

Neutrons for nuclear technology: experiments

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ARIEL-H2020 International on-line school on nuclear data: the path from the detector to the reactor calculation – NuDataPath - 2022

Accelerator and Research reactor Infrastructures for Education and Learning

ARIEL



Disclaimer:

I assume:

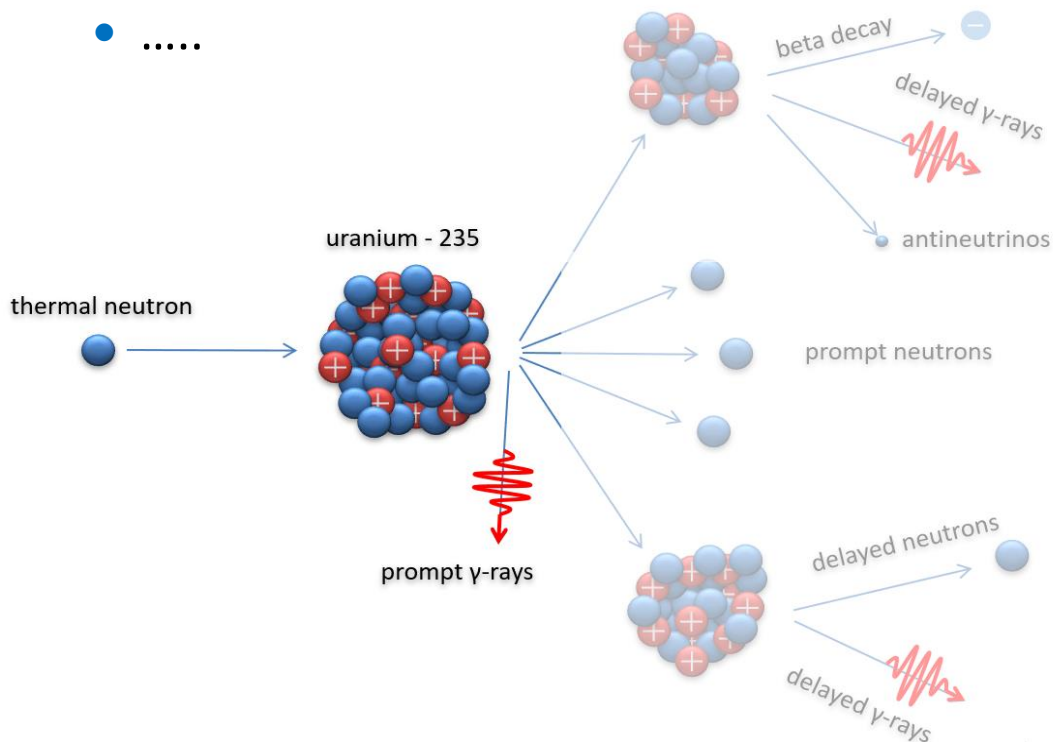
- You know now how neutron detectors work
- For other particles, at least you know what are scintillators, gas detectors, semiconductors,...
- You know the principle of time-of-flight experiments
- You understand why the study of neutron reactions is important for Nucl. Tech.
- You know the role of the different reactions and neutron energies

Now let's "just" see a few examples of different type of measurements at some of the facilities that were mentioned yesterday .

Reactions and observables

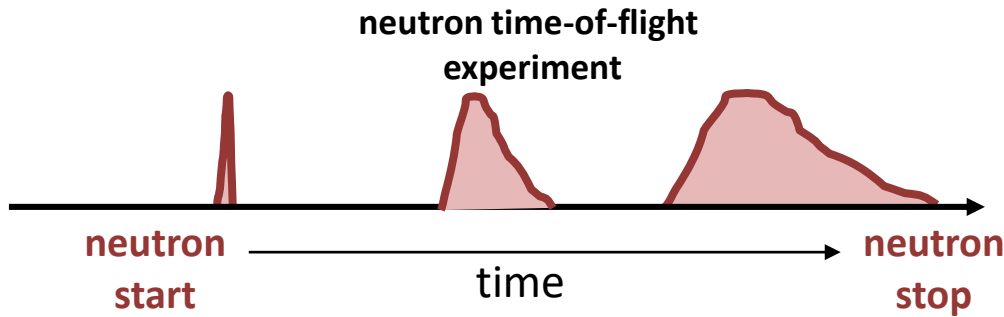
We need to detect different types of particles:

- g-rays from neutron neutron capture/fission: $(n,g)/(n,f)$
- g-rays from inelastic reactions: (n,n') , (n,xn) , ...
- Light charged particles from (n,lcp) reactions: (n,p) , (n,a) , ...
- Heavy charged nuclei from fission: (n,f)
- Neutrons from many reactions: (n,f) , (n,n) , (n,n') , (n,xn)
-



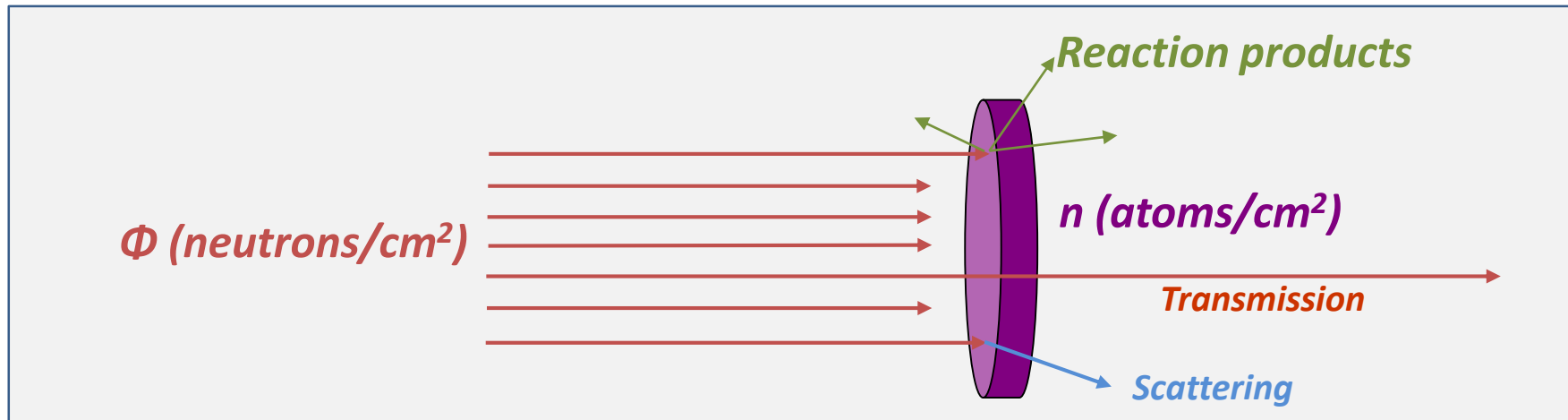
The types of **particles**, **processes**, **energies** and **observables** of interest are so different that their study requires a large variety of detector types and detection arrays.

The time-of-flight technique (again)



Time-of-Flight to E_n relation (non-rel.):

$$ToF = \frac{L}{v} \propto \frac{L}{\sqrt{E_n}}$$

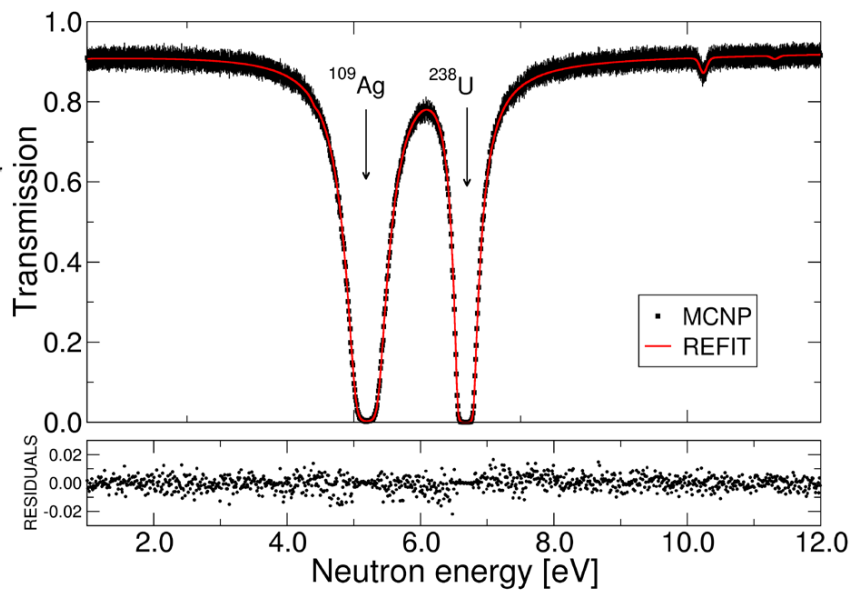
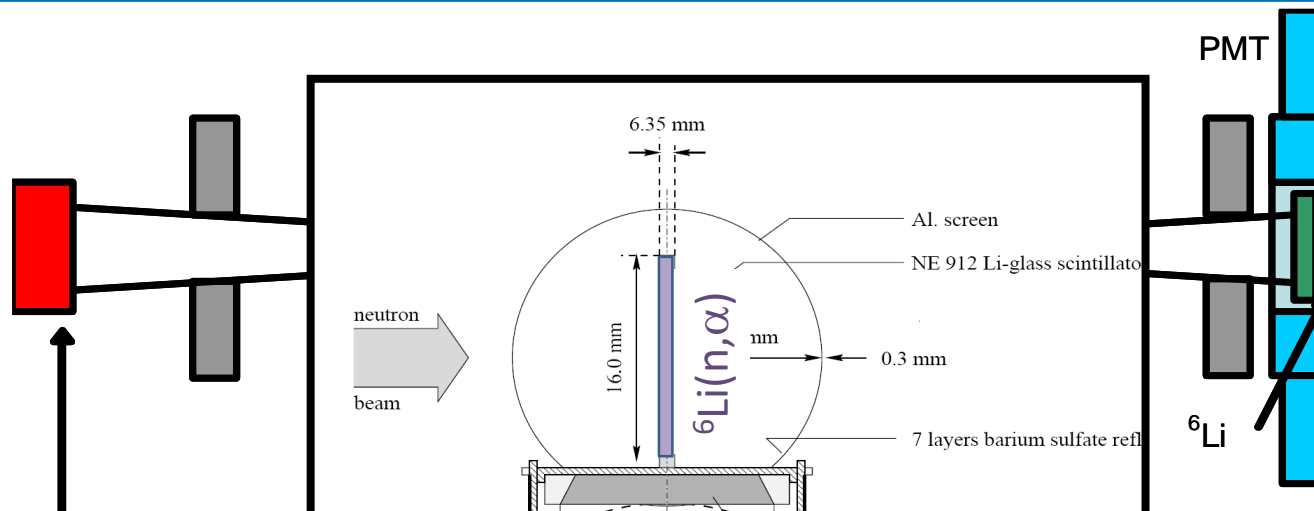


Transmission probability: $T = e^{-n\sigma_{n,tot}}$

Reaction probability (Yield): $Y_{n,x} = (1 - e^{-n\sigma_{n,tot}}) \frac{\sigma_{n,x}}{\sigma_{n,tot}}$ (1st order approx.)

