 2007-18 1998 1998 1988/1989 1993 2001-2006 2007 1982 Ideas, one co-**Budget ITER EDA Political** constraints, ITER continues. **ITER** agreed. 3 ordinated decision using **INTOR** activity not covered by a "reduced sites. **Agreement INTOR** basis to formal agreement - beginning of design (NET) CDA performance 2001 Final 2006 Director launch ITER among the the project design", US the report. Design abandons Design with the partners. KO, CN entered the development leaves CDA. One Report Review join, US rejoins Conceptual Construction of "next redesigns machine Phase Design machine generation **Activities (CDA)** with tokamaks " hosted at NET options 2006-2018 Various EU, JA, RF 1990s 2002-2005 1989-1991 and US groups ITER CDA, four Successful realization of **Manufacturing** participate **Negotiations** groups participate on how to design, "7 large projects" in parallel representing the industrialisation and build ITER. Inwith design iterations of a qualification major fusion kind supply

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research programs

(EU, JA, RF, US)

single ITER machine concept

during EDA

invented

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Magnet Design: Challenges

Having worked out what the magnets had to do, and argued about the basic components/concepts, we then had to design them

The top engineering innovation issues were (in 1991)

- ■high fields (12-13T) and current densities for industrial scale superconductors
- •tight manufacturing tolerances
- very high voltages for a cryogenic environment
- severe multidirectional loading requirements
- high frequency AC operation
- spectacularly complicated interfaces

Issues with magnets in fusion are the familiar engineering ones that arise when components move out of the research field:

- ➤ Design doesn't quite work as expected (eg structural and electrical fatigue considerations imposed by the lifetime usage)
- ➤ Manufacturing doesn't produce quite was expected, either
- ➤ focus on reliability and repairability suggests both have challenges
- >€€€€ keep increasing.....

LESSON 1 from ITER

How to Plan

INNOVATION? Quantify REALISTICALLY the benefits.....and RISKS

IMPLEMENT:

- a planned, phased development program
- qualification
- industrialization

INNOVATION:

- Requires **DEVELOPMENT**
- Possible implementation of **SOPHISTICATED TECHNOLOGIES**
- Is likely to be CHALLENGING/EXPENSIVE....risk recognition and MITIGATION

Plan and cost adequate testing and TEST FACILITIES



therefore:

- RECOGNIZE the steps required
- **IMPLEMENT** a dynamic responsiveness
- **DIFFERENTIATE** between branch points where a change and additional resources are needed and those where a branch should be cut

LESSON 2

The successful implementation of an innovation

from ITER

EARLY ANALYSIS of the full route through to the implementation



A limitation to one industrial supplier constitutes a high risk and potentially high cost



Most difficult: subjective, difficult to quantify, requires broad engineering knowledge Plagued by short-termism: those who decide are often not those who have to do

At the start, engage multiple suppliers and keep competition. Not only price, also ensures critical reviews

Innovations come with **BENEFITS** and **DISADVANTAGES**



COST of development **AND** implementation

RISK if it fails fully or partially

TIME for development

Increased **PERFORMANCE**

Overall COST saving

3. Industrial Development: Planning & Learning Lessons

Four Key Innovations in ITER Magnets

Insulation Systems (from 1988) Superconductors (from 1987) Structural Metals (from 1991) High Strength Composites (from 1999)

The challenges of these systems had a common theme:

☐ Significant impact on overall machine size and cost if not implemented

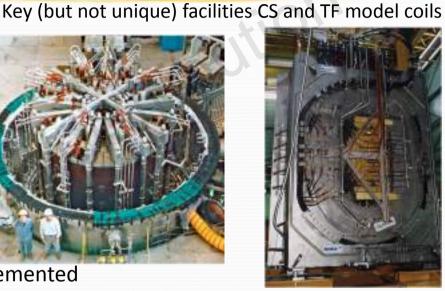


☐ Need to select the R&D targets at levels that are reasonable, promise a cost effective manufacturing route and maintain the positive advantages for the machine.

For each example we can look back and see how the Innovations were Implemented, using more-or-less successful process of learning lessons (....eventually)

With hindsight, it is possible to develop and trace a logic that was not there at the time





Strategy of Conductor Development

Decide strand concept......1987

Develop conductor test facilities 1988-1991 (FENIX then SULTAN)

Decide strand parameters....1991, more or less fixed for 20 years

Nb3Al persisted as R&D activity until 2002, obvious unsuitability after 1996

Consider composite conductor options

Choose conductor concept....more or less fixed for 20 years

Argue about conductor concept for first 10 years

Consider conductor details

Strand stability and copper...fixed 2003

Conductor jacket material....fixed 2003

Cable configuration...fixed 2003, iterate 2006, iterated 2010

Industrialisation (from 2007)

Engage multiple strand suppliers (limited by ITER politics)

Couple strand and cabling (reduce interface), extensive IO support on cabling

Special jacketing lines (ITER politics dictated more than needed)

Continuous Performance Checks

CS conductor problems 2009 => solved 2013 (Crash Program 2)

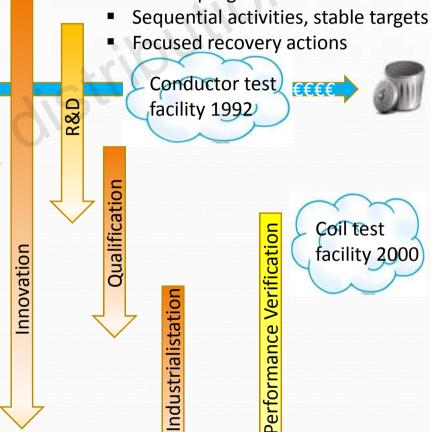
TF conductor problems 2006, adjusted 2008 (Crash Program 1), further issues

2017, solved 2018 (Crash Program 3)

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Classic example of successful innovation.

