
My three-month internship at IBA

— B. Cabouat in collaboration
with F. Stichelbaut —



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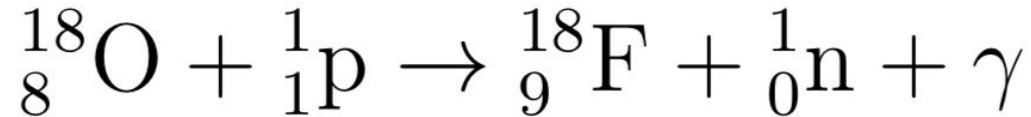


Ion Beam Applications (IBA)

- Around 1500 employees, offices in Belgium, USA, Germany, China and Russia.
- 4 business units:
 - Proton therapy solutions (worldwide leader)
 - Dosimetry solutions
 - RadioPharma solutions (260 cyclotrons produced)
 - Industrial and sterilization solutions (250 electron and proton accelerators)
- Headquarters in Louvain-la-Neuve, Belgium.

My project: activation calculations for the KIUBE

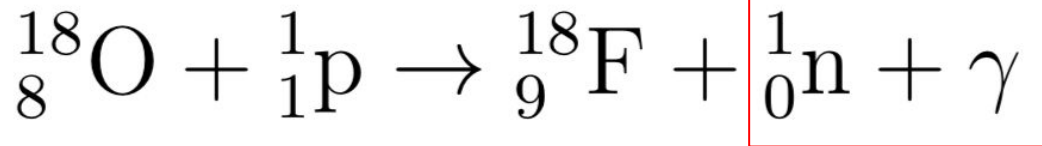
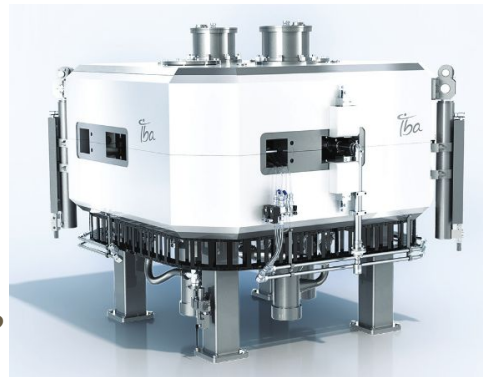
- KIUBE: **18 MeV cyclotron** for the production of **radioisotopes** (Fluorine 18, Oxygen 15, Gallium 68, ...)
- Beam current from 100 μA to 300 μA , with 8 exit ports.
- Ex: production of **fluorine 18**: proton beam on enriched water H_2^{18}O



- Useful for **PET** (positron emission tomography): ${}^{18}_9\text{F} \rightarrow {}^{18}_8\text{O} + {}^0_1\text{e}^+$

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My project: activation calculations for the KIUBE

- Need to **track** the **particles** (neutrons, photons and electrons) passing through the materials of the KIUBE.

➡ **Space** evolution with *MCNPX*

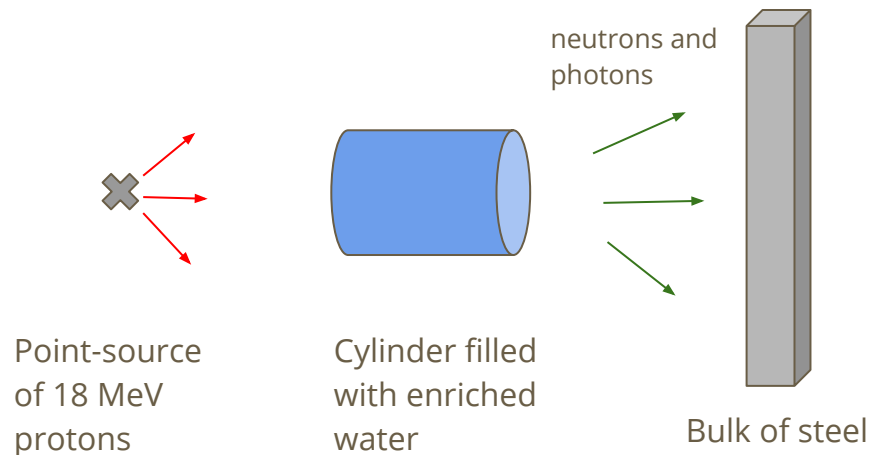
- Need to study the **impact** of those particles on the **materials: activation**.