Measuring transmission: total cross sections (n_{tot})

Transmission measurement

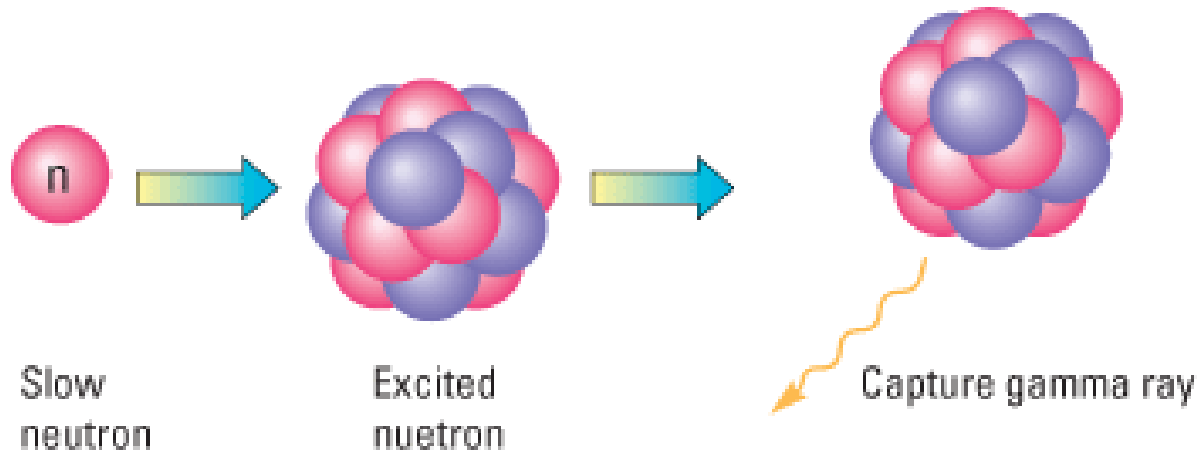


$$T = \frac{C_{in}}{C_{out}} \propto e^{-n \sigma_{tot}}$$

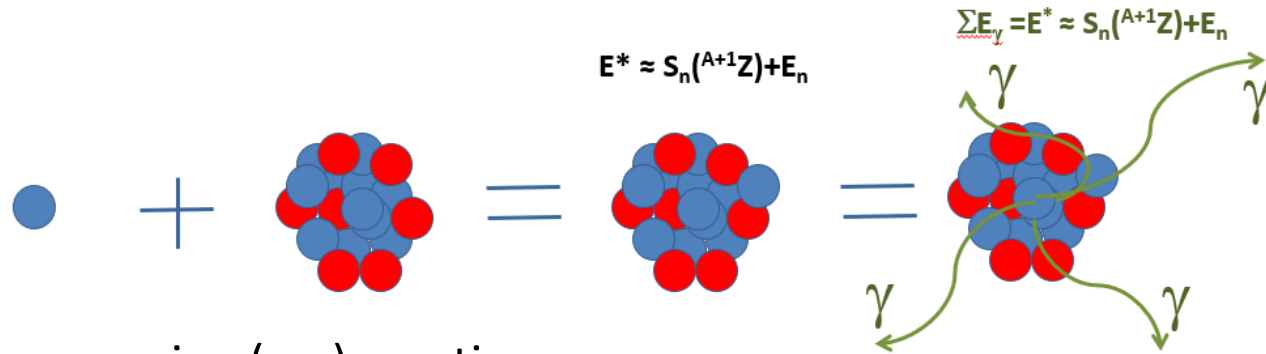
It may seem simple but:

- Background needs to be measured carefully (black resonances)
- Sample inhomogeneities important (powder's grain size)
- Thick targets needed (difficult for radiact.)
- Resolution broadening to be considered
-

Measuring capture (n,γ)



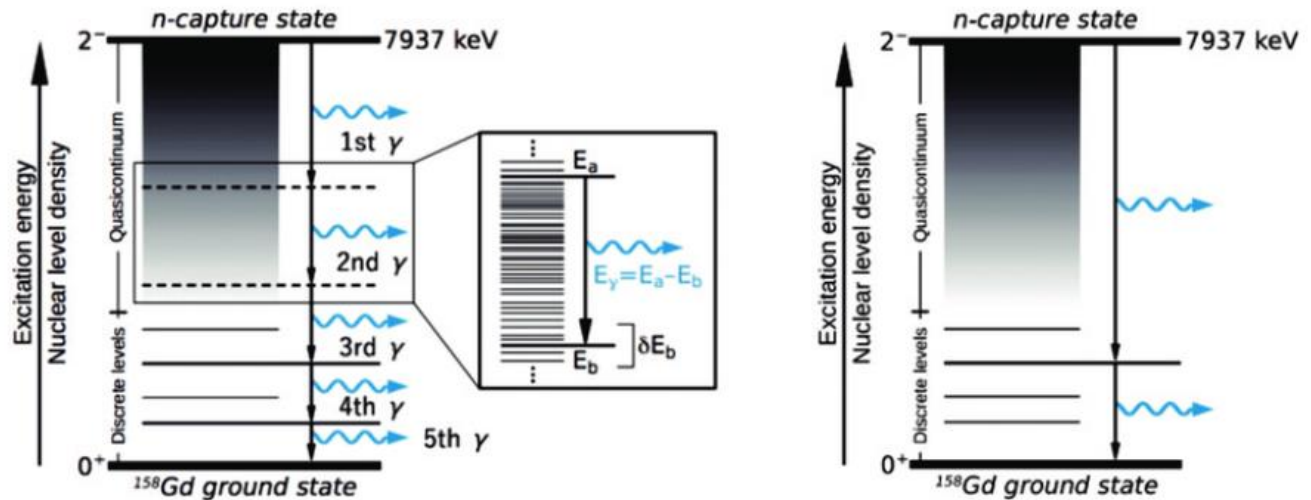
Observables in neutron capture



Strategies for measuring (n,g) reactions:

- Detect a **“characteristic”** g-ray from the cascade (PGA, prompt gamma analysis)
- Detect **only one** g-ray of the cascade (TED, Total Energy Detectors)
- Detect **all** (ideally) g-rays of the cascade (Calorimetry)

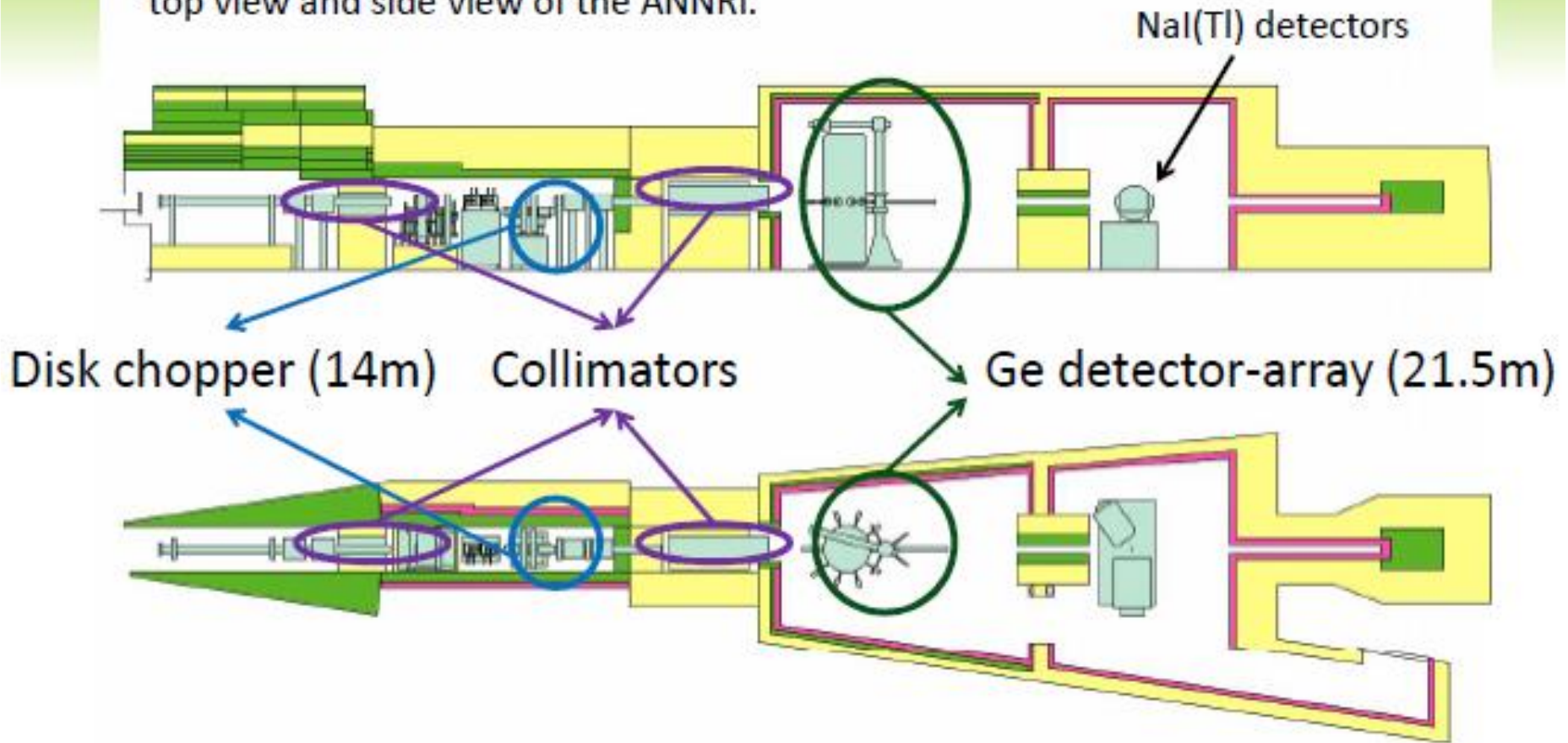
Main challenge:



PGA: Detecting a “characteristic” γ -ray (I)

- ANNRI @ J-PARC (Japan)

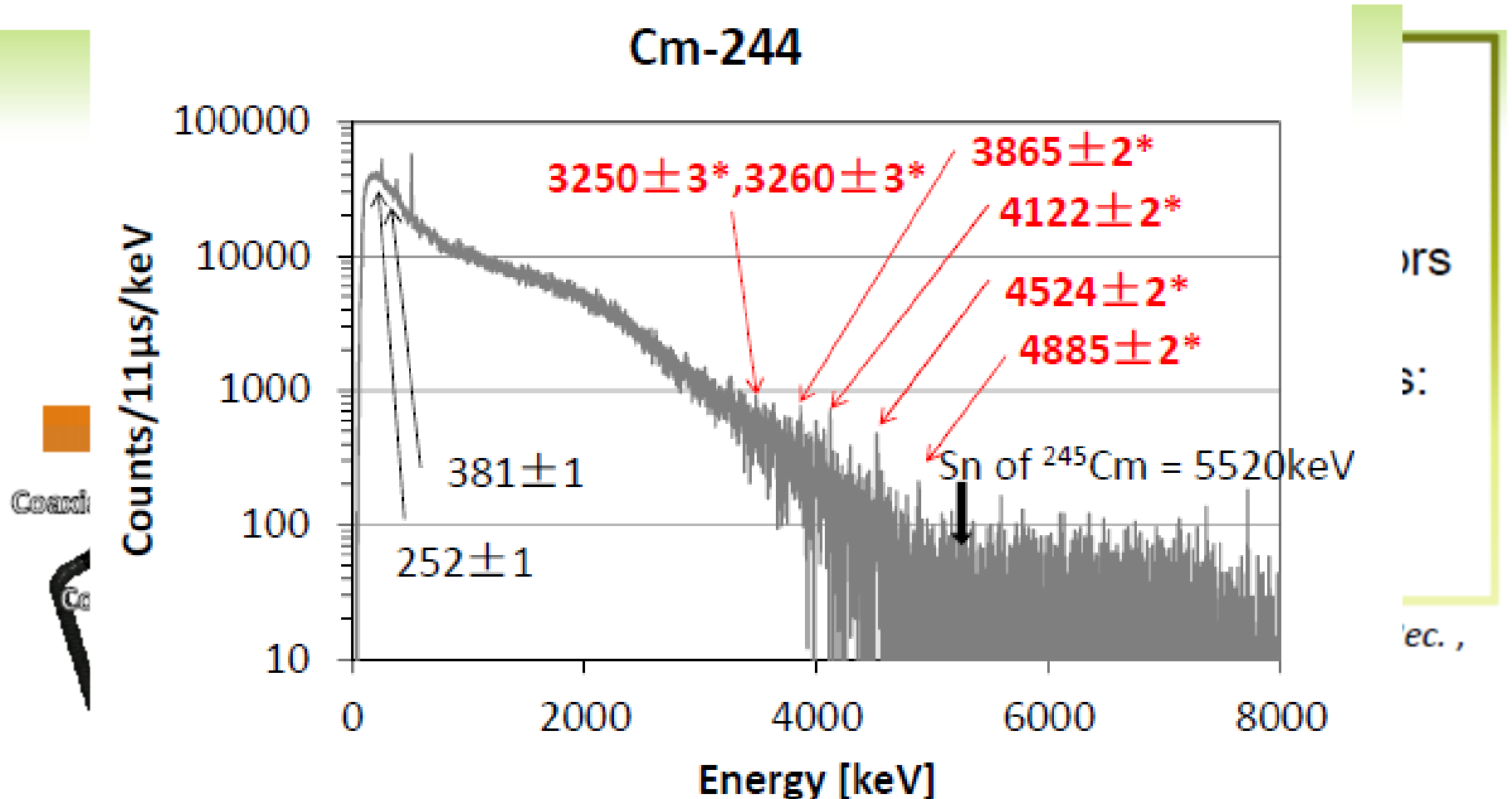
top view and side view of the ANNRI.



PGA: Detecting a “characteristic” γ -ray (II)

Limitation in accuracy:

One needs to know the “intensity” of the characteristic(s) γ -ray line(s)



Brief TED: why detecting a “single” γ -ray?

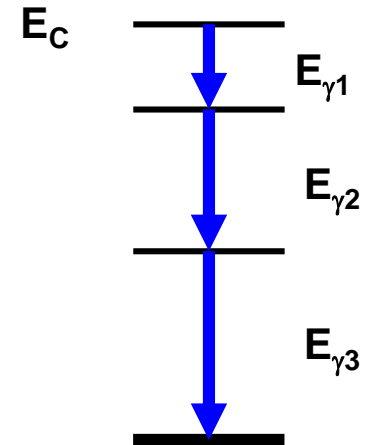
Every decay cascades is different,

then the efficiency to detect each one is different,

how to measure then?

if $\varepsilon_{\gamma i}$: total efficiency for γ -ray of energy $E_{\gamma i}$

then: Total efficiency of the cascade : $\varepsilon_c = 1 - \prod(1 - \varepsilon_{\gamma i})$

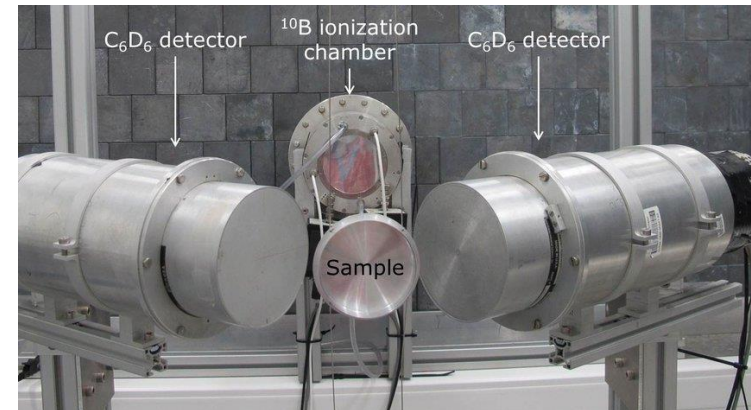


But if ($\varepsilon_{\gamma i} \ll 1$) then $\varepsilon_c = \sum \varepsilon_{\gamma i}$

Total Energy Detector (TED)

Now, if we had a detector for which $\varepsilon_{\gamma i} = kE_{\gamma i}$

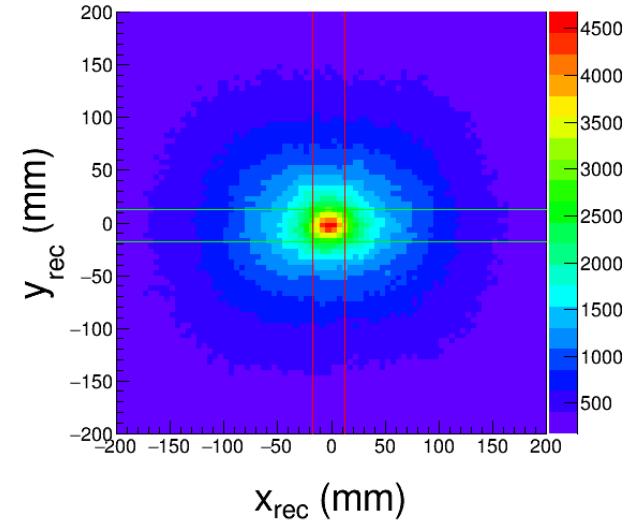
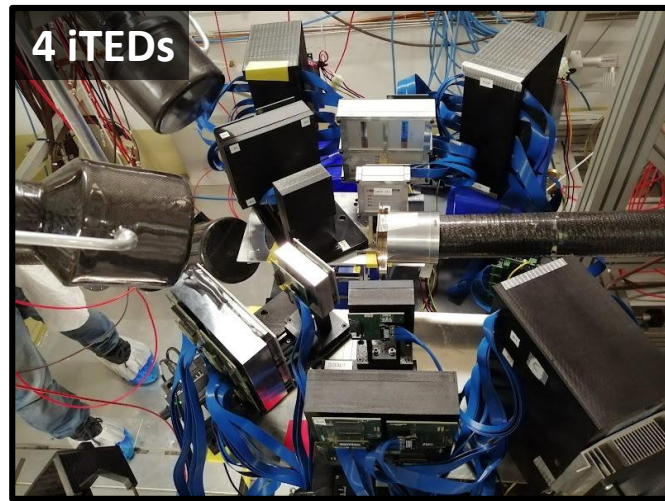
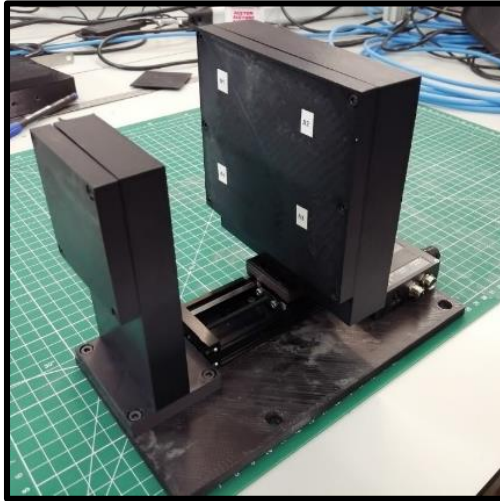
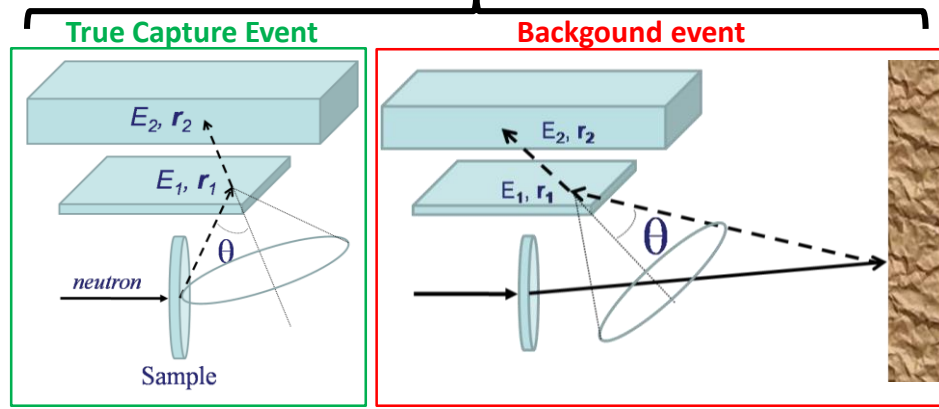
then $\varepsilon_c = \sum \varepsilon_{\gamma i} = \sum kE_{\gamma i} = k \sum E_{\gamma i} = kE_c$