Overall programme ~600M€

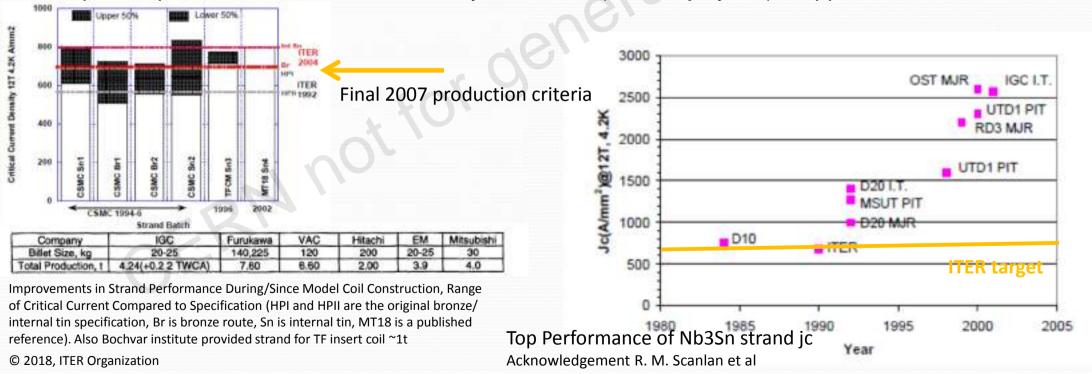


Recovery

Actions and a section s

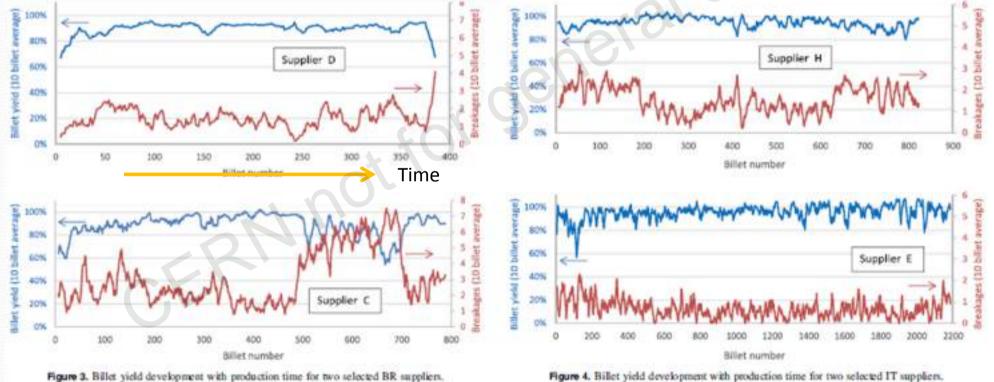
Step I: Base Material Development

- ■In 1987 even basic Nb3Sn strand fabrication was difficult. Few suppliers, low yield, 'individual' strands not standard material. Launched multiple contracts of ~50kg with common target, 4 production routes (jelly roll, bronze, IT, PIT)
- ■ITER target kept well below "technology frontier". ITER model coils gave first steps in industrialisation
- ■7 companies produced a few tonnes each by 1998, one (TWCA, jelly roll) dropped out



- ■Focused on industrial qualities: unit length, wastage, NDT processes and inherent process cost
- ■Contracts >1988 had incentives to improve usability: Larger billets, lower breakage
- ■Unit length increased from a few 100m in 1988 to ~1km in 1993 to >5km in 2008

ITER TF Production (courtesy Alex Vostner). 2 Bronze Suppliers (unit length ~21km) and 2 Internal Tin (unit length 9-13km)



Internal
Tin billet
after restacking

Yield is based on piece length wastage (breakage) not billet usability

Step II: Composite Conductor Selection

- ■Differences in use of basic material (strand). Substantial difference in coil manufacturing (react and wind vs wind and react)
- ■Key element to choice of conductor was current capacity (reduces voltage and/or copper for protection)

■Test facilities for conductor samples were critical: SULTAN was constructed in 1980s and became

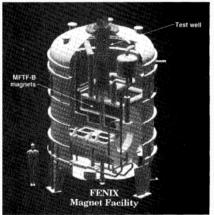
available as split coil test facility in 1992. Still running 2018



Key for testing:
SULTAN facility for
conductor testing
with open end cap
showing conductor
sample hanging
vertically in front,
1990s

FENIX conductor test facility constructed at LLNL 1988-1990

- Used MFTF-B choke coils (Nb3Sn)
- First large test facility
- Very noisy high current power supply limited useful results





ITER TF Conductor Concept selected (by decree) 1993
Significant iterations on the details (Nb3Sn strand quantity, void fraction, twist pitch). For example Nb3Sn strand weight for the TF with temperature margin, standardised to a strain of -0.5%

1998 822t 1K 2001 351t 1K

2004 369t 2.25K

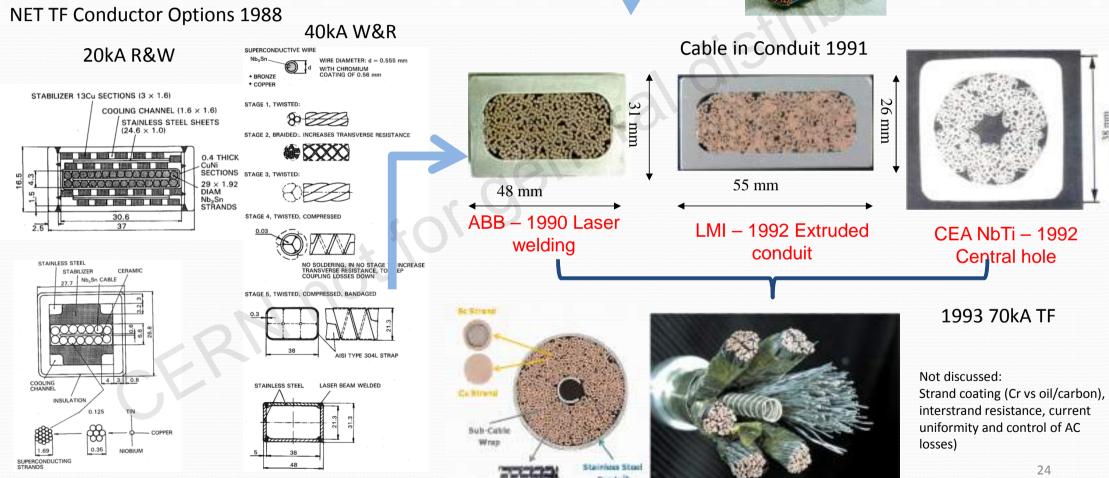
In 2003-4 recovery for degradation implemented Key to 2003-4 changesCu:nonCu ratio