➡ **Time** evolution with *FISPACT II*

MCNPX

- Monte-Carlo (MC) code that models the **transport of neutral particles** (NP) through a medium. Tracks neutrons and photons, but also electrons, protons, ...
- Created and maintained by Los Alamos National Laboratory.
- A lot of physics included: elastic and inelastic scattering, neutron capture, photon absorption, ...
- Works with input files where one specifies:
 - Geometry: surfaces and cells
 - Materials of the cells: density, isotopic composition, cross-section library
 - Source of primary particles
 - Observables to be calculated (tallies)

MCNPX: an example



MCNPX estimates the F4 tally as:

F4 tally: neutron (or photon) fluence averaged over the volume of the bulk (in 1 / cm²):

$$F4 = \frac{1}{V} \int_V dV \int_E dE \int_{4\pi} d\Omega \Phi(\mathbf{r}, E, \boldsymbol{\Omega})$$

Energy and angular distribution of the fluence as a function of the position

$$F4 = \frac{1}{N} \sum_{i=1}^N \frac{W_i T_i}{V}$$

Number of events

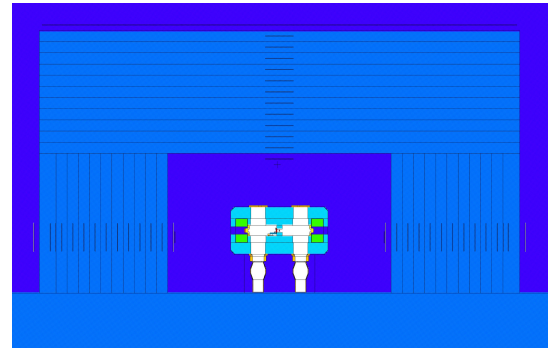
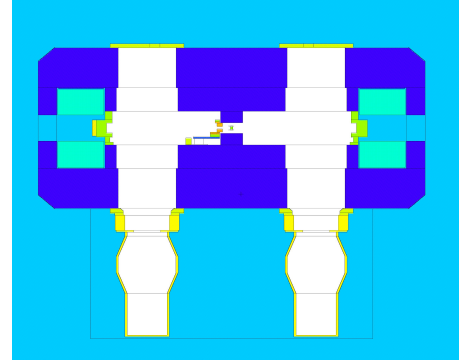
Volume of the bulk

Track-length of particle i through the bulk

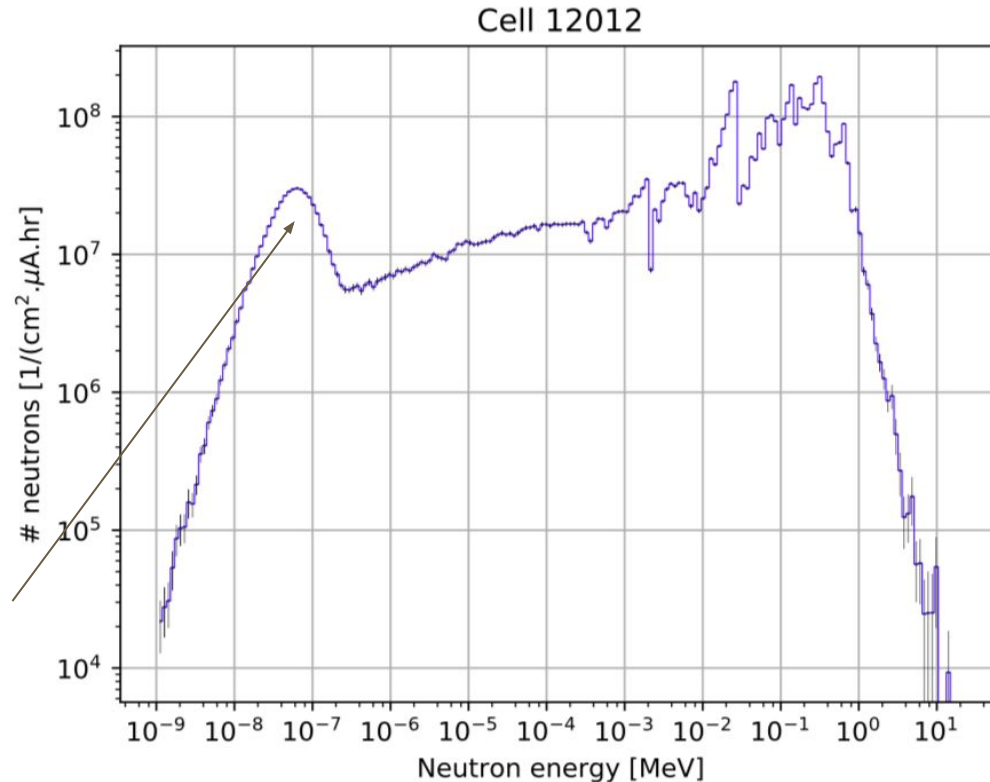
Weight of particle i (used for variance reduction)