This type of detector can be constructed (Moxon-Rae detectors),
but the proportionality is only approximate => **MC used instead**

The latest: TED + Imaging

GAMMA IMAGING

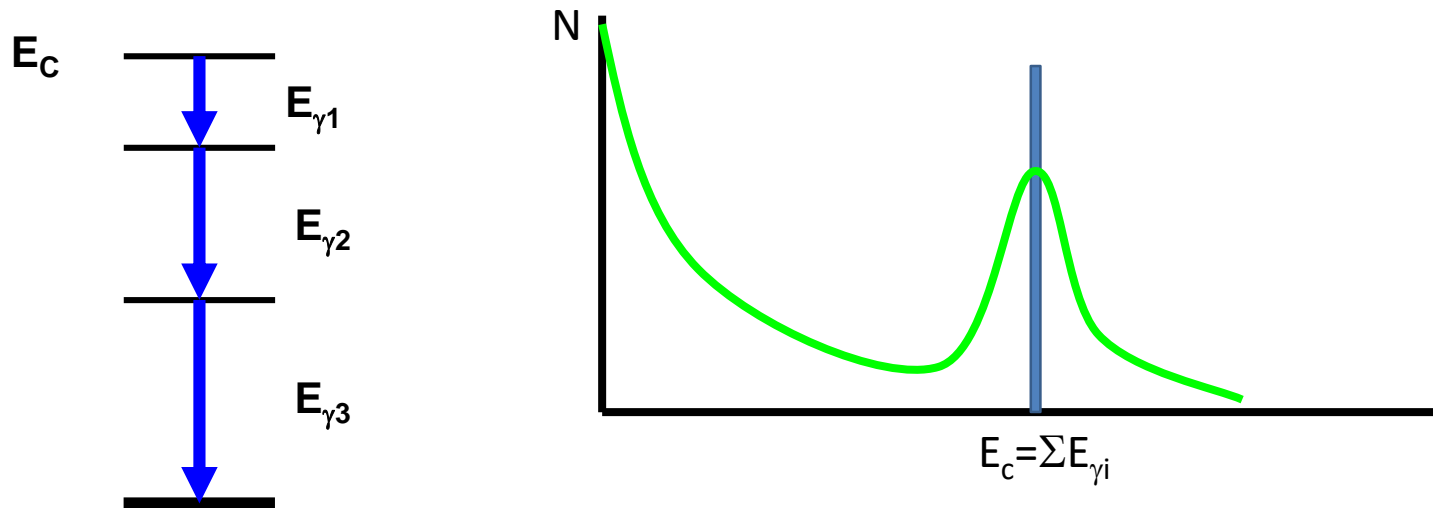


Total Absorption technique

If intrinsic and solid angle efficiencies are large:

- Total efficiency of the cascade : $\varepsilon_c = 1 - \prod(1 - \varepsilon_{\gamma i})$
- Peak efficiency : $\varepsilon_c^p = \prod \varepsilon_{\gamma i}^p$

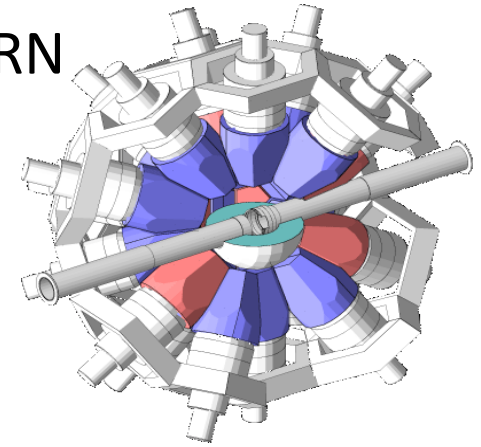
If $\varepsilon_{\gamma i}^p = \varepsilon_{\gamma i} = 1$ THEN $\varepsilon_c = \varepsilon_c^p = 1$



(n,g) calorimeters worldwide

n_TOF Total Absorption Calorimeter (TAC) at CERN

40 BaF₂ crystals covering 4 π
(based on the original at FZK)



Similar one at the Back-n at CSNS

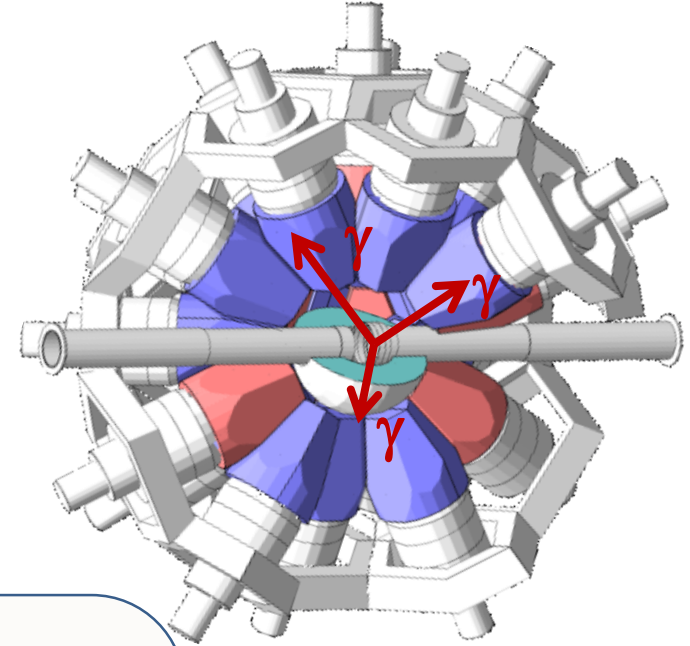
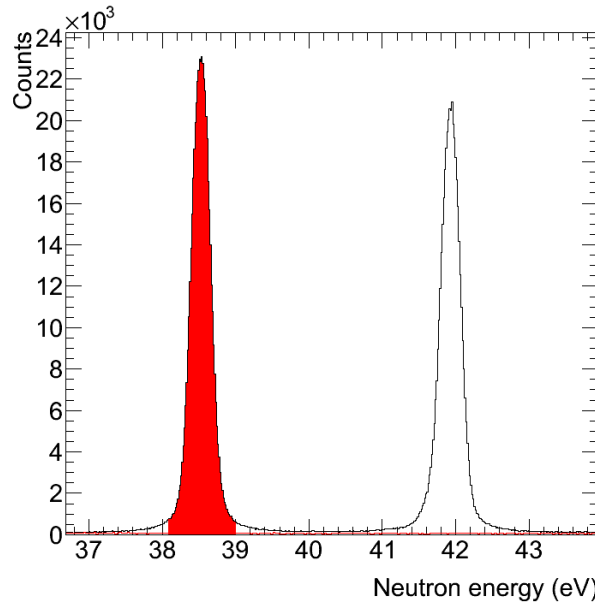
DANCE (Detector for Advance Neutron Capture Experiments) at Los Alamos (USA)

162 BaF₂ crystals covering 4 π

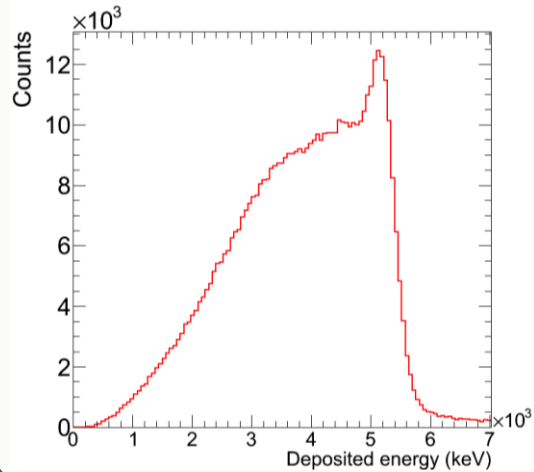


Future: a lot going on within the n_TOF collaboration

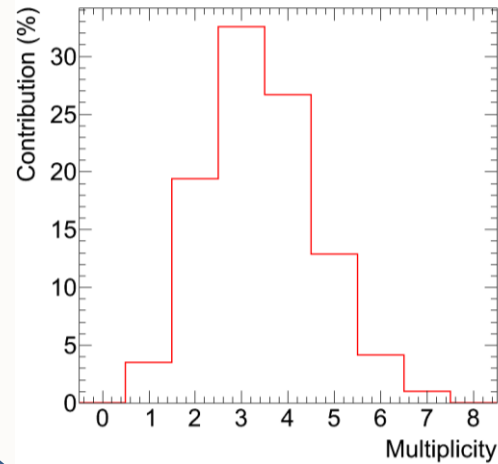
Brief TAC: observables (E_n , E_{sum} , m_{cr})