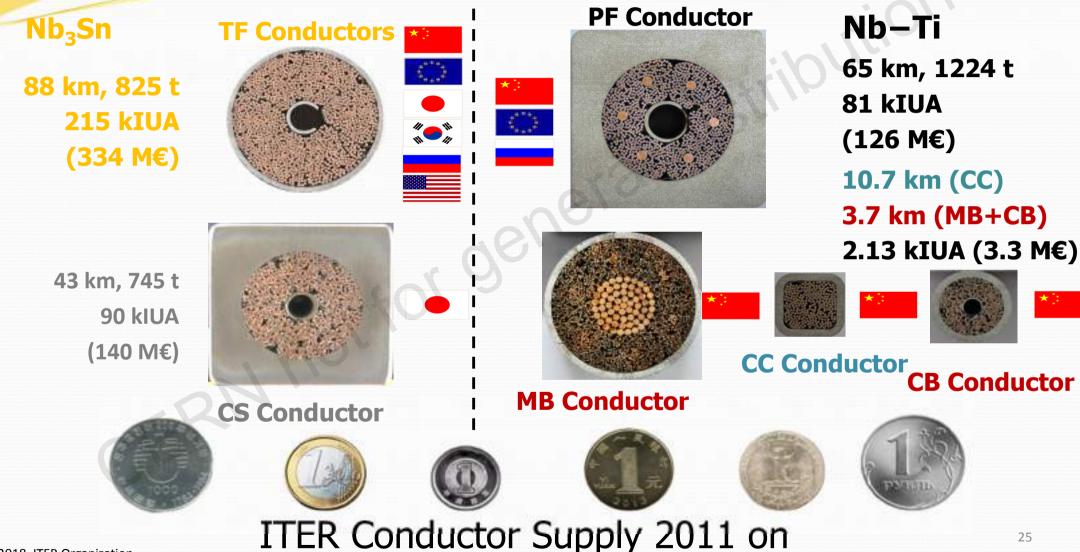
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Ancestry of ITER Composite Conductors



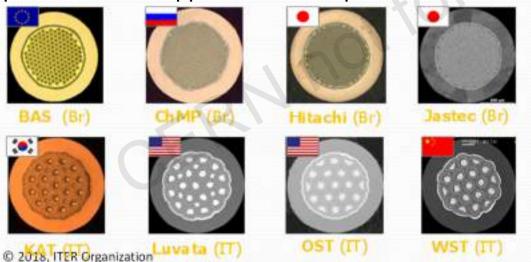
DPC Nb3Sn cable in conduit 1985





Step III: Industrial Base for strand

- ■Even by 2007, minimal industrial base when we started (ITER scale up was about 1 order of magnitude over 4 years in world production)
- Raw material supply (Nb alloys, barriers) an early concern, eventually no issue
- •ITER procurement specification set to encourage multiple suppliers; staged ramp up, repeated gates to demonstrate performance. Support from IO in problem resolution in 2-3 cases





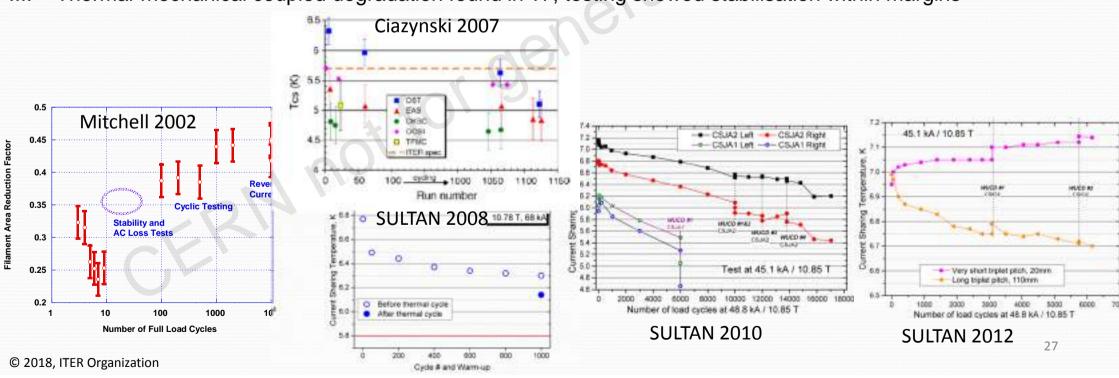


Pre-ITER world production estimated at ~15 t/year; ITER achieved ~100 t/year for ~5yrs.

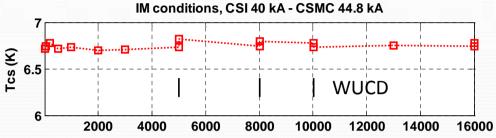
Compiled

Step IV: Recovery Programmes

- ■By necessity (budget, schedule) industrial qualification went in parallel with full integrated performance testing. Result was unexpected issue with filament fracture that had to be addressed three times
- I. Degradation discovered 2002-3. Details of cable design adjust 2003, then 2006-7, issue thought to be solved
- II. Testing of CS conductor 2010 showed issue not solved, cable redesign for CS, too late for TF
- III. Thermal-mechanical coupled degradation found in TF, testing showed stabilisation within margins



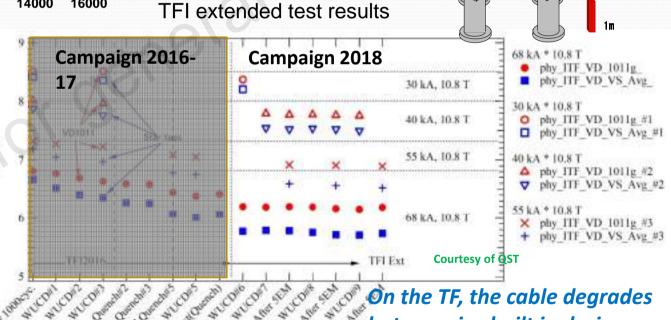
Final Recovery Programs: Second set of Model Coil Tests 2015 and 2018



CS Insert Test



On the CS, a new cable configuration based on a short twist pitch triplet avoids all degradation



but margins built in during

the development allow the

degradation to be absorbed

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Joint

Supporting

Structure

Inner Module Outer Module - Insert

Buffer Spacer

Development of Nb3Sn Superconductors for ITER

1987 2007 2018 1992 1998 2001 1987 1988-1988 1993 2002 2003 2007-1994-91 **NET and MIT** 2015 **ITER CDA New EDA** 2001 Nb3Sn ITER DG start decides DG decides degradation decides CDAcollaboration conductor **ITFR** ~1mm strand/cable issue in Multiple coil Multiple on Nb3Sn strand as concept, conductor Nb3Sn CICC copper concepts, stable conductor strands and circular production base recognised distribution conductor design CICC building CICC and jacket design options composite block material conductors 1989-91 **Construction of first** 1979-85 1993-2002 composite conductor high field test facilities: 2006-08 2017-18 US lead in **CS and TF Model Coil** FENIX (LLNL) and Nb3Sn strands **SULTAN III Projects TF Recovery TF Recovery** through LCT Programme #1 **Programme #3** coil, MFTF-B, 1987-91 **US-DPC** coils: 1995-98 2010-14 Airco & **NET, Kurchatov, MIT,** Teledyne **Incoloy SAGBO**

issue recognised

CS Recovery

Programme #2

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JAERI fabrication of

trial strands

Copper coiled tokamaks built to high voltage requirements on PF system since 1970s

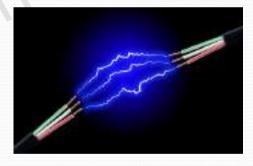
➤ Solid (VPI) glass-epoxy with kapton insulation as standard

- But not in vacuum!
- For example JET ground voltage is 20kV, test voltages about 40kV

Early s/c tokamaks low energy & did not need to address high voltage issue, generally copper coils for pulsed CS/PF and steady s/c for TF (Tore Supra, T-7, T-15).