Neutron transport through the cells of the KIUBE

- The geometry of the KIUBE was implemented inside MCNPX and divided into **235 cells**.
- Use a source of **18 MeV protons** on **enriched water**.
- We studied the **transport of the neutrons** through all the cells of the KIUBE. Use F4 tally on each cell (with energy bins) to get the **neutron fluence**.



Neutron transport through the cells of the KIUBE



Thermal peak:

neutrons lost some kinetic energy by recoiling against the atoms of the medium.

Cell 12012: cavity of the radio-frequency system of the cyclotron. Made of copper with density 8.96 g/cm³.

FISPACT II

- Deterministic code that solves numerically problems of **activation**.
- The output is a **time-dependent inventory** of all the **radioisotopes** present in the material.
- Works with input files where one specifies:
 - Material: density, isotopic composition, cross-section library
 - Source of incident particles
 - Irradiation and cooling times.
- **No geometry** involved: the material is assumed to be homogeneous and infinite.

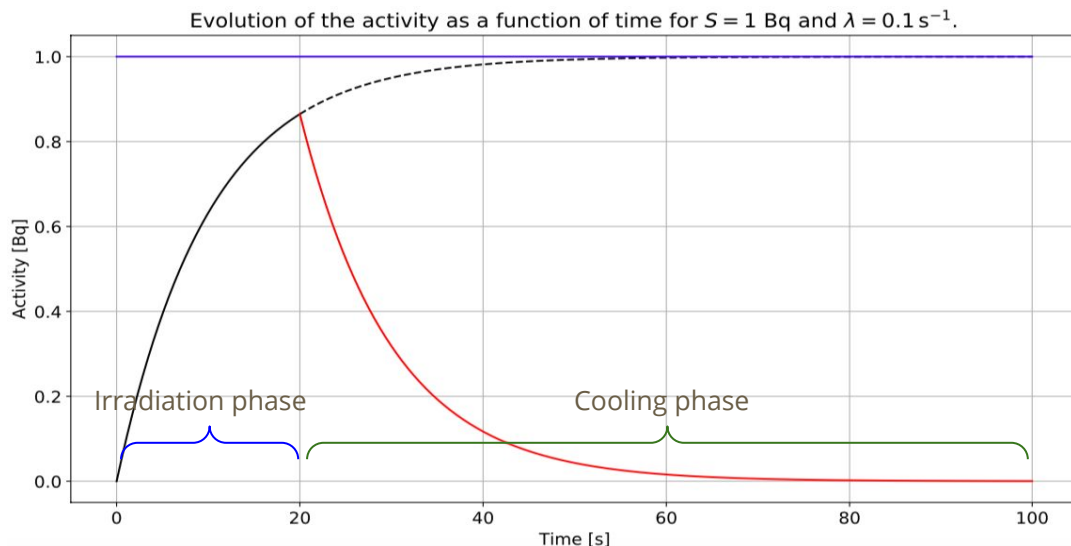
FISPACT II

Number of a given isotope in a material follows:

$$\frac{dN}{dt} = -\lambda N(t) + S$$

↑
Decay rate
↑
Source term
(activation rate)