Deposited energy



Multiplicity

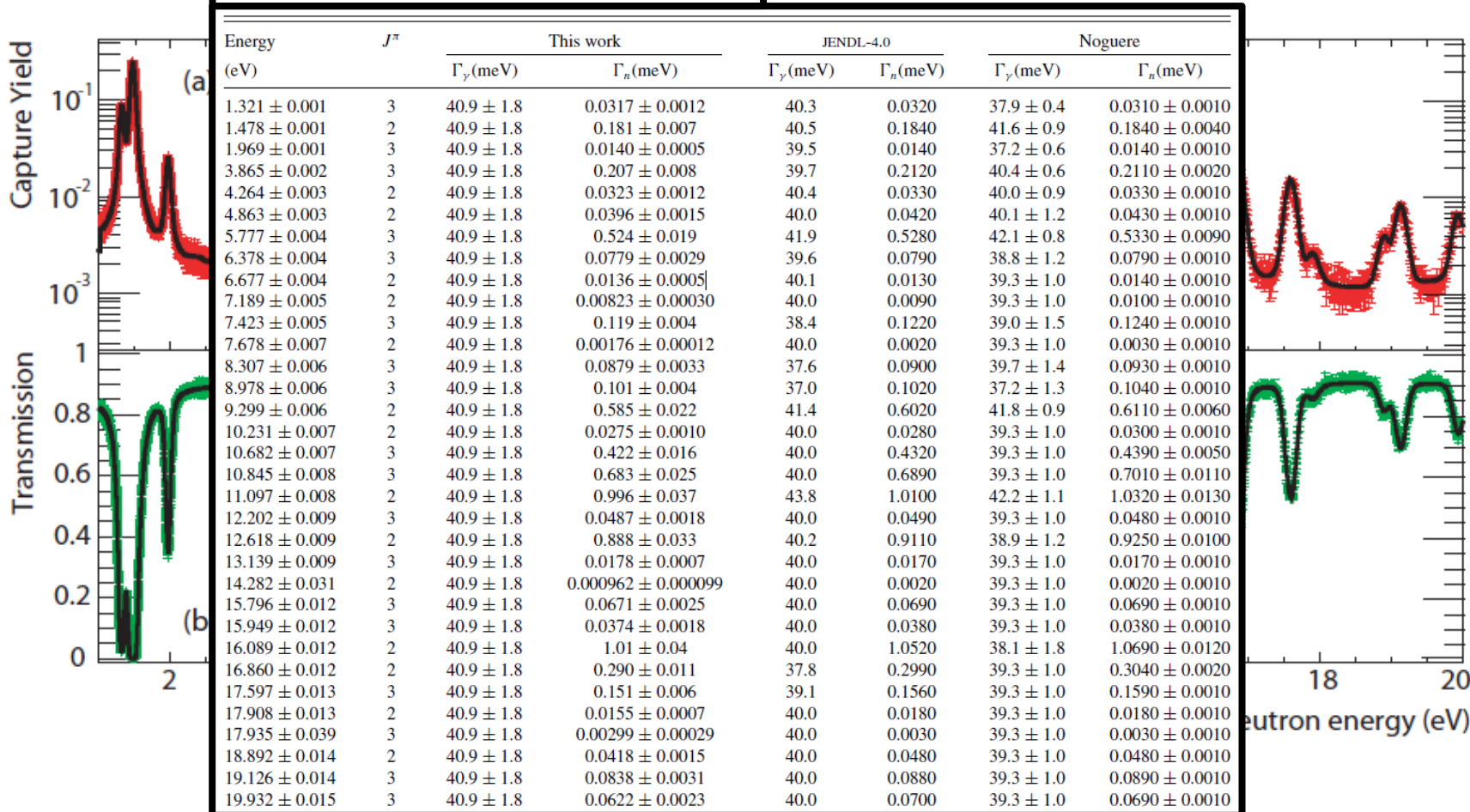


(n,tot)&(n,g) cross sections: resonance analysis

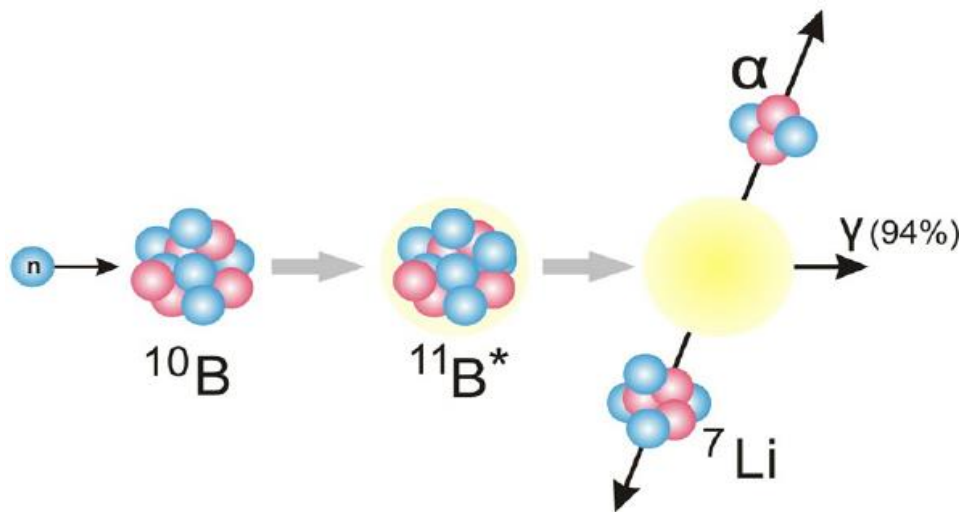
Transmission: $T = \frac{C_{in}}{C_{out}} \propto e^{-n \sigma_{tot}}$

Capture yield: $Y = \frac{C - B}{\epsilon \Phi} = (1 - e^{-n \sigma_{tot}}) \frac{\sigma_{n,g}}{\sigma_{tot}}$

Resonance parameters from RSA with SAMMY



Measuring light charged particle production (n,chp)

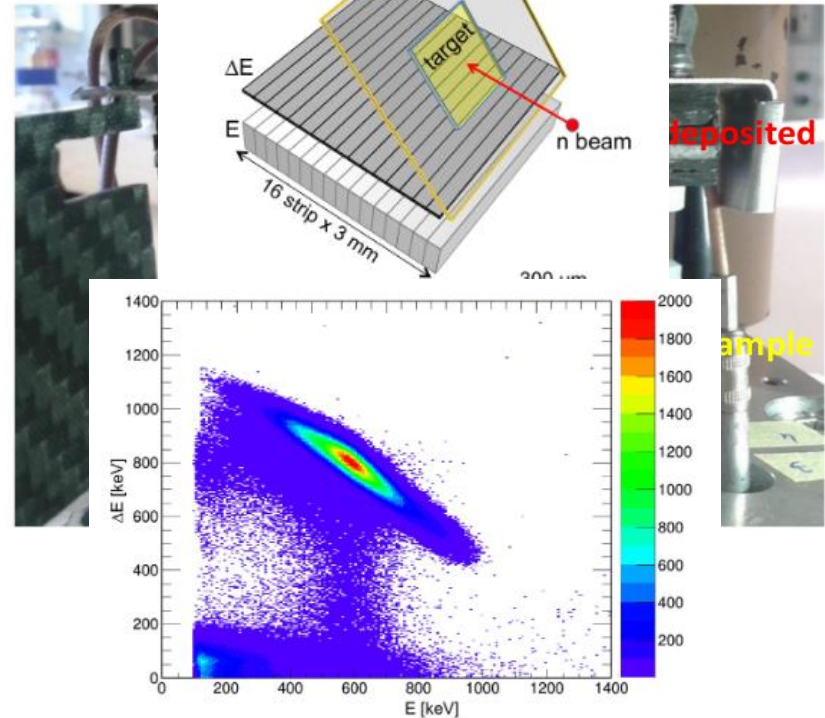
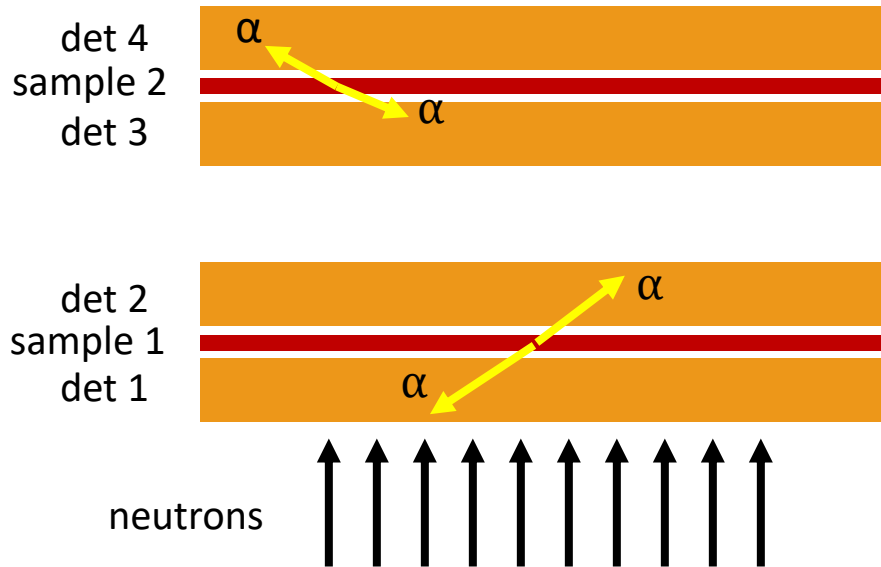


(n, α /p) with high radioactive (GBq) targets

$^7\text{Be}(n,\alpha)/(n,p)$ at CERN n_TOF => Unambiguous identification!