Now s/c voltage gradually increase

- ➤ITER CS model coil factory tested at 30kV
- >KSTAR tested at 15kV after installation
- ➤ EAST tested at 6kV after installation



Now glass-kapton-epoxy is standard, ITER developed and uses glass-kapton-cyanate ester blend to give improved bonding and radiation resistance

INSULATION TECHNOLOGY AS CRITICAL AS SUPERCONDUCTING

Strategy of Insulation Development

Solid insulation concept & discard pool boiling......1988
Define drivers 1988-1991

Radiation

R&W/I and W/I&R and W&R&I conductor concepts

Base Manufacturing Issues

Viability/Risk of Vacuum Pressure Impregnation on Large Magnets 1991-1998

Voltage Reinforcement (dielectrics) and impact on VPI/bonding 1991-2000

Insulation forming with pre-pregs on feeder conductors 2012-2015

Resin Issues

Radiation Hardness 2002-2008

VPI compatibility 2000-2005

Industrialisation 2005-9: Recovery actions due to:

H&S, pot life, mixing, curing

Detail (from 2010)

Recovery actions in:

Infilling and terminal regions, auxiliary systems

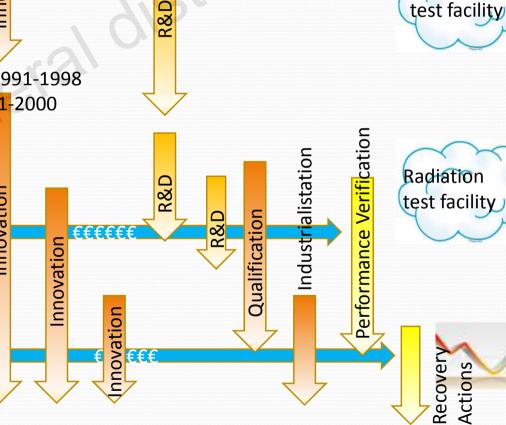
Instrumentation lead outs

Quality verification

Ultimately successful but close links to coil and conductor concepts created several restarts: insulation was considered as a secondary technology..... repeated innovation needs & late industrialisation. Lack of sophistication in early electrical testing

Radiation

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Impact on Insulation of R&W/I and W/I&R and W&R&I conductor concepts

- Early insulation systems <1994 did not integrate dielectric barrier within winding (only as ground reinforcement)
- Relied on stand-off produced by glass filled with epoxy....as long as no cracks
- Glass wrap was compatible with W/I&R coil winding process where the glass went through the Nb3Sn heat treatment
- Despite this from 1988 on TF coil voltages of 20kV to ground and 10kV on terminals were regularly chosen

Present experience that these insulation systems would not have worked. Fortunately we did not build them

From 1993 multilayer insulation (familiar in copper coils) was standard

CONDUCTOR INSULATION SHEME

POLYIMIDE



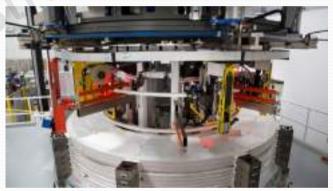
Issues to be addressed are well known and include outgasing of glass to avoid bubbles, resin penetration and cracking. Much more significant in cryogenic coils with thermal cycles and vacuum



Demonstrated on TF MC 1998

R=react W=wind I= insulate

Final selection of W&R&I from 1995





Implemented in ITER 2012=>

Top: CS, Below: TF

Requires controlled handling of (delicate) Nb3Sn reacted conductor

Test Facilities for Irradiation

Required shielding for coil insulation is a key parameter driving the machine build. Establishing limits is difficult

- Irradiation in test reactor is not same spectrum as tokamak
- Big variations in resistance with minor changes in composition
- Impact of degradation difficult to quantify

First facility at Garching (up to mid 1990s)

- Small samples
- Succeeded to carry out irradiation and testing <80K by installing a special facility above the reactor
- Ended when reactor shut down

Second facility at Atom Institute Wien ATI (2001 to 2010) Triga

- Larger samples
- Room temperature only







ATI

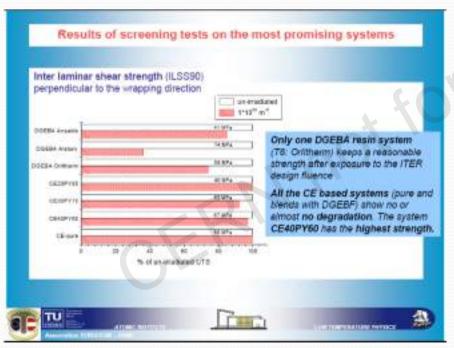
Insulation Irradiation Results

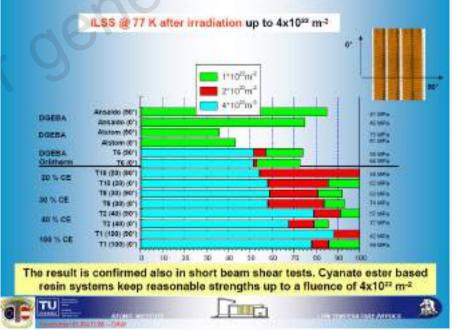
- Up to 2003 all coils impregnated with epoxy resin typically DGEBA
- At ITER fluence level (10MGy or 1*10²² neutrons/m²) marginal
- Cynate ester proposed in 2002 (CDT/TU Wien) as possible improvement
 - Due to cost Cynate Ester Epoxy blend investigated, 40% CE identified as acceptabl up to 4*10²² neutrons/m²

Tensile Tests of Unirradiated and Irradiated ALSTOM ITER Samples



Fracture at 77 K before and after irradiation to fast neutron fluence of 1x 10²² m⁻² (E>0.1 MeV)





Resin Systems

- ☐ Initially (too) focused on radiation resistance
- ☐ Used industrial standard resins and until 2005=> did not look properly at electrical issues