- **Irradiation** phase: $S \neq 0$ $A(t) = S (1 - e^{-\lambda t})$
- **Cooling** phase: $S = 0$ $A(t) = S (1 - e^{-\lambda t_{\text{irr}}}) e^{-\lambda(t-t_{\text{irr}})}$



Activity: number of decays per unit time

$$A(t) = \lambda N(t)$$

FISPACT II

➤ In fact:

$$S_i(t) = \phi \sum_{j \neq i} \sigma_i^j N_j(t)$$

Flux of incident
particles

Cross-section for
reactions on j
producing i

➤ System of nested equations:

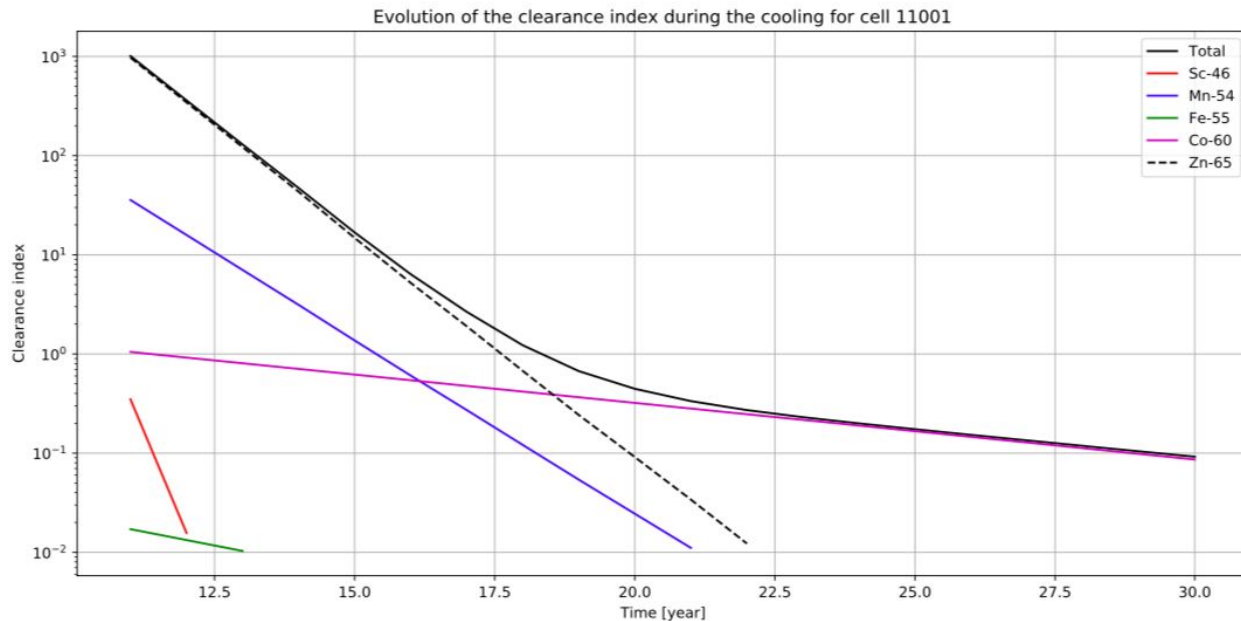
$$\frac{d}{dt} \mathbf{N} = \mathcal{M} \mathbf{N}(t)$$

Isotopic vector

Matrix with decay and
activation rates

Activation of the KIUBE

- The cells of the KIUBE are **irradiated by neutrons**. Use the energy spectra from MCNPX as energy distributions for the sources.
- **Isotopic vector** calculated for each cell of the KIUBE.