Discriminate background $^7\text{Be} \gamma$ and $^7\text{Be}(n,\alpha)$

Discriminate background $^7\text{Be} \gamma$ and $^7\text{Be}(n,p)$



Silicon detectors in the neutron beam
 3x3 cm² active area, 140 μm thickness
 2 ^7Be targets with ~ 18 GBq each (~ 1.4 μg)

Silicon detectors OFF the neutron beam
 3x3 cm² active area, 20 and 140 μm thickness
 ^7Be target with ~ 1 GBq each (~ 0.1 μg)

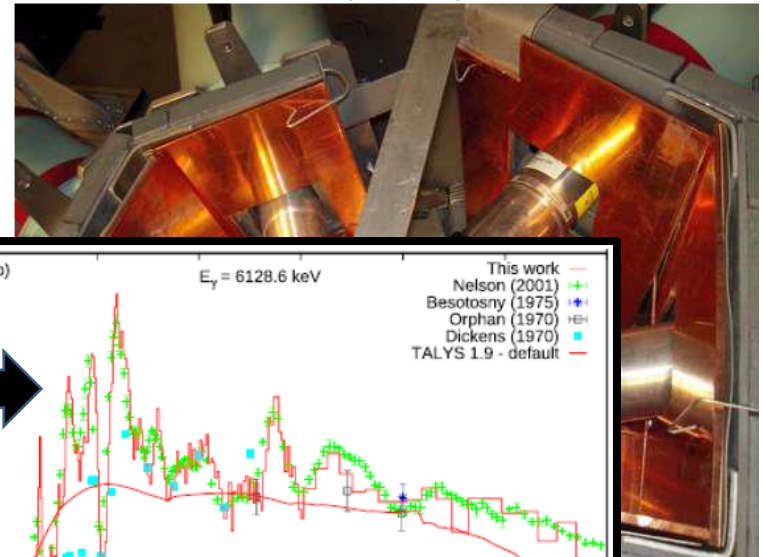
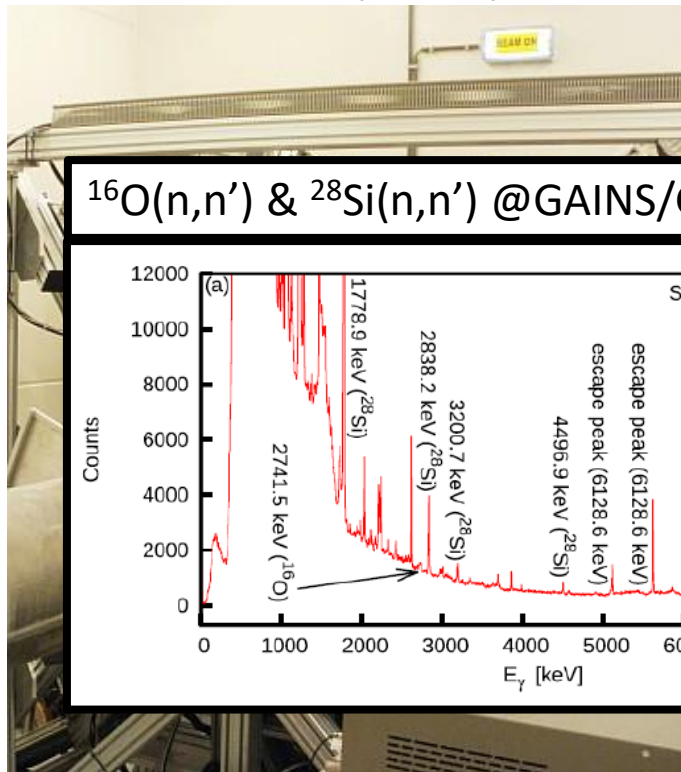
Measuring inelastic scattering (n,xn)

Detectors for inelastic reactions (I)

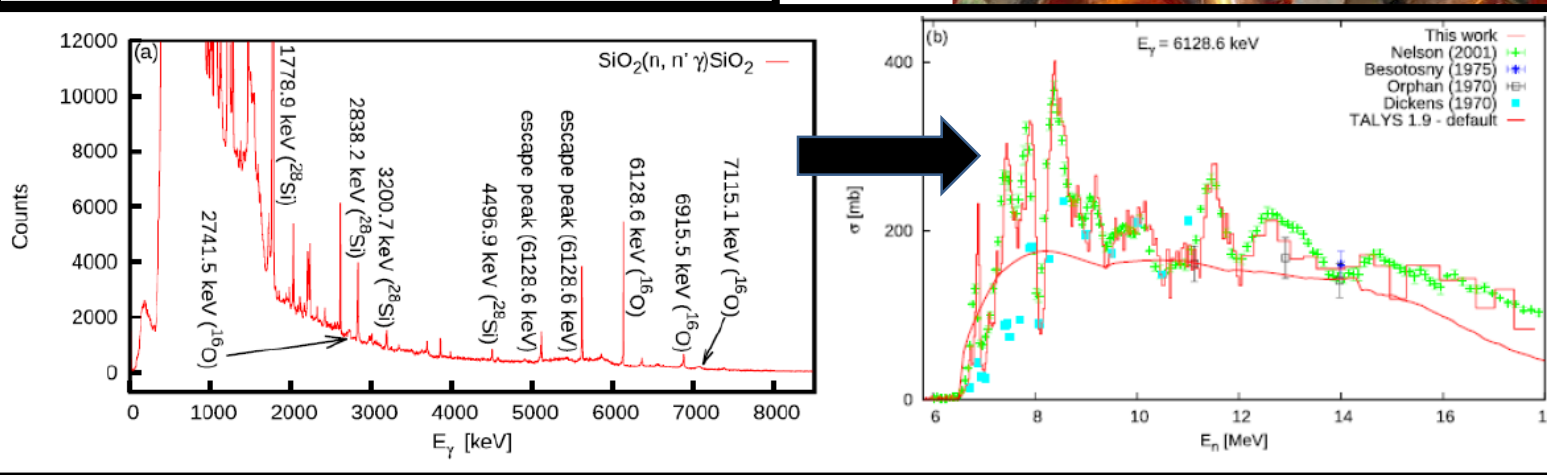
The γ -rays from (n,n') reactions are measured by HPGe detectors

GAINS (Gamma Array for Inelastic Neutron Scattering)
@ IRMM GELINA (100 m)

GRAPhEME (Germanium array for actinides precise measurements)
@ IRMM GELINA (30 m)

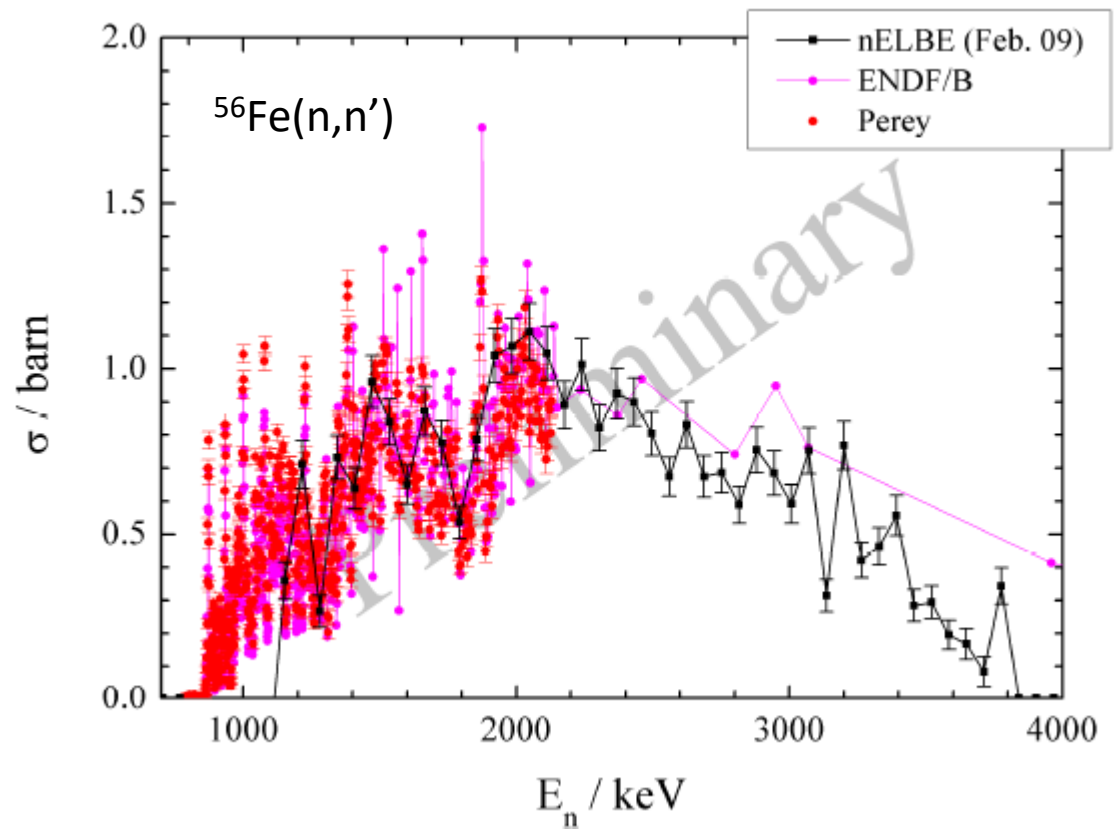
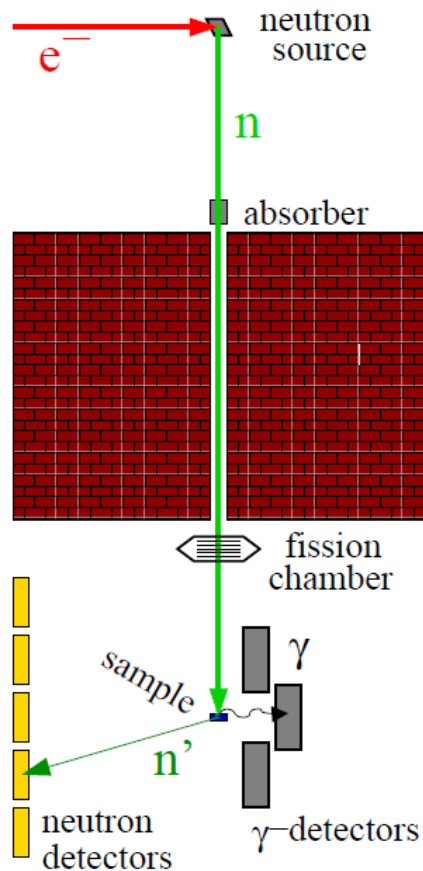