Only from 2009 addressed issues of

- Pot life (time to impregnate large winding at low viscosity before glassification)
- Exothermic curing
- Health issues (and regulation of perceived health risks) on composite chemicals (especially catalysts)
- Mixing and outgassing

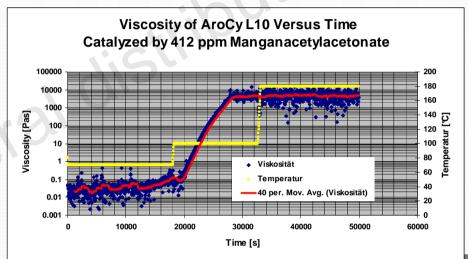
EXAMPLE: Industrialisation of Cyanate Ester blend produced several recovery actions

Cyanate Esters Polymerization Catalysts

- Pot life / speed of reaction strongly depends on catalyst type / concentration
- Catalysts must be added as homogeneous (filtered) solution to avoid any local high catalyst concentrations that could lead to uncontrollable reactions
- Polymerization is a highly exothermic reaction. Safety precautions!
 - ☐ Metal catalysts (typical concentrations 20-300 ppm)
 - ☐ Co, Zn, Mn, Cu ...
 - Soluble organic salts/complexes are used
 e.g. acetylacetonates, octoates, naphtenates
- ☐ Solutions in liquid alkyl phenols © 2018, ITER Organization

Lab-scale thermal runaway of cyanate ester

Pot-life extended in 2009 to more than 100h by exchanging the Mn-catalyst by a Co-catalyst.





Auxiliary Insulation Systems

•Insulation specimens manufactured with pre-preg from different suppliers. Processing conditions optimised. Many iterations to achieve quality

•Pre-preg surface conditions important for bonding and voids

•Pressure/vacuum bag important to reduce void fraction to 2-3%

Silicon wrap to compress during curing



Acknowledgement ASIPP



Some material / process combinations result in insulation with significant voids, leading to poor electrical performance (left)





The final selected materials produce largely void-free specimens (right)

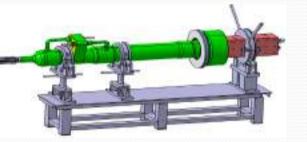




Key Technology: Coil Insulation

Art of applying polyimide

- Inflexible and therefore curved surfaces have to be smoothed
- Complicated patterns of lay-up



- The HTS current leads offer a challenging geometry to wrap due to changes in section and presence of helium pipes at right angles.
- Strategy is to lay up the GK tapes on the cone section.
- Root area of the pipes is first smoothed with green putty before application of the GK tapes.



Feeder Wrapping







Origami style cutting of sheets to fit curves



Principles well known but in ITER (with vacuum) failure to overlap adequately (and cure without resin rich areas) leads to cracks and Paschen failures

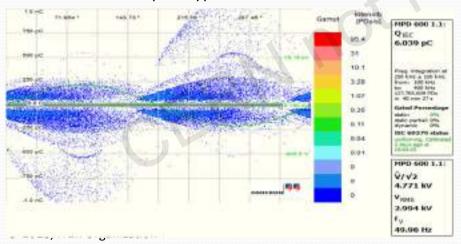


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Key Technology: Coil Insulation

Quality Testing

- ☐ Up to 2008, only HV DC testing
- ☐ From 2008, IO introduced Partial Discharge characterisation and Paschen breakdown testing. Now by far most critical tests, and used for development as well as qualification and production
- ☐ Void fraction measurements also improved
 - Insulation test of current lead
 - > PD test results show stable but relatively high absolute values.
 - Ground screen termination was improved on next prototype.



Paschen Tests on TF Terminal Region (first production coil): yellow arc indicates the breakdown











CS Lead Breakout (mock-up) before discharge (left) and during discharge (right), breakdown at <5kV, 10 mbar

Strategy of Metals Development

Identify areas where structural metals could be improved...1988
Define targets for properties of laboratory development
Innovations in conductor jacket material ...1991

Adopt properties into design 1991

Base design around ideas (and therefore commit to achieving innovations)

Research and development

Gradual descoping of innovations:

Reject all jacket material innovations....fixed 2003

Reject all structural material innovations....fixed 2005

New industrial innovations 1996->

Working/processing of common materials, forging, casting options

Large scale manufacturing trials and industrialisation

Further adjustments to achievable parameters 2008=>

Manufacturing design of jacket material production (extrusion, drawing,

inspection)...recovery actions on low C SS

Manufacturing design of coil structures: innovative forging, welding and

machining

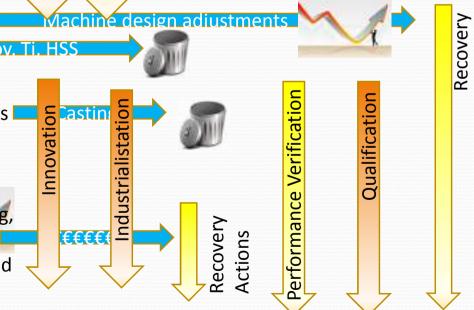
Relaxation of tolerances

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Almost classic example of a programme where early innovations produced nothing. Initial ideas were not thought through and the difficulties of industrialisation by far underestimated. Later innovations ultimately produced benefits, but unrelated to the initial innovations. Machine redesign was required

Actions

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Innovation

- Accuracy and adjustability of magnetic fields critical to plasma performance
- Coil set built for one function must minimise fields that affect other functions
- •Impossible to build and fit everything in ITER magnets to <1mm tolerances.</p>
 Difficult & expensive to achieve <5mm overall</p>
- Difficult to establish coil tolerances (or field accuracy) on many existing tokamaks
- Generally 'a few mm'....regardless of size
- ■Only recently (last 10 years) is field quality (→ tolerances) a design issue

Major effort with ITER coils to identify and minimise <u>critical</u> manufacturing manufacturing/assembly tolerances but NOT to demand unnecessary accuracy

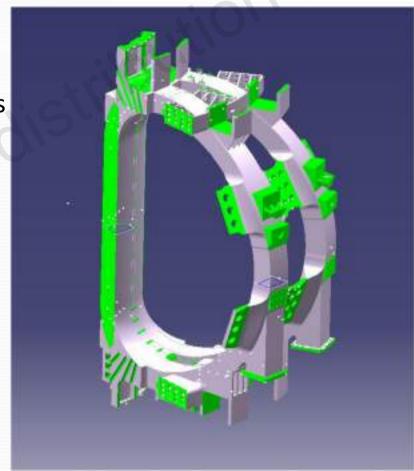
Example of Tolerances: Structures

Where dimensional errors have an impact

- Fitting of components during assembly so that load paths still match design intention
- Inability to place component in available space
- Field errors