Clearance index:

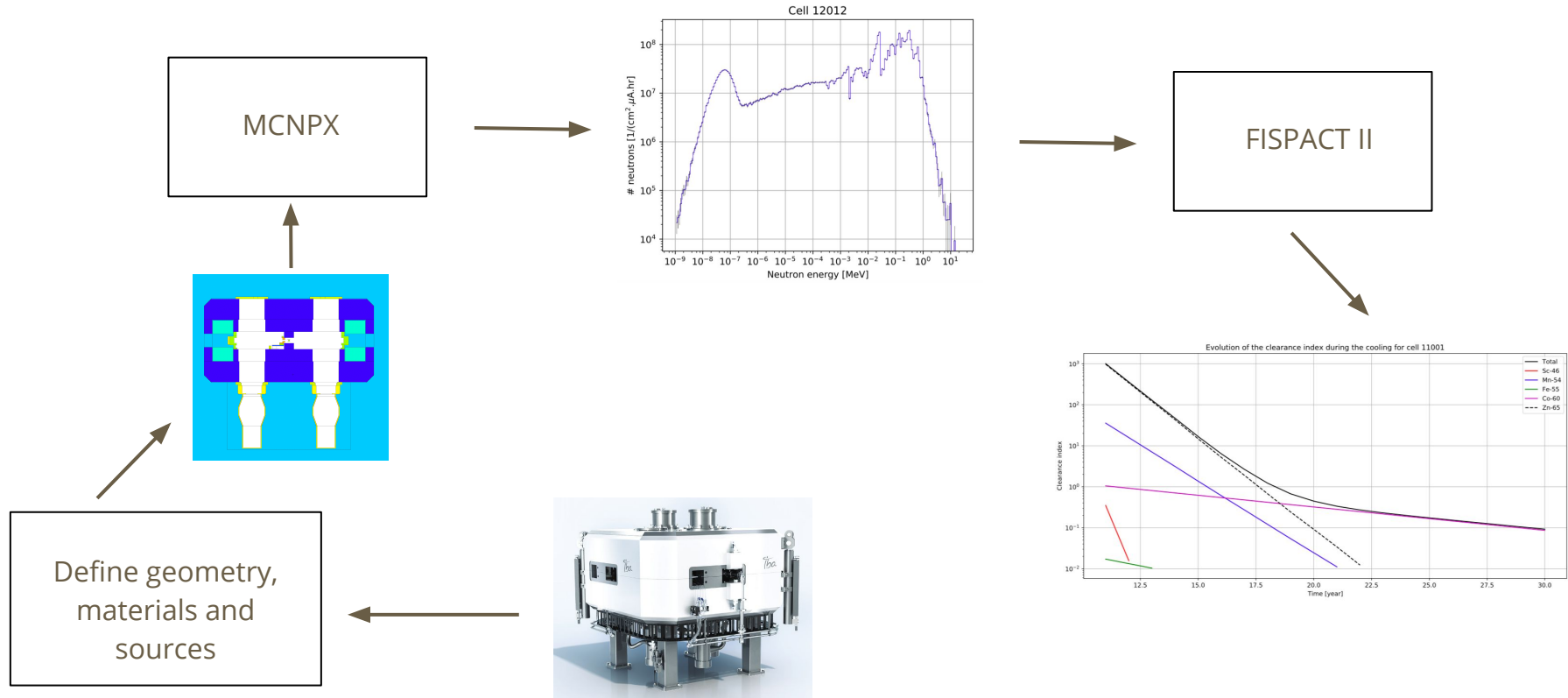
$$CI_i(t) = \frac{A_i(t)}{A_i^{\text{ref}}}$$

Safety standards:

$$CI_i < 1$$

→ isotope i can be considered as **non-radioactive**

Summary¹: activation calculations



¹This work has received funding from the European Union's Horizon 2020 research and innovation programme as part of the Marie Skłodowska-Curie Innovative Training Network MCnetITN3 (grant agreement no. 722104).

My experience: it was interesting to

- Work in a company (different atmosphere, pace, expectations).
- Apply my theoretical knowledge of particle physics to an industrial case.
- Learn new methods of variance reduction and how to better assess errors (cf. MCNP primer), even though I was already familiar with Monte-Carlo methods.
- Set the safety standards as a priority in order to validate my results.
- Deal with different softwares and interface them.

References

- J. K. Shultis and R. E. Faw, An MCNP Primer.
- M. Fleming, T. Stainer, M. Gilbert, The Fispack-II User Manual, 2018.