$^{16}\text{O}(n,n')$ & $^{28}\text{Si}(n,n')$ @GAINS/GELINA

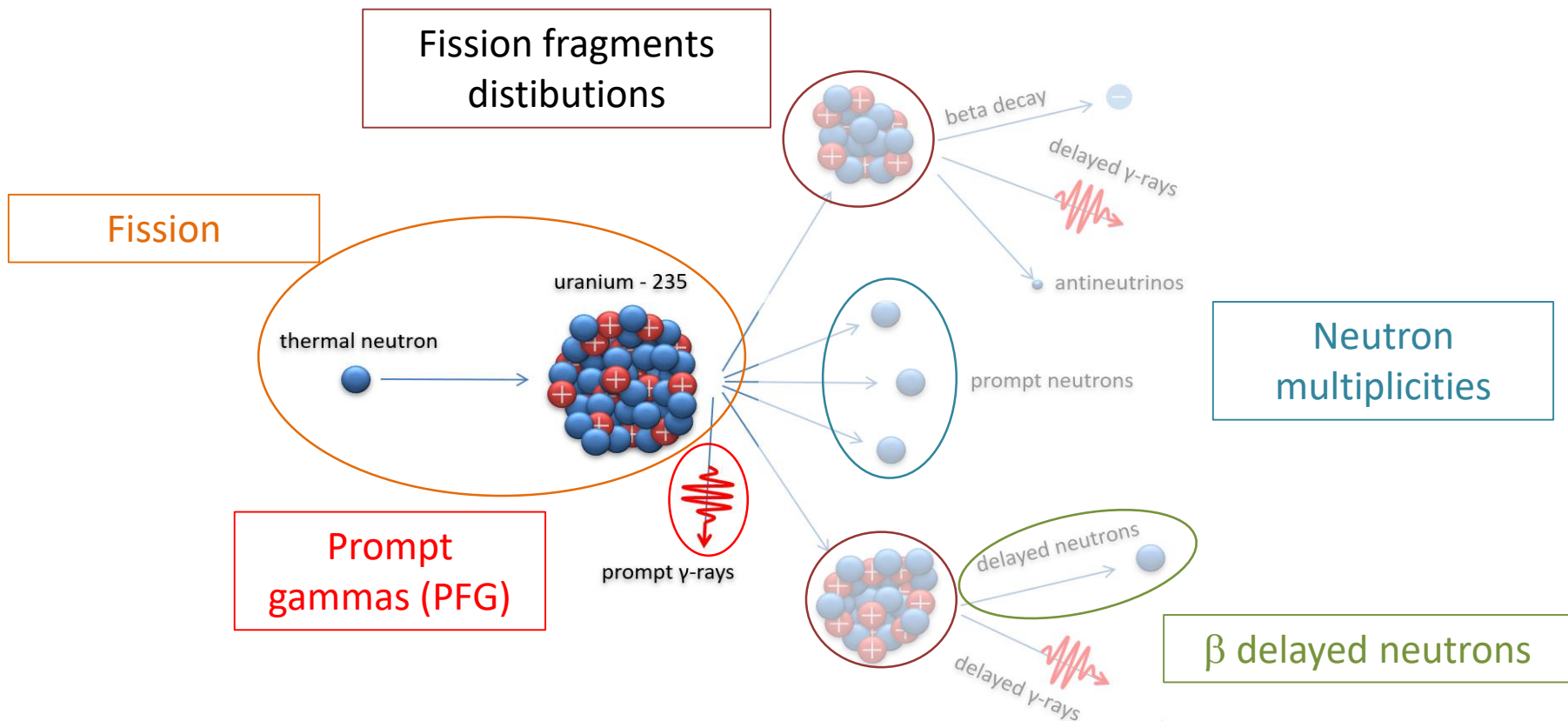


Detectors for inelastic reactions (II)

At nELBE, the HPGe are replaced by 16 BaF2 (lower resolution but faster and with higher efficiency) and 5 neutron detectors (fast plastic scintillators) are added.

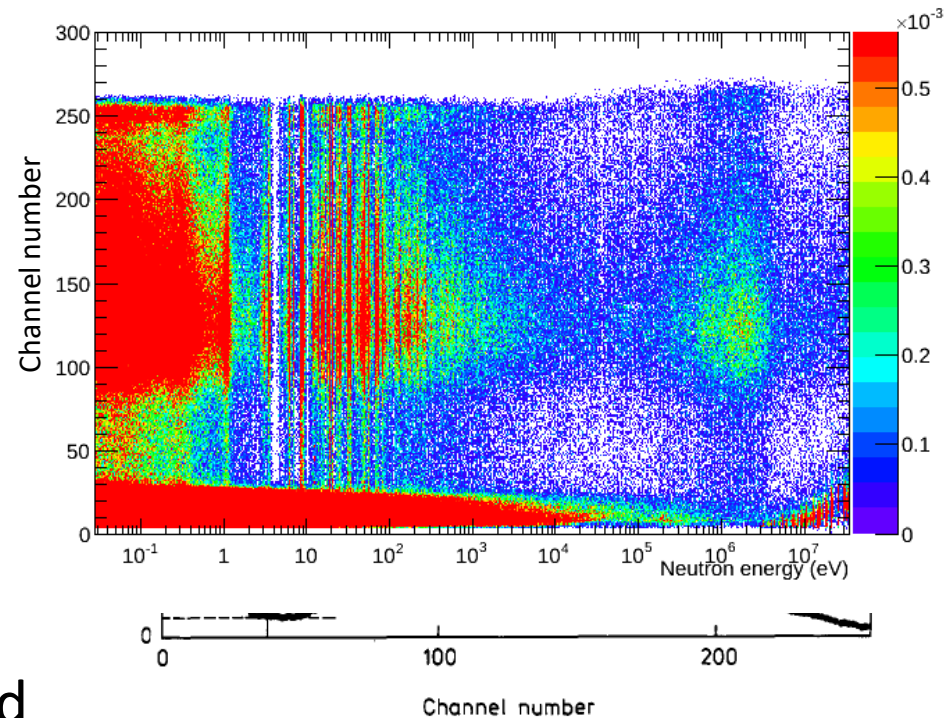
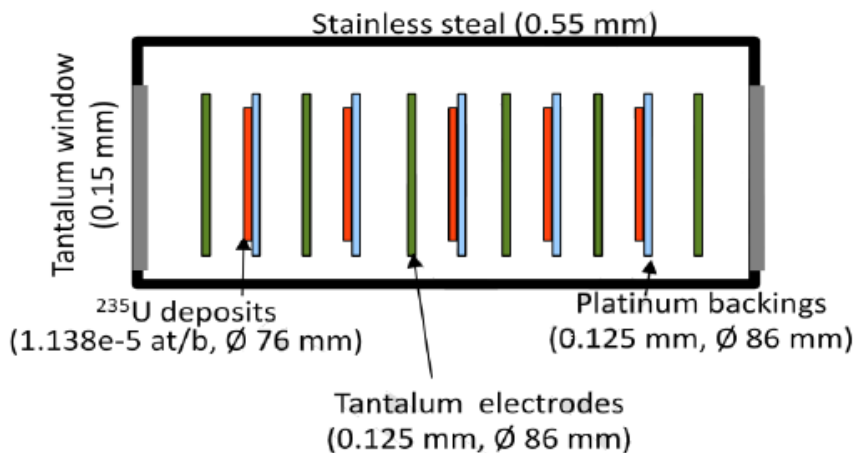


Measuring fission



A “reference” fission chamber (PTB)

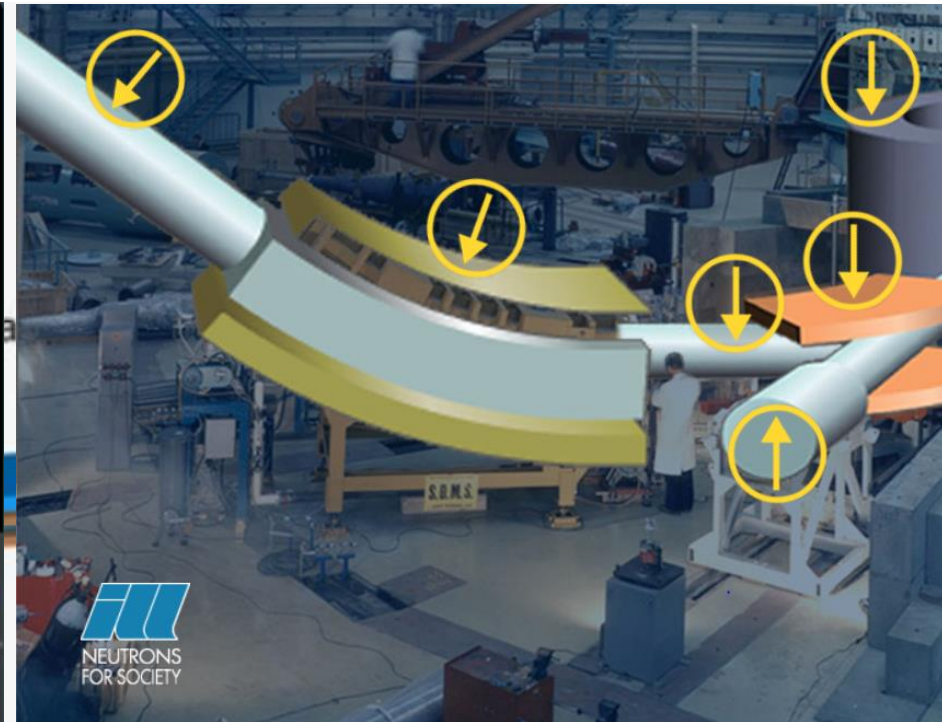
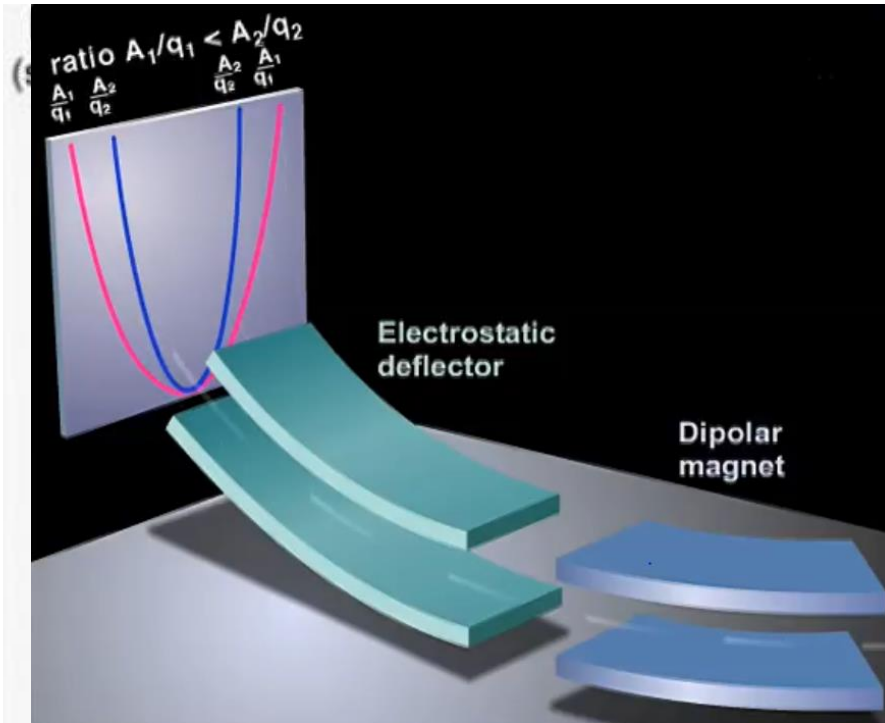
A pair of multiplate fission chambers loaded with ^{235}U and ^{238}U were developed in 1990 and can, still today, be used for flux measurements and intercomparison experiments.