What drives tolerances

- Manufacturing requirements/capability typically +/- 1-2mm locally +/-0.5mm
- Installation requirements/capability typically +/- 2mm
- Measurement errors and component deformations under gravity
- Cumulative build up during manufacturing & assembly....
 tolerances depend on other components
- For some interfaces we can adapt to +/-10mm



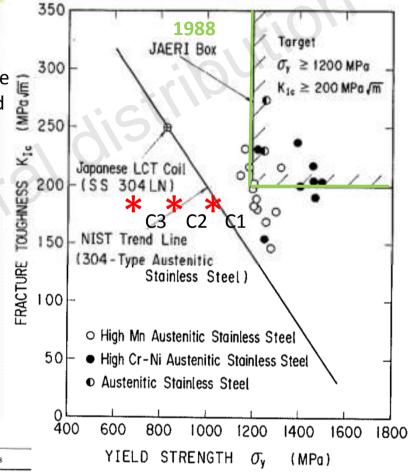
Multiple TF coil interfaces (green)

Base Materials for Structures

- Basic material research launched in 1988 as perception that higher structural metal properties could bring saving in overall machine cost
- Programme launched in JA, EU, RF
- Success claimed in laboratory scale research but universal failure on industrial scale.
- Problems of production of highly compositon specific alloys underestimated
- Issues such as welding, forging, corrosion neglected
- By 2008 only JJ1 remains (TF coil nose) at C1 level and steel properties at same level as obtainable industrially in 1980s

indicates the 3

ITER material grade specifications used in 2009 C1, C2, C3



The relation between fracture toughness and yield strength of the JCS at 4 K. 1988

Table 1. Chemical compositions of the JCS.

JCS	C	Si	Mn	P	S	Ni	\mathbf{Cr}	Mo	N	Others
CSUS-JN1	0,026	0.99	4.2	0.026	0.002	14.74	24,2	_	0.34	
CSUS-JKAI	0.023	0.42	0.49	0.006	0.001	14.0	25.0	0.68	0.268	
CSUS-JN2	0.050	0.34	22.4	0.010	0.002	3,22	13.4	0.70	0.24	V: 0.30
CSUS-JK2	0.05	0.36	21.79	0.013	0.005	4.94	12.82	-	0.212	Cu: 0,70
CSUS-JJ1	0.046	0.44	9.74	0.020	0.002	11.92	12.21	4.89	0,203	

Base Materials for Conductor Jackets I

"Exotics"

Considerations on requirements (in 1991)

- Perception that metal contraction coefficient from 600C to 4K should match that of Nb3Sn to avoid critical current degradation
- The thermal contraction significance in CICC optimisation vastly over-estimated (still seen in new cable development in 2018) leading to incorrect cost impact assessment
- Many other issues drive cable in jacket performance (In particular degradation)
- Environmental issues ignored: corrosion
- Production issues vastly under-estimated but became obvious in period 1998-2002

Candidates Incoloy 908 and Ti. SS was neglected

Corrosion 1

Typical SAGBO cracking in Incoloy 908, in CS Model Coil jacket sections (K. Hamada and JAERI)



Corrosion 2

CS JK2LB conductor samples 2012-13 - corrosion leaks originating from halides present in solder flux accidently contaminating the metal surface

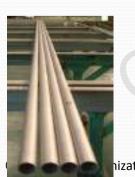


Base Materials for Conductor Jackets

"Conventional"

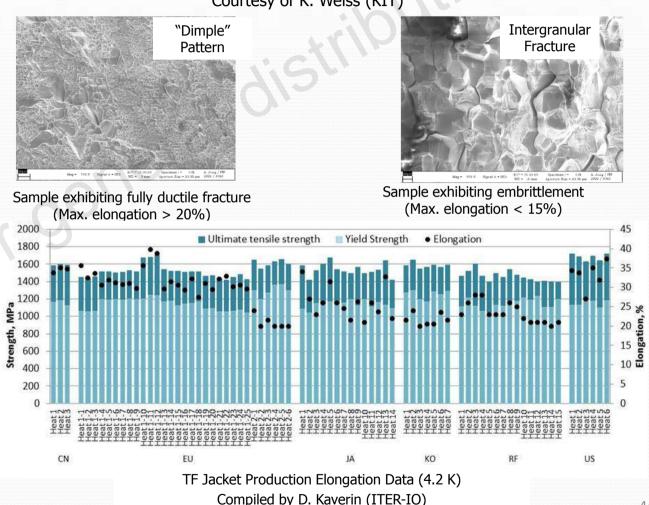
Late development of SS jackets

- Nb3Sn heat treatment leads to carbon precipitation and embrittlement of SS enhanced by cold work of jacket
- For TF needed to develop low carbon steel.
 Worked with industrial partners to optimise production process and control cold working
- For CS JADA continued with JK2LB and eventually achieved success after several material composition adjustments
- JK2LB remains highly sensitive to halogen stress corrosion

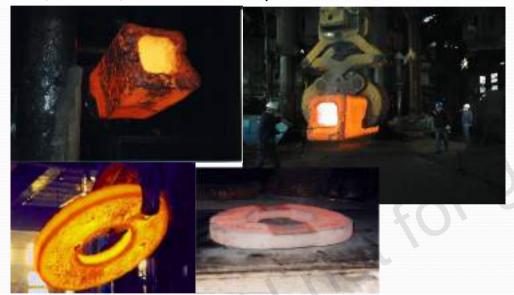


- 4 TF jacket suppliers (1 in EU SMST, 1 in JA KSST, 1 in KO POSCOSS and 1 in CN JIULI) have been qualified and produced tubes for all 6 DAs.
- •Tubes extruded in ~12m lengths and butt welded

Tensile Tests at Low Temp. (< 7K)
Courtesy of K. Weiss (KIT)



1996-2000 Various forged sub-sections of the ITER TF coil case, showing the complexity of the forged forms. Top: seamless TF case, bottom, seamless radial plate for TFMC



Trials on TF Structures: curved hollow section of coil case. Ultimately too complex but the know-how obtained by the company (Kind) was used to produce almost all the forgings for the TF coil cases and VV under contracts with EU, KO and JA

Trial Casting of Components: rejected because of poor properties (low modulus, low strength) Forging Challenges: Size (for CS tie plate, longer than reheat furnace), shape complexity to reduce machining, narrow temperature window for forging high strength steel



2015-16 Offset forging of a 12m CS tie plate





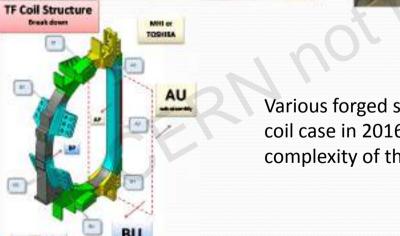




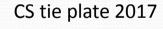








Various forged sub-sections of the ITER TF coil case in 2016-17, showing the complexity of the forged forms.