The key:

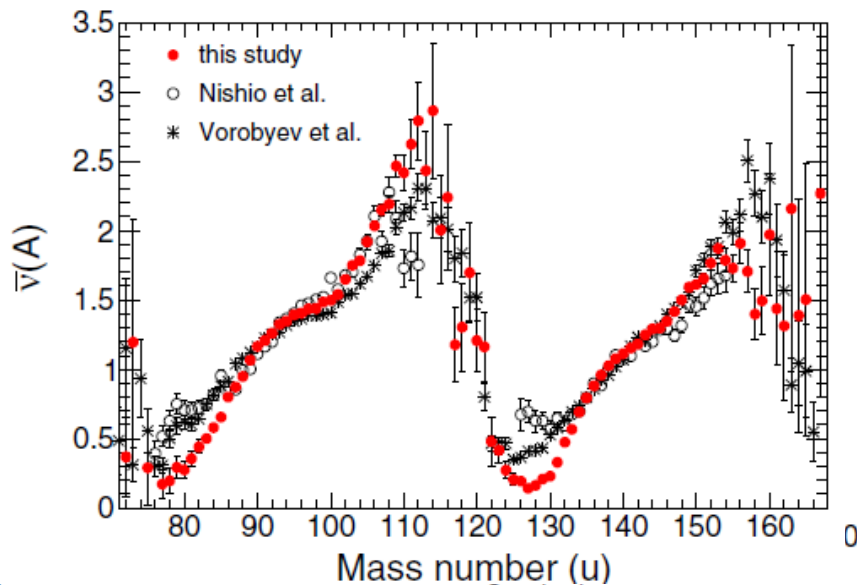
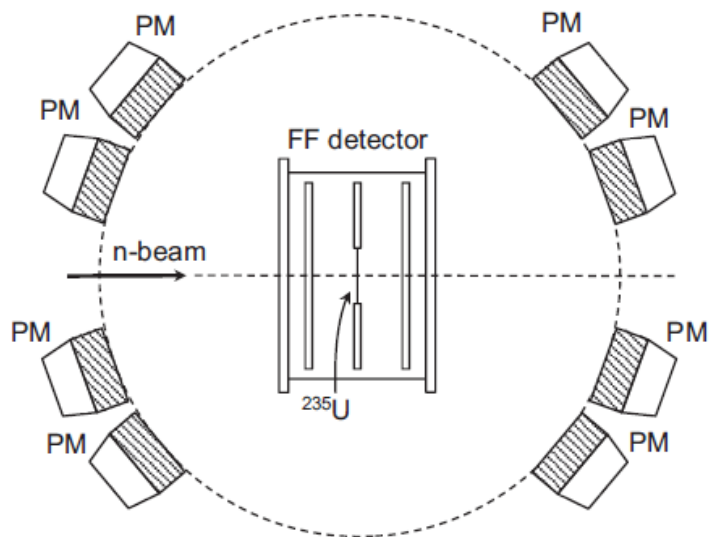
- Samples very well characterized
- Efficiency very well characterized: 98% for $E > 0,45\text{P}$

Fission fragments with a fragment separator

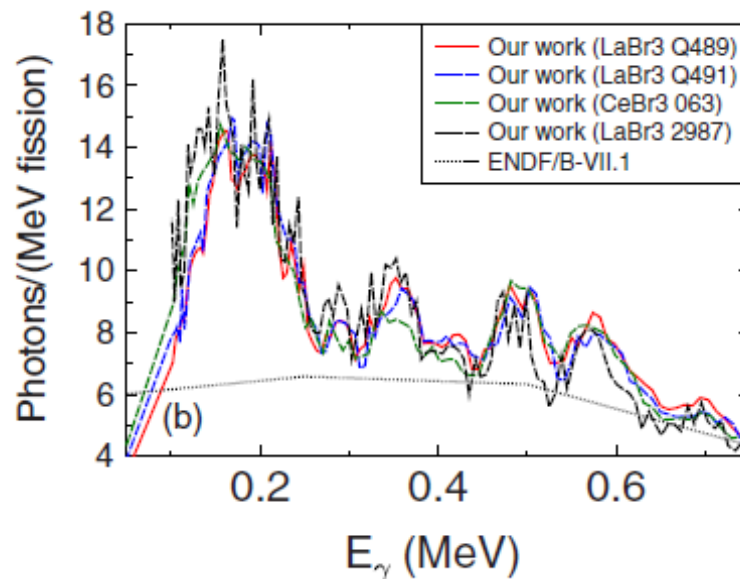
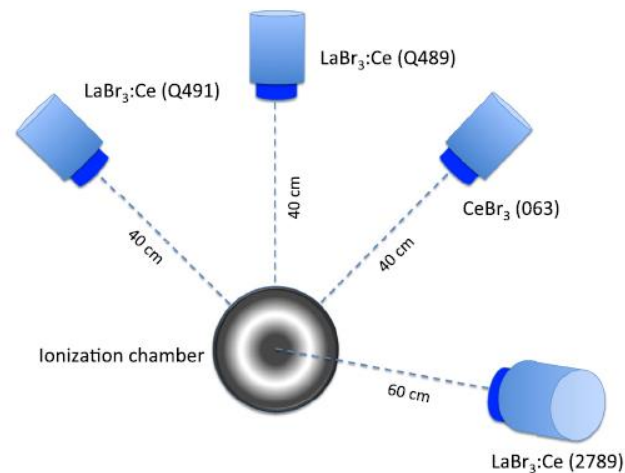


Prompt fission neutrons (PFN) & gammas (PFG)

SCINTIA @GELINA



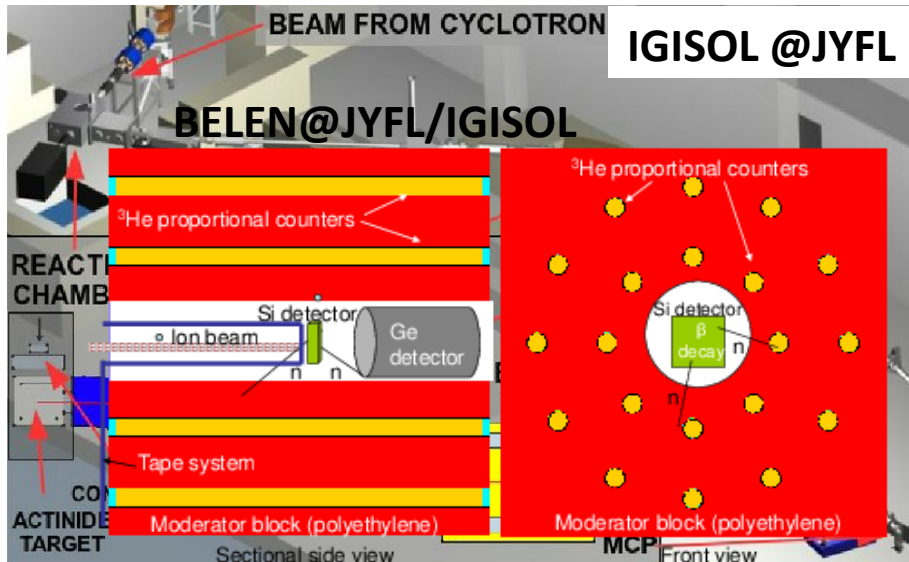
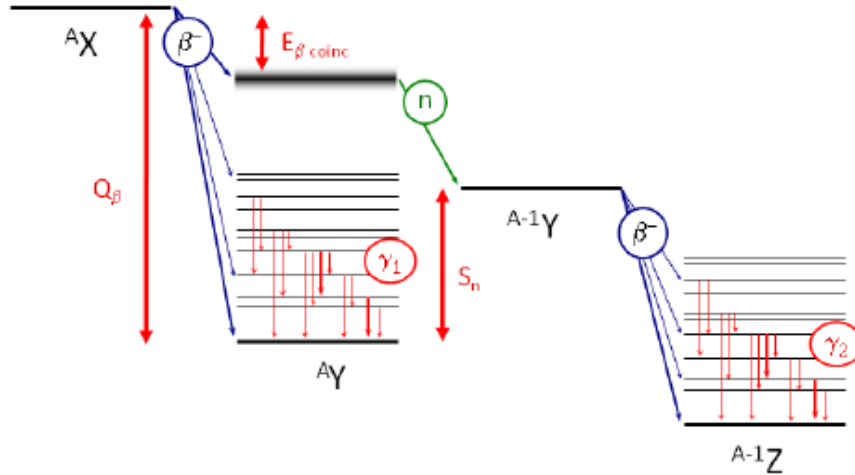
Twin FGIC@BRR



Beta delayed neutron emission

Exotic nuclei at IGISOL (JYFL) => detection beta delayed neutrons

MONSTER@JYFL/IGISOL



“Neutrons for nuclear technology: experiments”

Carlos GUERRERO @H2020-ARIEL HISPANOS School, Sevilla, Spain 22/9/2022

Summary

- Many neutron induced reactions of interest for nucl. Tech.
- Many observables from each reaction
=> Large variety of detection systems and techniques