Strategy of Use

Decide on Pre-compression ring concept......2000...too vague Decide performance parameters....overestimate

Consider options

Choose winding concept.....wet winding of monofilament glass...2002 No industrial input, attractive as subsize tests can be performed

R&D and qualification

Change winding concept...VPI of monofilament glass...2004-5 Construct 1/5 scale test facility 2005-6 Wind 1/5 scale samples in laboratory, successful test 2006-7

Industrialisation (from 2009)

No industrial suppliers prepared to offer full scale monofilament ring 2011

Change concept to AFP, new process, 2012

Process not down-scalable to 1/5 test facility, go to full scale

AFP manufacturing issues 2015

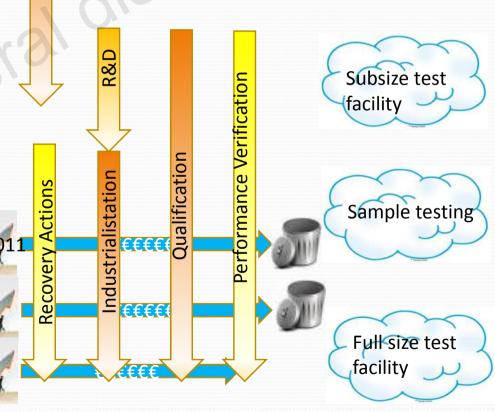
Change main line concept to pultruded process, 2017

Construct full scale test facility 2017

Full size pultruded ring test end 2018 (?)

Classic case of poor implementation of innovation

- Industrialisation too late
- Cost saving on test facilities
- Everything ended up in parallel
- Ultimately seems successful



nnovation

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4

Critical component introduced in ITER in 1999-2001 to preload TF coils and compress shear keys

Relies on specific properties of strength, modulus and thermal contraction.

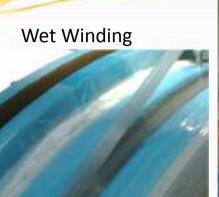
Practically only one solution: structure made with glass fibre

Classic example of a high risk innovation: limited opportunity for risk mitigation if it does not work, limited possibilities to find alternatives

Cost saving did not allow proper risk mitigation. Too small testing, very late industrial involvement. In the end cost more than a structured programme from the start

Classic example of a (too) late recovery plan based on R&D to produce a 'Plan B' that seems successful linked to implementation of full testing









Ring before



1/5 scale test facility 2009

1/5 scale test facility



and

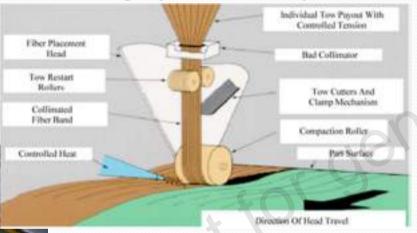
AFP Automated Filament Placement Method

Fibers' Wrinkles: waves on fibers with pitch of few mm

• Fibers' Undulations: waves slightly taller and with pitch of

tens of mm

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Full diameter prototype ring after curing, showing wrinkles

Pultruded Route

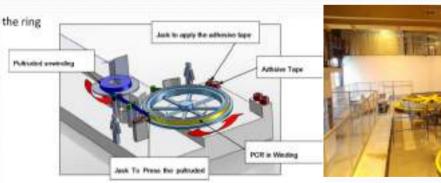
Full size 1/3 thickness prototype July 2018



Full size test facility Jul 2018



Winding line completed

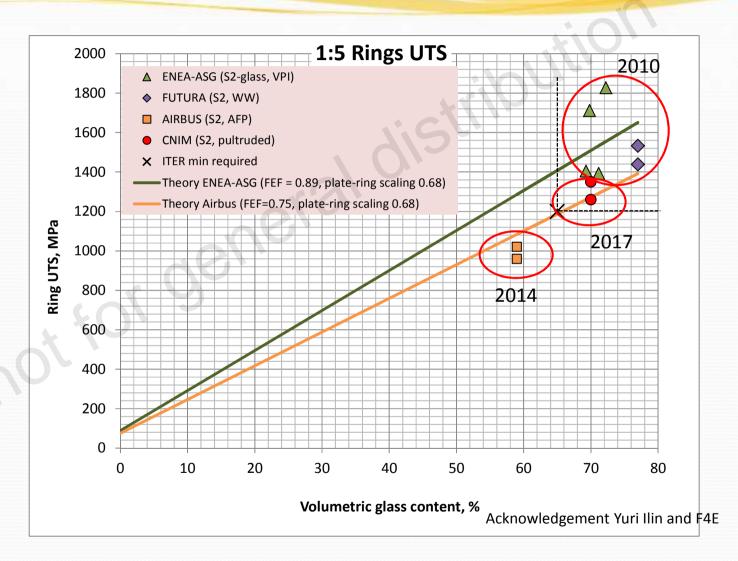


Acknowledgement F4E, A. Bonito-Oliva, T. Boutboule, CASA and CNIM

Summary of recovery actions

Tests of 1/5 scale rings to failure

UTS= ultimate tensile strength



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What Lessons were learned?

Innovation will happen at many levels in a project.... great ideas everywhere

ITER top level: Series of innovations in tokamak concepts provoked big oscillations in the technological development Medium level: 4 innovations were ultimately successful but followed paths of highly variable roughness

What Lessons do we expect to learn from all this? We are not going to repeat exactly the ITER experience

Generalise the specific examples to write rules/guidelines on how to start a basic research project in an international environment (which in the present world is omnipresent)...essentially try to carry out a root cause analysis

- 1. Always carry out a FMEA assessment based on the full, partial or complete failure of a great idea. A risk analysis is not the same....failure is black and white, risk is shades of grey. A FMEA should identify the necessary escape routes...
- 2. When assessing a proposed innovation, confirm that the innovation is really the key issue in the area it affects. Good example here is the structural materials....laboratory scale R&D created a series of red herrings that took years to straighten out. If it is not, what is? And can it be improved?
- 3. Successful engineering implementation is much more difficult than a small R&D programme. If an innovation is selected, plan, plan, plan! You need industrial suppliers and (while recognising the key part industrial collaboration can play) never ever create a monopoly supplier situation
- 4. Successful innovation is a long grind, with many forks. You cannot follow them all, but nor can you cut off the branches too early. Plan frequent reviews, adapt, ensure wide input, get in early with recovery plans, and ruthlessly discard when the end of the road seems to be reached.

3. Brief Status of the ITER Magnets

Manufacturing Status (July 2018)

Very approximate overview

Conductors: 99% complete

TF Coil Windings: 60% complete

TF Structures: 50% complete

PF Coils: 25% complete

Feeders: 25% complete

Supports: 60% complete

CS coils: 50% complete

Worksite Progress: Feb. 2015 – July 2018

Feb 15 July 18





Nov 17





April 18





© 20 Morre, than halfway to First Plasma:

Assembly Hall





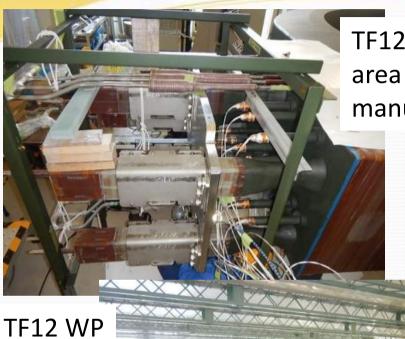
Before being integrated in the machine, the components will be prepared and pre-assembled in this 6,000 m2, 60-metre high building.

The Assembly Hall is equipped with a double overhead travelling crane with a total lifting capacity of 1,500 tons.

To the right, the installation of the first sub-assembly tool (SSAT-1) is nearing completion

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JA TF coils – Manufacturing progress at MHI Kobe

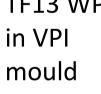


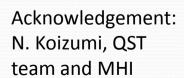
TF12 terminal area (under manufacturing



TF13 DP joint assembly

TF13 WP

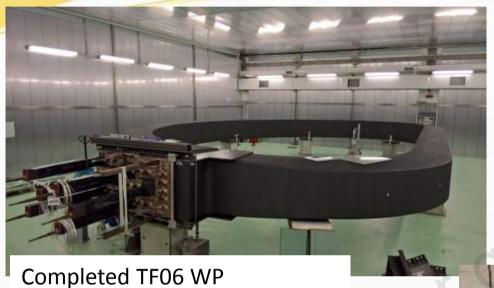




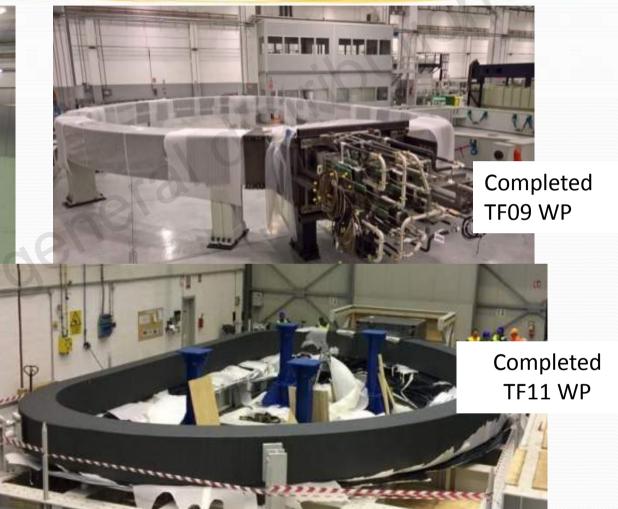


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EU TF coils – Progress at ASG La Spezia



Of the 10 TF EU Winding Packs as of Sept 2018: stacking complete for 8, vacuum impregnation completed for 4, and 3 WP finally completed and delivered to the insertion manufacturer, 2 successfully cold tested.



Acknowledgement F4E and ASG

TF coils - Production Status

TF STATUS SEP 2018	EU	JA
DP Winding	70/70	43/63
DP Heat Treatment	70/70	43/63
DP Transfer, Turn Insulation, Welded	66/70	33/63
DP Completed	56/70	25/63
WP Insulation	6/10	3/9
WP Impregnated	5/10	2/9
WP Terminal Region Assembled	4/10	2/9
Winding Pack Cold Cycled	2/10	1/9

(Number presently produced/Total number to be produced)

TF Coil Structure BU-AU fitting tests at MHI Japan & HHI Korea





Fitting achieved to better than 0.5mm at interfaces

Left: TF09 fitting, HHI, January 2018

Right: TF12 fitting, MHI, Aug 2018

Case-WP Insertion Assembly Rig at SIMIC

- The 1st Assembly Rig has been tested and commissioned in 2017
- Results showed that the parts can be moved with a precision better that
 0.2mm
- First set of structures has undergone a trial fitting in July 2018
- First winding pack insertion starts in September 2018



Supports at various locations in China

- Lots of PF clamps in various stages of completion at HTXL China
- TF GS complete and components for 3-4 more near completion



Water jet machining PF5 clamps

TF GS plates



Behind: PF clamps (left), TF GS plates (right), PF6 plates and blanket shielding blocks (front)



Balancing a PF5 clamp



First complete TF Gravity Support (July 2017)

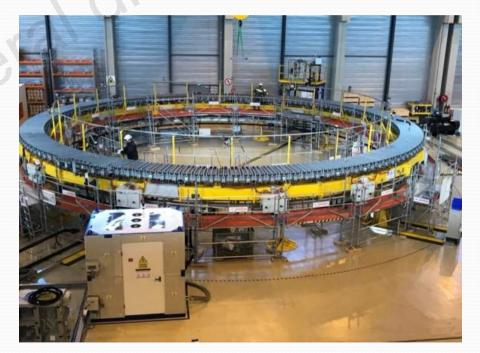
Acknowledgement CNDA, SWIP & HTXL

PF5 Status (ITER Site)

- Five PF5 real Double Pancakes (DPs) windings have been completed. The third series production DP6 has been started.
- First VPI on DP7 finished. VPI on DP8 and DP6 is on-going.



PF5: 2nd DP(DP8) Winding Completed



PF5: Dummy DP VPI Under Preparation

PF6 Status in Hefei, China

- All DPs windings have been completed.
- Completed resin impregnation (VPI) for six DPs.
- Coil stacking underway





Acknowledgement F4E & ASIPP © 2018, ITER Organization

CS Coil at GA San Diego

Left: CS module 1 on turn insulation station after heat treatment. Right: Module 1 entering mold (Aug 18). Resin impregnation in September 2018

Acknowledgement US-IPO, GA)

Winding of 4nd module is completed; winding of 5th module has started.

Modules 1, 2 & 3 are heat treated

